

UNITED STATES NUCLEAR REGULATORY COMMISSION

REGION II SAM NUNN ATLANTA FEDERAL CENTER 61 FORSYTH STREET, SW, SUITE 23T85 ATLANTA, GEORGIA 30303-8931

July 7, 2006

Mr. Russell B. Starkey, Jr.
Vice President - Operations
United States Enrichment Corporation
Two Democracy Center
6903 Rockledge Drive
Bethesda, MD 20817

SUBJECT: NRC INSPECTION REPORT NO. 70-7001/2006-003

Dear Mr. Starkey:

This refers to the inspection conducted from April 20, through June 14, 2006, at the Paducah Gaseous Diffusion Plant. The purpose of the inspection was to determine whether activities authorized by the certificate were conducted safely and in accordance with NRC requirements. At the conclusion of the inspection on June 15, 2006, the NRC senior resident inspector discussed the findings with members of your staff.

As a result of the inspection, the enclosed NRC Forms 591FF, Safety Inspection Report, are being issued. The enclosed forms indicate that no violations were identified during the inspection period.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be made available electronically for public inspection in the NRC Public Document Room or from the NRC's document system (ADAMS), accessible from the NRC Web site at http://www.nrc.gov/reading-rm/adams.html.

Should you have any questions concerning this letter, please contact us.

Sincerely,

/RA/

Jay L. Henson, Chief Fuel Facility Inspection Branch 2 Division of Fuel Facility Inspection

Docket No. 70-7001 Certificate No. GDP-1

Enclosures: (See page 2)

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Enclosures: 1. NRC Form 591FF Part 1

2. NRC Form 591FF Part 3

cc w/encls:

S. Penrod, Paducah General Manager

S. R. Cowne, Paducah Regulatory Affairs Manager

W. Jordan, Portsmouth General Manager

S. A. Toelle, Director, Nuclear Regulatory Affairs, USEC

R. M. DeVault, Regulatory Oversight Manager, DOE

G. A. Newtown, Paducah Site Office, DOE

Dewey Crawford, Department of Public Health, Commonwealth of Kentucky

Distribution w/encls:

D. Martin, NMSS

B. Von Till, NMSS

J. Henson, RII

D. Hartland, RII

M. L. Thomas, RII

M. Chitty, RII

E-MAIL COPY?

YES

NO

X PUBLICLY AVAILABLE		□ NON-PUBLICLY AVAILABLE		□ SENSITIVE		X NON-SENSITIVE	
ADAMS: XYes ACCESSION NUMBER:							
OFFICE	RII:DFFI	RII:DFFI	RII:DFFI	RII:DFFI			
SIGNATURE	DH for 7/6/06	JJ 7/7/06	NR 7/7/06	DH 7/7/06			
NAME	MThomas	JJimenez	NRivera	DHartland			
DATE	7/ /2006	7/ /2006	7/ /2006	7/ /2006	7/ /2006	7/ /2006	7/ /2006

YES

YES

NO

YES

NO

YES

NO

OFFICIAL RECORD COPY DOCUMENT NAME: E:\Filenet\ML061880369.wpd

YES

NRC FORM 591FF PART 1			U.S. NUCLEAR REGULATO	DRY COMMISSION	
(11-2005) 10 CFR 2.201					
	SAFETY INSPEC				
LICENSEE/LOCATION INSPECTED: United States Enrichment Corporation 6903 Rockledge Drive Bethesda, MD 20817	2.		latory Commission of Fuel Facilities Ins	pection	
REPORT NUMBER(S): 2006-003	TA LIGENOSS NUMBER(O)	,	Le DATE(O) OF MODEOTI		
3. DOCKET NUMBER(S): 70-7001	4. LICENSEE NUMBER(S):		5. DATE(S) OF INSPECTION 04/20-06		
LICENSEE:	<u> </u>	1 -1	04/20-00	, 14700	
The inspection was an examination of the activities conducted under your license as they relate to radiation safety and to compliance with the Nuclear Regulatory Commission (NRC) rules and regulations and the conditions of your license. The inspection consisted of selective examinations of procedures and representative records, interviews with personnel, and observations by the inspection findings are as follows: X					
I hereby state that, within 30 days, the actions des corrective actions is made in accordance with the	requirements of 10 CFR 2.201	vill be taken to correct the vio	olations identified. This stater lken, corrective steps which w	vill be taken,	
date when full compliance will be achieved). I und Title	derstand that no further written re Printed Name		quired, unless specifically requ qnature	uested. Date	
	Not applicable	319	gracio	Date	

D. Hartland for

NRC FORM 591FF PART 1 (11-2005)

Mary L. Thomas

NRC INSPECTOR

7/7/06

NRC FORM 591FF PART 3

(11-2005) 10 CFR 2.201

U.S. NUCLEAR REGULATORY COMMISSION

SAFETY INSPECTION REPORT AND COMPLIANCE INSPECTION

1. LICENSEE

2. NRC/REGIONAL OFFICE

United States Enrichment Corporation 6903 Rockledge Drive Bethesda, MD 20817 U.S. Nuclear Regulatory Commission Region II, Division of Fuel Facilities Inspection 61 Forsyth Street, Suite 23T85 Atlanta, GA 30303

REPORT NUMBER(S):

2006-003

70-7001	GDP-1	04/20-06/14/06
3. DOCKET NUMBER(S):	4. CERTIFICATE NUMBER(S):	5. DATE(S) OF INSPECTION:

6. INSPECTOR(S):

M. L. Thomas, J. G. Jimenez, N. Rivera

7. INSPECTION PROCEDURES USED: 88010, 88025, 88100, 88102, 88103, 92700, TI 2600/012

SUPPLEMENTAL INSPECTION INFORMATION

Executive Summary

Summary of Plant Status

• The certificatee performed routine operations throughout the inspection period. Plant load was decreased to 900 megawatts and product assay was decreased to 2.0 percent in accordance with the production schedule.

Plant Operations

- The inspectors observed routine operations in the cascade buildings and area control rooms, the feed vaporization facilities, product and tails withdrawal facilities, and the central control facility. The operations staff were alert and generally knowledgeable of the current status of equipment associated with their assigned facilities.
- The inspectors identified an unresolved item involving the opening of a high assay boundary valve in the C-331 Process Building to evacuate uranium hexafluoride from the Number 2 Hortonsphere in the C-315, Tails Withdrawal Building, without the implementation of the required procedural controls. (URI 70-7001/2006-003-01)

Maintenance and Surveillance

During the observation of maintenance and surveillance activities, the inspectors verified one or more of the
following: activities observed were performed in a safe manner; testing was performed in accordance with
procedures; measuring and test equipment was within calibration; Technical Safety Requirement (TSR) Limiting
Conditions for Operations were entered, when appropriate; removal and restoration of the affected components
were properly accomplished; test and acceptance criteria were clear and conformed with the TSR and the Safety
Analysis Report; and any deficiencies or out-of-tolerance values identified during the testing were documented,
reviewed, and resolved by appropriate management personnel.

NRC FORM 591FF PART 3 (11-2005) Enclosure 2

(11-2005) 10 CFR 2.201

Executive Summary (continued)

Operator Training/Retraining

- The training program for initial and refresher training in nuclear criticality safety, radiation protection and general
 emergency areas was in compliance with regulatory requirements. Training material and examinations were
 adequate to measure the knowledge level of the workers and were current. Lessons learned from past facility
 events were appropriately communicated in morning meetings to enhance worker safety.
- Operating procedure and facility change control training was effective. Operators were knowledgeable of their
 operating processes and pending changes. Changes to nuclear material processing requirements were readily
 reviewed by the operators who had to acknowledge their understanding before implementing changes.
- Each new operator's on-the-job training was tracked by his/her immediate supervisor or front line manager using a training requirement matrix. The matrices were maintained by the supervisor and reviewed weekly with each operator.

<u>Temporary Instruction 2600/012</u>: Institutionalizing Concern Regarding Safety Issues Identified in selected Past Generic Communications

• Information Notice (IN)-99-05, "Inadvertent Discharge of Carbon Dioxide Fire Protection System and Gas Migration": The certificatee had two carbon dioxide fire protection systems, one located in the basement of the C-533 switchhouse and the other in the C-101 cafeteria. Both were non-safety related and alarmed locally as well as in the Central Control Facility and fire station. Both were inspected on an annual basis in accordance with plant procedures. Neither system would have created a white-out condition or reduced the oxygen content in the area below 19.5 percent. This IN is considered closed

Items Opened, Closed, and Discussed

Item Number	Status	Description

URI 70-7001/2006-003-01 Open Operation of a high assay boundary valve in C-331 without the

implementation of required procedural controls.