

JUN 28 2006

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LCR S05-08



United States Nuclear Regulatory Commission
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SUPPLEMENT TO LCR S05-08: REQUEST FOR CHANGE TO TECHNICAL SPECIFICATIONS, CONTAINMENT ISOLATION VALVES SALEM GENERATING STATION, UNIT NOS. 1 AND 2 FACILITY OPERATING LICENSES DPR-70 AND DPR-75 DOCKET NOS. 50-272 AND 50-311

In accordance with the requirements of 10 CFR 50.90, PSEG Nuclear LLC (PSEG) previously submitted, on September 26, 2005, License Change Request (LCR) S05-08 requesting an amendment of the Technical Specifications (TS) for Salem Generating Station, Unit Nos. 1 and 2. This submittal supplements the September 26, 2005 submittal, providing clarifying information based on discussions with the NRC Staff. Pursuant to the requirements of 10 CFR 50.91(b)(1), a copy of this request for amendment supplement has been sent to the State of New Jersey.

LCR S05-08 proposed eliminating the Surveillance Requirement (SR) for containment isolation valves (Salem Unit 1 SR 4.6.3.1.1 and Salem Unit 2 SR 4.6.3.1) that are being returned to service after maintenance, repair or replacement work to be consistent with the requirements in NUREG-1431, "Standard Technical Specifications, Westinghouse Plants." This proposed change will minimize unnecessary testing and plant transients while continuing to assure that the necessary quality of systems and components is maintained.

In Section 3, "Background" of Attachment 1 to LCR S05-08, a discussion was provided on post-maintenance testing after valve stem packing adjustments and backseating of valves. The purpose of this discussion was to provide one example of maintenance on a valve that would not adversely affect valve performance parameters, and would therefore not need to be subjected to the SR requirements; post-maintenance testing would ensure that the equipment meets all applicable SRs before restoring the equipment to operable status. It was further stated in Section 3 that:

"The effect of Salem Unit 1 SR 4.6.3.1.1 and Salem Unit 2 SR 4.6.3.1 is to require valve stroke testing after repairs, such as valve packing adjustments, even when it can be demonstrated that the repairs would not adversely affect valve performance parameters."

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JUN 28 2006

The discussion of valve packing adjustments was not meant to be exclusive; it was meant as an example of any maintenance, repair, or replacement work that is performed on the valve or its associated actuator, control, or power circuit that would not adversely affect valve performance parameters. The deletion of the SR requirement will provide flexibility in determining the appropriate post-maintenance test based on the work performed, and that when appropriate, a timed valve stroke test will be performed to ensure component operability following maintenance activities.

This change is consistent with Westinghouse Standard Technical Specifications (STS); STS SR 3.6.3.5, verification of isolation time of containment isolation valves, is strictly a periodic surveillance, there is no reference to the conduct of maintenance. . A similar TS amendment was provided to Millstone Unit 3 via Amendment 200 dated December 21, 2001.

If you have any questions concerning this request, please contact Mr. Jamie Mallon at (610) 765-5507.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on

6/28/06
(Date)

Sincerely,



Thomas P. Joyce
Site Vice President
Salem Generating Station

JUN 28 2006

C Mr. S. Collins, Administrator – Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

U. S. Nuclear Regulatory Commission
Mr. S. Bailey, Project Manager - Salem Unit 1 and Unit 2
Mail Stop 08B1
Washington, DC 20555-0001

USNRC Senior Resident Inspector – Salem Unit 1 and Unit 2 (X24)

Mr. K. Tosch, Manager IV
Bureau of Nuclear Engineering
PO Box 415
Trenton, New Jersey 08625