



June 28, 2006

10 CFR 50.73(a)(2)(iv)(A)

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Palisades Nuclear Plant  
Docket 50-255  
License No. DPR-20

Licensee Event Report 06-004, Reactor Protection System Actuation

Licensee Event Report (LER) 06-004 is enclosed. The LER describes an actuation of the reactor protection system and is reportable in accordance with 10 CFR 50.73(a)(2)(iv)(A).

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.

Paul A. Harden  
Site Vice President, Palisades Nuclear Plant  
Nuclear Management Company, LLC

Enclosure (1)

CC Administrator, Region III, USNRC  
Project Manager, Palisades, USNRC  
Resident Inspector, Palisades, USNRC

**ENCLOSURE 1**

**LER 06-004, Reactor Protection System Actuation**

2 Pages Follow

<b>NRC FORM 366</b> (6-2004)	<b>U.S. NUCLEAR REGULATORY COMMISSION</b>	<b>APPROVED BY OMB NO. 3150-0104</b>	<b>EXPIRES 6-30-2007</b>
<b>LICENSEE EVENT REPORT (LER)</b>  (See reverse for required number of digits/characters for each block)		Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0066), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.	

<b>FACILITY NAME (1)</b> <b>Palisades Nuclear Plant</b>	<b>DOCKET NUMBER (2)</b> <b>05000-255</b>	<b>PAGE (3)</b> <b>1 of 2</b>
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**TITLE (4)**  
**Reactor Protection System Actuation**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
05	04	2006	2006	-- 004 --	00	06	28	2006	FACILITY NAME	DOCKET NUMBER	
<b>OPERATING MODE (9)</b>		<b>5</b>		<b>THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR * : (Check all that apply) (11)</b>							
<b>POWER LEVEL (10)</b>		<b>000</b>		20.2201(b)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)(B)		50.73(a)(2)(ix)(A)	
				20.2201(d)		20.2203(a)(4)		50.73(a)(2)(iii)		50.73(a)(2)(x)	
				20.2203(a)(1)		50.36(c)(1)(i)(A)		X 50.73(a)(2)(iv)(A)		73.71(a)(4)	
				20.2203(a)(2)(i)		50.36(c)(1)(ii)(A)		50.73(a)(2)(v)(A)		73.71(a)(5)	
				20.2203(a)(2)(ii)		50.36(c)(2)		50.73(a)(2)(v)(B)		<b>OTHER</b> Specify in Abstract below or in NRC Form 366A	
				20.2203(a)(2)(iii)		50.46(a)(3)(ii)		50.73(a)(2)(v)(C)			
				20.2203(a)(2)(iv)		50.73(a)(2)(i)(A)		50.73(a)(2)(v)(D)			
				20.2203(a)(2)(v)		50.73(a)(2)(i)(B)		50.73(a)(2)(vii)			
				20.2203(a)(2)(vi)		50.73(a)(2)(i)(C)		50.73(a)(2)(viii)(A)			
				20.2203(a)(3)(i)		50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)			

**LICENSEE CONTACT FOR THIS LER (12)**

<b>NAME</b> <b>Daniel G. Malone</b>	<b>TELEPHONE NUMBER (Include Area Code)</b> <b>(269) 764-2463</b>
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**COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)**

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		
YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO				

**ABSTRACT**

On May 4, 2006, with the plant in Mode 5, an actuation of the reactor protection system (RPS) occurred during performance of startup testing activities. The RPS actuation signal was generated from activation of the 'A' steam generator low level trip circuitry.

At the time of occurrence, a slight nitrogen overpressure of approximately 1.3 psi was being maintained on the secondary side of 'A' steam generator for chemistry control, as is typical for idle steam generators when in Mode 5.

Subsequently, when the main steam isolation valves were opened to support a planned test activity, a sudden release of the nitrogen overpressure in 'A' steam generator occurred. This resulted in a momentary level oscillation within 'A' steam generator between the tube bundle region and the downcomer region where steam generator level is measured.

While there was no loss of water inventory from 'A' steam generator, level in the downcomer region oscillated approximately 15% above and below the initial indicated level of approximately 40%. Since the RPS trip setpoint for low steam generator level is approximately 28%, the magnitude of the level oscillation was sufficient to exceed the RPS trip setpoint.

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
<b>Palisades Nuclear Plant</b>	<b>05000-255</b>	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 of 2
		<b>2006</b>	<b>-- 004</b>	<b>-- 00</b>	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

### EVENT DESCRIPTION

On May 4, 2006, with the plant in Mode 5, an actuation of the reactor protection system (RPS) [JC] occurred during performance of startup testing activities. The RPS actuation signal was generated from activation of the 'A' steam generator [SG;AB] low level trip circuitry.

At the time of occurrence, the RPS was reset and a single control rod was fully withdrawn for testing. As a result of the RPS actuation, the withdrawn control rod inserted into the core as expected.

This actuation is reportable in accordance with 10 CFR 50.73(a)(2)(iv)(A).

### CAUSE OF THE EVENT

At the time of occurrence, a slight nitrogen overpressure of approximately 1.3 psi was being maintained on the secondary side of 'A' steam generator for chemistry control, as is typical for idle steam generators when in Mode 5.

Subsequently, when the main steam isolation valves (MSIVs) [ISV;SB] were opened to support a planned test activity, a sudden release of the nitrogen overpressure in 'A' steam generator occurred. This resulted in a momentary level oscillation within 'A' steam generator between the tube bundle region and the downcomer region where steam generator level is measured.

While there was no loss of water inventory from 'A' steam generator, level in the downcomer region oscillated approximately 15% above and below the initial indicated level of approximately 40%. Since the RPS trip setpoint for low steam generator level is approximately 28%, the magnitude of the level oscillation was sufficient to exceed the RPS trip setpoint.

### CORRECTIVE ACTIONS

Procedures will be revised to appropriately address the potential consequences of opening MSIVs when the reactor is reset as observed in this occurrence.

### SAFETY SIGNIFICANCE

The safety significance of this occurrence is considered minimal since the reactor was already in a shutdown condition. There was no adverse safety impact on any equipment or the plant.

### PREVIOUS SIMILAR EVENTS

None