

Power Reactor

Event # 42658

Site: CALLAWAY		Notification Date / Time: 06/21/2006 10:02 (EDT)				
Unit: 1	Region: 4	State : MO	Event Date / Time: 06/19/2006 16:00 (CDT)			
Reactor Type: [1] W-4-LP		Last Modification: 06/22/2006				
Containment Type: DRY AMB						
NRC Notified by: DENNIS GRIFFITH		Notifications: DAVID GRAVES R4				
HQ Ops Officer: JEFF ROTTON		OMID TABATABAI NRR				
Emergency Class: NON EMERGENCY						
10 CFR Section:						
21.21		UNSPECIFIED PARAGRAPH				
Unit	Scram Code	RX Crit	Init Power	Initial RX Mode	Curr Power	Current RX Mode
1	N	Yes	100	Power Operation	100	Power Operation

INITIAL PART 21 NOTIFICATION - PRIME MEASUREMENT PRODUCTS, MODELS 763 AND 763A GAGE PRESSURE TRANSMITTERS AND MODEL 764 DIFFERENTIAL PRESSURE TRANSMITTERS

"This report is made per 10CFR21.21 Notification of failure to comply or existence of a defect and its evaluation.

"Callaway Plant received a notification dated May 18, 2006, from PRIME Measurement Products (formerly ITT Barton) stating Barton Model 763 and 763A Gage Pressure Transmitters and Model 764 Differential Pressure Transmitters may have defective external lead-wire connectors, which could affect their performance during an accident. The notification provided by PRIME stated the affected transmitters were manufactured after May 1982 and shipped from the factory prior to April 1, 2006.

"The potential defect is characterized as follows: The transmitters' external lead wires enter the electronics enclosure through a hermetic seal called a connector assembly. The external lead wires are soldered to the glass sealed pins of the hermetic seal. Epoxy potting is used to structurally support the soldered wire connections and establish a seal to protect the solder connections from shorting, which could be caused by an electrically conductive accident environment. The defect is that the insulated portions of the wires in the connectors, manufactured after May 1982, may not be embedded deeply enough into the epoxy potting to provide an electrical connection that would not be affected in an accident environment.

"The notification also states that actual transmitter installation may preclude shorting of exposed conductors due to the existence of conduit, conduit seals, and special wire connectors which could protect the exposed wires at a defective connector from conductive moisture.

"Callaway Plant identified thirty applicable Barton transmitters and connector assemblies in warehouse stock. Inspections of these operational spare parts identified three connectors, which had exposed conductors external to their seal. One connector had this defect on one lead wire and two connectors had this defect on both lead wires.

IE19

Power Reactor

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"Callaway has also identified that thirty-nine potentially affected transmitters are installed in the plant. An inspection plan has been developed to perform the required inspections, based on safety significance, ALARA considerations, and potential accident environmental conditions. Additional reporting requirements associated with the installed components will be evaluated under 10CFR50.72 as required.

"On 06/19/06, Callaway Plant personnel completed evaluations and determined the defective connections constitute a defect per 10CFR21 require initial NRC notification within two days.

"The NRC resident inspectors have been notified of this issue."

*** UPDATE FROM LICENSEE (D. GRIFFITH) TO M. RIPLEY 1515 EDT 06/22/06 ***

"This report is a revision to report EN # 42658 reported on 06/21/2006.

Added statement: "The suppliers of the identified transmitters were Prime Measurement Products and Westinghouse Electric Corporation."

The licensee notified the NRC Resident Inspector. Notified NRR (O. Tabatabai), R4 DO (D. Graves)

NRC FORM 361 (12-2000)	U.S. NUCLEAR REGULATORY COMMISSION OPERATIONS CENTER EN # 42058
REACTOR PLANT EVENT NOTIFICATION WORKSHEET	

NRC OPERATION TELEPHONE NUMBER: PRIMARY - 301-818-5100 OR 800-532-3469*, BACKUPS - (1ST) 301-951-0550 OR 800-448-3594* (2ND) 301-415-0550 AND (3RD) 301-415-0553 *Licensees who maintain their own ETS are provided these telephone numbers.

NOTIFICATION TIME 10:02 EDT ✓	FACILITY OR ORGANIZATION Callaway Plant ✓	UNIT 1 ✓	NAME OF CALLER Dennis Wade Griffith ✓	CALL BACK # David R. Waller ✓ (573) 676-8595
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EVENT TIME & ZONE 16:00 CDT	EVENT DATE 6/19/2006	POWER / MODE BEFORE 100% POWER / MODE 1	POWER / MODE AFTER 100% POWER / MODE 1
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EVENT CLASSIFICATIONS		1-Hr. Non-Emergency 10 CFR 50.72(b)(1)		60-Day Optional 10 CFR 50.73(a)(1)	
GENERAL EMERGENCY	GEN/AAEC	TS Deviation		(v)(A) Safe S/D Capability	AINA
SITE AREA EMERGENCY	SIT/AAEC	4-Hr. Non-Emergency 10 CFR 50.72(b)(2)		(v)(B) RHR Capability	AINB
ALERT	ALE/AAEC	(i)	TS Required S/D	(v)(C) Control of Rad Release	AINC
UNUSUAL EVENT	UNU/AAEC	(iv)(A)	ECCS Discharge to RCS	(v)(D) Accident Mitigation	AIND
50.72 NON-EMERGENCY	(see next column)	(iv)(B)	REFS Accumulation (acram)	(v)(E) Offsite Medical	AINES
PHYSICAL SECURITY (7171)	DDDD	(v)	Offsite Notification	(v)(F) Loss Control/Asmt/Resp	AINF
MATERIAL/EXPOSURE	DM	8-Hr. Non-Emergency 10 CFR 50.72(b)(3)		Invalid Specified System Actuation	AINV
FITNESS FOR DUTY	HIT	(ii)(A)	Degraded Condition	Other Unspecified Requirement (Identify)	
X OTHER UNSPECIFIED REQMT	(see last column)	(ii)(B)	Unspecified Condition	ADRG	AINA
INFORMATION ONLY	NNNF	(iii)(A)	Specified System Actuation	ALSA	AINR

DESCRIPTION

Include: Systems affected, actuations and their initiating signals, causes, effect of event on plant, actions taken or planned, etc (Continue on back)

INITIAL PART 21 NOTIFICATION - PRIME MEASUREMENT PRODUCTS, MODELS 763 AND 763A GAGE PRESSURE TRANSMITTERS AND MODEL 764 DIFFERENTIAL PRESSURE TRANSMITTERS

This report is made per 10CFR21.21 Notification of failure to comply or existence of a defect and its evaluation.

Callaway Plant received a notification dated May 18, 2006, from PRIME Measurement Products (formerly ITT Barton) stating Barton Model 763 and 763A Gage Pressure Transmitters and Model 764 Differential Pressure Transmitters may have defective external lead-wire connectors, which could affect their performance during an accident. The notification provided by PRIME stated the affected transmitters were manufactured after May 1982 and shipped from the factory prior to April 1, 2006.

The potential defect is characterized as follows: The transmitters' external lead wires enter the electronics enclosure through a hermetic seal called a connector assembly. The external lead wires are soldered to the glass sealed pins of the hermetic seal. Epoxy potting is used to structurally support the soldered wire connections and establish a seal to protect the solder connections from shorting, which could be caused by an electrically conductive accident environment. The defect is that the insulated portions of the wires in the connectors, manufactured after May 1982, may not be embedded deeply enough into the epoxy potting to provide an electrical connection that would not be affected in an accident environment.

The notification also states that actual transmitter installation may preclude shorting of exposed conductors due to the existence of conduit, conduit seals, and special wire connectors which could protect the exposed wires at a defective connector from conductive moisture.

NOTIFICATIONS	YES	NO	WILL BE	ANYTHING UNUSUAL OR NOT UNDERSTOOD?
NRC RESIDENT	X			<input type="checkbox"/> YES (explain above) <input checked="" type="checkbox"/> NO
STATE(s)		X		DID ALL SYSTEMS FUNCTION AS REQUIRED?
LOCAL		X		<input checked="" type="checkbox"/> YES <input type="checkbox"/> NO (explain above)
OTHER GOV AGENCIES		X		MODE OF OPERATION UNTIL CORRECTED: 1
MEDIA/PRESS RELEASE		X		ESTIMATED RESTART DATE: N/A
				ADDITIONAL INFO ON BACK? <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO

ADDITIONAL INFORMATION

PAGE 3 OF 2

RADIOLOGICAL RELEASES: CHECK OR FILL IN APPLICABLE ITEMS (specific details/explanations should be covered in event description).						
LIQUID RELEASE	GASEOUS RELEASE		UNPLANNED RELEASE	PLANNED RELEASE	ONGOING	TERMINATED
MONITORED	UNMONITORED		OFFSITE RELEASE	T'S EXCEEDED	PM ALARMS	AREAS EVACUATED
PERSONNEL EXPOSED OR CONTAMINATED			OFFSITE PROTECTIVE ACTIONS RECOMMENDED			* State release path in description
	Release Rate (Ci/sec)	% T.S. LIMIT	HOO GUIDE	Total Activity (Ci)	% T.S. LIMIT	HOO GUIDE
Noble Gas			0.1 Ci/sec			1000 Ci
Iodine			10 uCi/sec			0.01 Ci
Particulate			1 uCi/sec			1 mCi
Liquids (excluding tritium and dissolved noble gases)			10 uCi/min			0.1 Ci
Liquid (tritium)			0.2 Ci/min			5 Ci
Total Activity						
	PLANT STACK	CONDENSER/AIR EJECTOR	MAIN STEAM LINE	SG BLOWDOWN	OTHER	
RAD MONITOR READINGS						
ALARM SETPOINTS						
% T.S. LIMIT (if applicable)						
RCS OR SG TUBE LEAKS: CHECK OR FILL IN APPLICABLE ITEMS: (specific details/explanations should be covered in event description)						
LOCATION OF THE LEAK (e.g., SG #, valve, pipe, etc)						
LEAK RATE	UNITS: gpm/gpd	T.S. LIMITS	SUDDEN OR LONG-TERM DEVELOPMENT			
LEAK START DATE	TIME	COOLANT ACTIVITY AND UNITS:	PRIMARY	SECONDARY		
LIST OF SAFETY RELATED EQUIPMENT NOT OPERATIONAL						

EVENT DESCRIPTION (Continued from front)

Callaway Plant identified thirty applicable Barton transmitters and connector assemblies in warehouse stock. Inspections of these operational spare parts identified three connectors, which had exposed conductors external to their seal. One connector had this defect on one lead wire and two connectors had this defect on both lead wires.

Callaway has also identified that thirty-nine potentially affected transmitters are installed in the plant. An inspection plan has been developed to perform the required inspections, based on safety significance, ALARA considerations, and potential accident environmental conditions. Additional reporting requirements associated with the installed components will be evaluated under 10CFR50.72 as required.

On 06/19/2006, Callaway Plant personnel completed evaluations and determined the defective connectors constitute a defect per 10CFR21 requiring initial NRC notification within two days.

All other affected customers have been notified via the PRIME Nuclear Industry Advisory discussed above.

The NRC resident inspectors have been notified of this issue.

UPDATE TO 42658

NRC FORM 361
(12-2000)U.S. NUCLEAR REGULATORY COMMISSION
OPERATIONS CENTERREACTOR PLANT
EVENT NOTIFICATION WORKSHEET

EN # 42658

NRC OPERATION TELEPHONE NUMBER: PRIMARY - 301-816-5100 OR 800-532-3469*, BACKUPS - (1ST) 301-951-0550 OR 800-449-3894*
(2ND) 301-415-0550 AND (3RD) 301-415-0553 *Licensees who maintain their own ETS are provided these telephone numbers.

NOTIFICATION TIME 15:15 EDT	FACILITY OR ORGANIZATION Callaway Plant	UNIT 1	NAME OF CALLER Dennis Wade Griffith	CALL BACK # David R. Waller (573) 676-8595
EVENT TIME & ZONE 16:00 CDT	EVENT DATE 6/19/2006	POWER / MODE BEFORE 100% POWER / MODE 1	POWER / MODE AFTER 100 % POWER / MODE 1	
EVENT CLASSIFICATIONS		1-Hr. Non-Emergency 10 CFR 50.72(b)(1)		(v)(A) Safe S/D Capability AINA
GENERAL EMERGENCY	GEN/AAEC	TS Deviation ADEV		(v)(B) RHR Capability AINB
SITE AREA EMERGENCY	SIT/AAEC	4-Hr. Non-Emergency 10 CFR 50.72(b)(2)		(v)(C) Control of Rad Release AINC
ALERT	ALP/AAEC	(i)	TS Required S/D ASHU	(v)(D) Accident Mitigation AIND
UNUSUAL EVENT	UNU/AAEC	(iv)(A)	ECCS Discharge to RCS ACCS	(xii) Offsite Medical AMED
50.73 NON-EMERGENCY	(see next column)	(iv)(B)	RPS Actuation (steam) ARPS	(xiii) Loss Control/Alarm/Reset ACOM
PHYSICAL SECURITY (71.71)	DDDD	(xi)	Offsite Notification APRF	60-Day Optional 10 CFR 50.73(a)(1)
MATERIAL EXPOSURE	B777	8-Hr. Non-Emergency 10 CFR 50.72(b)(3)		Invalid Specified System Actuation AINV
FITNESS FOR DUTY	HFIT	(ii)(A)	Degraded Condition ADEG	Other Unspecified Requirement (Identify)
X OTHER UNSPECIFIED REQMT	(see last column)	(ii)(B)	Unanalyzed Condition AUNA	X 10 CFR 21.21 Reporting of Defects/Noncompliance NONR
INFORMATION ONLY	NINF	(iv)(A)	Specified System Actuation AESF	NONR

DESCRIPTION

Include: Systems affected, actuations and their initiating signals, causes, effect of event on plant, actions taken or planned, etc (Continue on back)

This report is a revision to report EN # 42658 reported on 06/21/2006.

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STATE(s)		X		DID ALL SYSTEMS FUNCTION AS REQUIRED?
LOCAL		X		<input checked="" type="checkbox"/> YES <input type="checkbox"/> NO (explain above)
OTHER GOV AGENCIES		X		MODE OF OPERATION
MEDIA/PRESS RELEASE		X		UNTIL CORRECTED: 1
				ESTIMATED RESTART DATE: N/A
				ADDITIONAL INFO ON BACK? <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO

NRC FORM 361 (12-2000)

UPDATE

ADDITIONAL INFORMATION

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RADIOLOGICAL RELEASES: CHECK OR FILL IN APPLICABLE ITEMS (specific details/explanations should be covered in event description).						
LIQUID RELEASE	GASEOUS RELEASE	UNPLANNED RELEASE	PLANNED RELEASE	ONGOING	TERMINATED	
MONITORED	UNMONITORED	OFFSITE RELEASE	T.S. EXCEEDED	PM ALARMS	AREAS EVACUATED	
PERSONNEL EXPOSED OR CONTAMINATED		OFFSITE PROTECTIVE ACTIONS RECOMMENDED		* State release path in description		
	Release Rate (Ci/sec)	% T.S. LIMIT	HOO GUIDE	Total Activity (Ci)	% T.S. LIMIT	HOO GUIDE
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Iodine			10 uCi/sec			0.01 Ci
Particulate			1 uCi/sec			1 mCi
Liquid (excluding tritium and dissolved noble gases)			10 uCi/min			0.1 Ci
Liquid (tritium)			0.2 Ci/min			5 Ci
Total Activity						
	PLANT STACK	CONDENSER/AIR EJECTOR	MAIN STEAM LINE	SG BLOWDOWN	OTHER	
RAD MONITOR READINGS						
ALARM SETPOINTS						
% T.S. LIMIT (if applicable)						
RCS OR SG TUBE LEAKS: CHECK OR FILL IN APPLICABLE ITEMS: (specific details/explanations should be covered in event description)						
LOCATION OF THE LEAK (e.g., SG #, valve, pipe, etc)						
LEAK RATE	UNITS: gpm/gpd	T.S. LIMITS	SUDDEN OR LONG-TERM DEVELOPMENT			
LEAK START DATE	TIME	COOLANT ACTIVITY AND UNITS:	PRIMARY	SECONDARY		
LIST OF SAFETY RELATED EQUIPMENT NOT OPERATIONAL.						
EVENT DESCRIPTION (Continued from front)						
<p>Callaway Plant identified thirty applicable Barton transmitters and connector assemblies in warehouse stock. Inspections of these operational spare parts identified three connectors, which had exposed conductors external to their seal. One connector had this defect on one lead wire and two connectors had this defect on both lead wires.</p> <p>Callaway has also identified that thirty-nine potentially affected transmitters are installed in the plant. An inspection plan has been developed to perform the required inspections, based on safety significance, ALARA considerations, and potential accident environmental conditions. Additional reporting requirements associated with the installed components will be evaluated under 10CFR50.72 as required.</p> <p>→ UPDATE The suppliers of the identified transmitters were Prime Measurement Products and Westinghouse Electric Corporation.</p> <p>On 06/19/2006, Callaway Plant personnel completed evaluations and determined the defective connectors constitute a defect per 10CFR21 requiring initial NRC notification within two days.</p> <p>All other affected customers have been notified via the PRIME Nuclear Industry Advisory discussed above.</p> <p>The NRC resident inspectors have been notified of this issue.</p>						