

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR, Chapter 1: (1) Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70, (2) is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and (3) is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 1772 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

The licensee shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the licensee's Fire Plan, and as referenced in the Updated Safety Analysis Report, and as approved in the Safety Evaluation Reports, dated November 25, 1977, and December 12, 1978 (and supplement dated February 13, 1981) subject to the following provision:

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission, only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

(4) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Nuclear Management Company Kewaunee Nuclear Power Plant Physical Security Plan (Revision 0)" submitted by letter dated October 18, as supplemented by letter dated October 21, 2004.

(5) Fuel Burnup

The maximum rod average burnup for any rod shall be limited to 60 GWD/MTU until completion of an NRC environmental assessment supporting an increased limit.

(6) Steam Generator Upper Lateral Supports

The design of the steam generator upper lateral supports may be modified by reducing the number of snubbers from four (4) to one (1) per steam generator.

(7) Deleted

(8) Operator Actions

The auxiliary feedwater system local manual operator actions as described in the License Amendment Request submitted May 5, 2005, and supplemented on June 9, 2005, shall be eliminated no later than completion of Kewaunee refueling outage R-29.

(9) Surveillance Test Interval Relaxation

In lieu of the specified frequencies, Dominion Energy Kewaunee, Inc. may complete the surveillance requirements noted in Table 2.C.(9) on page 4a during the fall 2006 refueling outage, but not later than October 7, 2006.

- D. The licensee shall comply with applicable effluent limitations and other limitations and monitoring requirements, if any, specified pursuant to Section 401(d) of the Federal Water Pollution Control Act Amendments of 1972.
- E. This license is effective as of the date of issuance, and shall expire at midnight on December 21, 2013.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachment:

Appendices A and B - Technical Specifications

Date of Issuance: December 21, 1973

Table 2.C.(9)

Surveillance Requirement	Table Item Number	Title
Table 4.1-1	5	Reactor Coolant Flow - Calibration
Table 4.1-1	6	Pressurizer Water Level - Calibration
Table 4.1-1	7	Pressurizer Pressure - Calibration
Table 4.1-1	11a	Steam Generator Low Level - Calibration
Table 4.1-1	11b	Steam Generator High Level - Calibration
Table 4.1-1	21	Containment Sump Level - Test
Table 4.1-1	30	Fore Bay Water Level - Test
Table 4.1-1	33	PORV Block Valve Position Indicator - Calibration
Table 4.1-1	36	Reactor Coolant System Subcooling Monitor – Calibration and Test
Table 4.1-1	42	Steam Generator Level (Wide Range) - Calibration
Table 4.1-3	4	Containment Isolation Trip - Test
4.4.c.1.b		Shield Building Ventilation System Tests
4.5.a.1		Safety Injection System Tests
4.5.a.2		Containment Vessel Internal Spray System
4.5.a.3		Containment Fancoil Units Tests
4.5.b.2.F		Residual Heat Removal System valve interlocks
4.6.a.2		Automatic start of each diesel generator, load shedding, and restoration to operation of particular vital equipment
4.6.a.3		Diesel Generator Inspection
4.6.a.4		Diesel Generator Load Rejection Test
4.14		Testing And Surveillance Of Shock Suppressors (Snubbers)
4.17.a.2		Control Room Postaccident Recirculation System
6.12.b		System Integrity Program Integrated Leak Tests