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June 29, 2006

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: Licensee Event Report 50-458 / 06-005-00
River Bend Station – Unit 1
Docket No. 50-458
License No. NPF-47

File Nos. G9.5, G9.25.1.3

RBG-46582
RBF1-06-0105

Ladies and Gentlemen:

In accordance with 10CFR50.73, enclosed is the subject Licensee Event Report.
This document contains no commitments.

Sincerely,

A handwritten signature in black ink, appearing to read "David N. Lorfing".

David N. Lorfing
Manager – Licensing

DNL/dhw
Enclosure

IE22

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cc: U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011

NRC Sr. Resident Inspector
P. O. Box 1050
St. Francisville, LA 70775

INPO Records Center
E-Mail

Mr. Jim Calloway
Public Utility Commission of Texas
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Austin, TX 78711-3326

Mr. Jeff Meyers
Louisiana Department of Environmental Quality
Office of Environmental Compliance
P.O. Box 4312
Baton Rouge, LA 70821-4312

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME River Bend Station – Unit 1	2. DOCKET NUMBER 05000-458	3. PAGE 1 of 3
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4. TITLE
Automatic Start of Standby Service Water During Realignment of Reactor Plant Cooling Water

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV. NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	01	2006	2006	- 005 -	00	06	29	2006		05000
									FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE
5

10. POWER LEVEL
0

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: (Check all that apply)

<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)
<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	

Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME David N. Lorfing, Manager – Licensing	TELEPHONE NUMBER (Include Area Code) 225-381-4157
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
X	CC	v	Velan	Y					

14. SUPPLEMENTAL REPORT EXPECTED
 YES (If yes, complete 15. EXPECTED SUBMISSION DATE) NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On May 1, 2006, at approximately 6:59 a.m., an unplanned automatic actuation of the Division 1 standby service water system occurred during a realignment of the reactor plant component cooling water system. The plant was in cold shutdown for a refueling outage at the time. This event is being reported in accordance with 10CFR50.73(a)(2)(iv) as an automatic actuation of an emergency cooling water system. The actuation occurred due to a low pressure signal in the CCP system, caused by a partial drainage of a section of the system during testing and maintenance. The manual valves that caused the drainage have been tentatively scheduled for repair in the next refueling outage. There was no loss of cooling water to any safety-related components, and the standby service water system responded to the actuation signal as designed. This event was of minimal safety significance.

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FAILURE CONTINUATION

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		2006	- 005	- 00		

REPORTED CONDITION

On May 1, 2006, at approximately 6:59 a.m., an unplanned automatic actuation of the Division 1 standby service water (SSW) (**BI**) system occurred during a realignment of the reactor plant component cooling (CCP) (**CC**) water system. The plant was in cold shutdown at the time for a refueling outage. This event is being reported in accordance with 10CFR50.73(a)(2)(iv) as an automatic actuation of an emergency cooling water system.

CAUSAL ANALYSIS AND IMMEDIATE ACTIONS

The CCP system provides cooling water to safety-related and non-safety related components in the auxiliary building and reactor building. Those safety-related components it serves are provided a back-up source of cooling water by the SSW system. This is accomplished by automatically starting the SSW pumps and realigning the CCP flowpath upon a low pressure condition in either of the redundant divisional CCP loops.

On April 29, a section of the CCP system inside the reactor building was isolated and drained under a clearance order to allow maintenance on the "A" reactor recirculation pump. The affected portion of the CCP system drained as expected, but the clearance boundary valves (**V**) were leaking approximately 2 gpm. The leakage did not affect the operation of the rest of the CCP system, as there is an automatic makeup feature that maintains system inventory.

On May 1, integrated emergency core cooling system testing was being conducted which involved a planned automatic closure of the containment isolation valves in the CCP system. The drained portion of the system under the clearance order was part of the header inside the reactor building that was isolated by the containment isolation signal. The containment isolation valves were closed for approximately four hours before post-test restoration was begun. During that time, the leakage past the clearance boundary valves partially drained the header inside the reactor building. When the containment isolation valves were opened to restore the system to its normal alignment, the partially drained header caused a sudden decrease in CCP system pressure. The low pressure signal actuated the Division 1 SSW subsystem as designed.

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CORRECTIVE ACTIONS TO PREVENT RECURRENCE

Following completion of the maintenance on the "A" reactor recirculation pump, the CCP system was fully restored to service. As such, the system is no longer susceptible to a repeat occurrence while the plant is operating. The valves used as a clearance boundary for the maintenance on "A" recirculation pump have been tentatively scheduled for repair in the next refueling outage to correct the seat leakage. This action will be tracked by the station's corrective action program.

PREVIOUS OCCURRENCE EVALUATION

No recent actuations of the SSW system were caused by the same mechanism as this event.

SAFETY SIGNIFICANCE

The Division 1 SSW system responded as designed to the low pressure condition in the CCP system. There was no loss of cooling water flow to any safety-related components as a result of this event. The seat leakage of the affected CCP valves did not cause any component to be incapable of performing its safety function. This event was of minimal safety significance.

(NOTE: Energy Industry Component Identification codes are annotated as (**XX**).)