

July 17, 2006

Mr. John S. Keenan
Senior Vice President and Chief Nuclear Officer
Pacific Gas and Electric Company
Diablo Canyon Power Plant
P.O. Box 770000
San Francisco, CA 94177-0001

SUBJECT: DIABLO CANYON POWER PLANT, UNIT NOS. 1 AND 2 - ISSUANCE OF
AMENDMENTS RE: REQUEST FOR RECOVERY OF LOW-POWER TESTING
TIME - IMPACT ON THE REACTOR VESSEL INTEGRITY ASSESSMENTS
(TAC NOS. MC8206 AND MC8207)

Dear Mr. Keenan:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 188 to Facility Operating License No. DPR-80 and Amendment No. 190 to Facility Operating License No. DPR-82 for the Diablo Canyon Power Plant, Unit Nos. 1 and 2 (DCPP-1/2), respectively. The amendments consist of changes to the Facility Operating Licenses in response to your application dated August 23, 2005, as supplemented on April 6, 2006.

The amendments extend the licensed lives of the DCPP-1/2 reactors by the amount of time the licensee had expended to perform low-power testing of the reactors prior to initial startup.

A copy of the related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

/RA/

Alan Wang, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-275
and 50-323

Enclosures: 1. Amendment No. 188 to DPR-80
2. Amendment No. 190 to DPR-82
3. Safety Evaluation

cc w/encls: See next page

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cc w/encls: See next page

ACCESSION NO.: Pkg ML062260278 (ML061660220; TS Pgs. ML062270019)

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| OFFICE | NRR/LPL4/PM | NRR/LPL4/LA | ADES/CVIB/BC | OGC | NRR/LPL4/BC |
| NAME | AWang | LFeizollahi | MMitchell | HEW | DTerao |
| DATE | 6/21/06 | 6/20/06 | 5/31/06 | 7/3/06 | 7/12/06 |

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PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-275

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 188
License No. DPR-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated August 23, 2005, as supplemented on April 6, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Facility Operating License No. DPR-80 as indicated in the attachment to this license amendment.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

David Terao, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to pages 3 and 9 of
Facility Operating License No. DPR-80

Date of Issuance: July 17, 2006

PACIFIC GAS AND ELECTRIC COMPANY

DOCKET NO. 50-323

DIABLO CANYON NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 190
License No. DPR-82

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Pacific Gas and Electric Company (the licensee) dated August 23, 2005, as supplemented on April 6, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Facility Operating License No. DPR-82 as indicated in the attachment to this license amendment.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

David Terao, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to pages 3 and 7 of
Facility Operating License No. DPR-82

Date of Issuance: July 17, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 188

TO FACILITY OPERATING LICENSE NO. DPR-80

AND AMENDMENT NO. 190 TO FACILITY OPERATING LICENSE NO. DPR-82

DOCKET NOS. 50-275 AND 50-323

Replace the following pages of the Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Operating License No. DPR-80:

REMOVE

- 3 -

- 9 -

INSERT

- 3 -

- 9 -

Facility Operating License No. DPR-82:

REMOVE

- 3 -

- 7 -

INSERT

- 3 -

- 7 -

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. DPR-80
AND AMENDMENT NO. 190 TO FACILITY OPERATING LICENSE NO. DPR-82
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON POWER PLANT, UNITS 1 AND 2
DOCKET NOS. 50-275 AND 50-323

1.0 INTRODUCTION

By application dated August 23, 2005, as supplemented on April 6, 2006 (Agencywide Documents Access and Management System Accession Nos. ML052420441 and ML061040224, respectively), Pacific Gas and Electric Company (PG&E, the licensee) requested changes to the Facility Operating Licenses (Facility Operating License Nos. DPR-80 and DPR-82) for the Diablo Canyon Power Plant, Units 1 and 2 (DCPP-1/2).

The proposed amendments would extend the licensed lives of the DCPP-1/2 reactors by the amount of time the licensee had expended to perform low-power testing of the reactors prior to initial startup. Specifically, the proposed changes would revise the licenses to reflect the following:

1. Extend the expiration date for DCPP-1 Operating License No. DPR-80 from September 22, 2021, to November 2, 2024. This revised expiration date equates to 35.2 effective full power years (EFPY) of power operations.
2. Extend the expiration date for DCPP-2 Operating License No. DPR-82 from April 26, 2025 to August 26, 2025. This revised expiration date equates to 35.8 EFPY.

The supplemental letter dated April 6, 2006, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on October 11, 2005 (70 FR 59087).

2.0 REGULATORY EVALUATION

2.1 Requirements for Upper Shelf Energy (USE)

Section IV.A.1 to Title 10, *Code of Federal Regulations* (10 CFR) Part 50, Appendix G (Reference 4), provides the Commission's requirements for demonstrating that reactor vessels (RVs) in U.S. pressurized-water reactor (PWR), light-water facilities will maintain adequate protection from failure by ductile tearing throughout their service lives. The rule requires RV beltline materials to have USE values equal to or above 75 ft-lb when the materials are in the unirradiated condition and equal to or above 50 ft-lb throughout the licensed life of the reactor. Regulatory Guide (RG) 1.99, Revision 2, "Radiation Embrittlement of Reactor Vessel Materials," provides an expanded discussion regarding the calculation of USE values and describes two methods for determining USE values for RV beltline materials, depending on whether or not a given RV beltline material is represented in the plant's Reactor Vessel Material Surveillance Program (RVMSP).

2.2 Requirements for Performing RT_{PTS} Calculations

Section 50.61 of 10 CFR (Reference 7) provides the Commission's requirements for demonstrating that RVs in U.S. PWR light-water reactor facilities will have adequate protection against the consequences of pressurized thermal shock (PTS) events throughout their service lives. The rule requires PWR licensees to calculate an adjusted reference temperature value for PTS (i.e., an RT_{PTS} value) for each base metal and weld material located in the beltline region of their RVs. The rule sets a screening limit of 270 EF for RT_{PTS} values that are calculated for base metals (i.e., forging and plate materials) and axial weld materials, and a screening limit of 300 EF for RT_{PTS} values that are calculated for circumferential weld materials. The rule also provides an expanded discussion regarding how the calculations of RT_{PTS} values should be performed and describes two methods for determining RT_{PTS} values for RV beltline materials, depending on whether or not a given RV beltline material is represented in the plant's RVMSP.

2.3 RVMSP Requirements

The NRC staff's regulatory requirements for the establishment of RVMSPs are given in 10 CFR Part 50, Appendix H (Reference 8).

2.4 Requirements for Pressure-Temperature (P-T) Limits

Section 50.36 of 10 CFR (Reference 5) requires that the Technical Specification (TS) limiting conditions of operation (LCOs) include LCOs on P-T limits for operating reactors. Section IV.A.2 of 10 CFR Part 50, Appendix G, requires that P-T limits for normal operations (including heatups, cooldowns, operation with the core critical, and operation during anticipated operational transients) and pressure test operations of operating reactors be at least as conservative as those that would be generated using the methodology in Appendix G to Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Section XI, Appendix G).

The guidance in Generic Letter (GL) No. 96-03, "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits" (Reference 6), permits licensees to relocate the actual P-T limit curves from the LCO's into an owner controlled

pressure-temperature limit report (PTLR). The PTLR is controlled under the TS Administrative Controls Section based on the licensee's use of an NRC-approved P-T limit calculational methodology and is provided the PTLR satisfies particular criteria that are provided in Attachment 1 of GL 96-03. NRC approval of a license amendment for a PTLR permits the particular licensee to make further changes of the P-T limits without having to submit them for NRC approval.

3.0 TECHNICAL EVALUATION

By letter dated January 27, 2006 (Reference 2), the NRC staff issued a request for additional information (RAI) requesting PG&E to provide the adjusted reference temperature (RT_{PTS}) value and Charpy-V notch USE value calculations for the DCPP-1/2 RV beltline materials, as assessed for the neutron fluence exposure (in units of n/cm^2 , $E > 1.0$ MeV) to the materials at the new expiration dates for the reactor units. The licensee responded to the NRC staff's RAI by letter dated April 6, 2006 (PG&E Serial Letter No. DCL-06-045/Reference 3). In this letter, the licensee provided the following information: (1) the amended neutron fluence values for the RV beltline materials, as assessed at the RV clad-to-base metal interface through the new expiration dates for the units, and (2) the limiting RT_{PTS} values and limiting USE values for the DCPP-1/2 RV beltline materials, as assessed through the new expiration dates for the units. All future references to the licensee's response to the NRC staff's RAI are made in reference to Serial Letter No. DCL-06-045.

The evaluations in Section 3.0 of this safety evaluation (SE) provide the NRC staff's assessment of the impact of the license amendment request on the following mandated RV programs and analyses:

- (1) the safety assessment that is required by 10 CFR Section 50.61 to ensure protection of the DCPP-1/2 RVs against the consequences of a PTS event,
- (2) the safety assessment that is required by 10 CFR Part 50, Appendix G, to ensure that the DCPP-1/2 RV beltline materials will have acceptable levels of USE at the expiration of the operating licenses,
- (3) the RV materials surveillance program withdrawal schedule that is mandated by 10 CFR Part 50, Appendix H, and
- (4) the pressure-temperature limits for the RVs, as mandated in accordance with 10 CFR 50.36 and regulated in accordance with the requirements of 10 CFR Part 50, Appendix G, and Administrative Technical Specification 5.9.6, which controls the licensee's PTLR process.

3.1 USE Assessment

The licensee performed updated USE assessments for the RV beltline materials at DCPP-1/2 and provided the updated limiting USE values in its response to the NRC staff's RAI. The USE assessments were based on the 1/4T neutron fluence values for the RV beltline materials, as assessed for the revised expiration dates of the DCPP-1/2 operating licenses. This is consistent with the recommended methods in RG 1.99, Revision 2, for performing USE calculation of RV beltline materials.

The NRC staff performed independent calculations of the USE values for the RV beltline materials and applied the 1/4T neutron fluence values for the DCP-1/2 RVs as its basis for its independent USE calculations. These values were based on the updated inside surface fluences provided in PG&E Serial Letter No. DCL-06-045, as attenuated using the methods of analysis in RG 1.99, Revision 2.

The NRC staff determined that for its USE assessment of DCP-1, Lower Shell Axial Weld No. 3-442C (Heat No. 27204) is the limiting beltline material for USE. The NRC staff calculated a USE of 64.5 ft-lb for this weld at 35.2 EFPY. This value is in reasonable agreement with the USE value calculated by the licensee for these welds (i.e., 61.1 ft-lb) at 35.2 EFPY. Both of these values meet the acceptance criterion in 10 CFR Part 50, Appendix G, for maintaining the USE values of the RV beltline materials above 50 ft-lbs throughout the licensed life of the plant.

The NRC staff determined that for DCP-2, Lower Shell Axial Weld No. 3-201B (Heat No. 33A277) is the limiting beltline material for USE. The NRC staff calculated a USE of 57.5 ft-lb for this weld at 35.8 EFPY. This value is in good agreement with the USE value calculated by the licensee for this welds (i.e., 57.7 ft-lb) at 35.8 EFPY. Both of these values meet the acceptance criterion in 10 CFR Part 50, Appendix G, for maintaining the USE values of the RV beltline materials above 50 ft-lbs throughout the licensed life of the plant.

Based on this assessment, the NRC staff concludes the RV beltline materials at DCP-1/2 will have acceptable remaining margins on USE through the amended expiration dates for the current DCP-1/2 operating licenses.

3.2 Impact on the PTS Assessments and RT_{PTS} Calculations

The licensee performed updated RT_{PTS} calculations for the RV beltline materials at DCP-1/2 and provided the updated limiting RT_{PTS} values in its response in PG&E Serial Letter No. DCL-06-045. The licensee's RT_{PTS} calculations were performed in accordance with the methods of calculation in 10 CFR 50.61, as assessed using the updated neutron fluences for the RV beltline materials at the clad-to-based metal interface for the revised expiration dates for the current operating licenses. This is in compliance with 10 CFR 50.61.

The NRC staff performed independent calculations of the RT_{PTS} values for the RV beltline materials at DCP-1/2 through the revised expiration dates for the DCP-1/2 operating licenses. The NRC staff used the methods of analysis in 10 CFR 50.61 as the basis for its independent RT_{PTS} calculations. The NRC staff applied the updated clad-to-base metal neutron fluences that were provided in PG&E Serial Letter No. DCL-06-045.

The NRC staff determined that for DCP-1, Lower Shell Axial Weld No. 3-442C (Heat No. 27204) is the limiting beltline material for PTS. The NRC staff calculated a RT_{PTS} value of 258.8 EF for this weld at 35.2 EFPY. This value is in agreement with the RT_{PTS} value calculated by the licensee for this weld (i.e., 258.7 EF) at 35.2 EFPY. Both of these values meet (i.e., are lower than) the PTS screening criterion of 270 EF in 10 CFR 50.61 for RV axial weld materials at the end of the operating license (EOL) and are acceptable.

The NRC staff determined that for DCP-2, Intermediate Shell Plate No. B5454-2 (Heat No. C5168-2) is the limiting beltline material for PTS. The NRC staff calculated a RT_{PTS} value of 214.8 EF for this weld at 35.8 EFPY. This value is in agreement with the RT_{PTS} value

calculated by the licensee for this weld (i.e., 214.8 EF) at 35.8 EFPY. Both of these values meet (i.e., are lower than) the PTS screening criterion of 270 EF in 10 CFR 50.61 for RV axial weld materials at the EOL and are acceptable.

Based on this assessment, the NRC staff concludes the RV beltline materials as DCP-1/2 will remain in compliance with the PTS screening criteria of 10 CFR 50.61 through the amended expiration dates for the current DCP-1/2 operating licenses.

3.3 Impact on the RVMSP

Section III.B.3 of 10 CFR Part 50, Appendix H, requires that designs of RVMSPs and RV surveillance capsule withdrawal schedules be implemented in accordance with the version of American Society for Testing and Materials Standard Practice E185 (ASTM E185 [Reference 9]) that was current at the time the RV was purchased in accordance with Section III of the ASME Boiler and Pressure Vessel Code, Division 1. The rule permits more current versions of ASTM E185 to be used inclusive of the 1982 version (ASTM E185-82). The rule also requires that changes to the RV surveillance capsule withdrawal schedule be reviewed and approved by the NRC staff.

The licensee stated that the adjustments of the EOL neutron fluences for the RV beltline materials at the clad-to-base metal locations of the RVs do not require the RV material surveillance capsule withdrawal schedules for DCP-1/2 to be altered. The NRC staff reviewed the limiting neutron fluence values reported in PG&E Serial Letter No. DCL-06-045 for the clad-to-base metal location of the RVs, in order to determine whether the revised fluence values would impact the RVMSP withdrawal schedules for DCP-1/2.

The ASTM E185 version of record for DCP-1 is ASTM E185-70. The most recent RVMSP withdrawal schedule for DCP-1 was requested in PG&E Serial Letter No. DCL-92-072, dated March 31, 1992 (Reference 10). This RVMSP withdrawal schedule was approved in an SE to PG&E dated September 4, 1992 (Reference 11). In the SE, the NRC staff concluded the supplemental RVMSP withdrawal schedule met the criteria of ASTM E185-70 and constituted an acceptable withdrawal schedule for implementation under 10 CFR Part 50, Appendix H. Under this supplemental program, four capsules, Capsule S, Y, V, and B, were designated for removal from the DCP-1 RV. Capsules S, Y, and V have been removed and tested in accordance with the licensee's program.

The request to recover the testing time for DCP-1 amends the projected withdrawal for Capsule B to approximately 20.7 EFPY, when the capsule is projected to achieve a neutron fluence of 2.9×10^{19} n/cm² (E > 1.0 MeV). Therefore, the capsule will achieve a neutron fluence approximately equal to twice the projected limiting inside RV fluence for DCP-1 at the EOL (i.e., approximately $2 * 1.43 \times 10^{19}$ n/cm² [E > 1.0 MeV]). This complies with the criterion in ASTM E185-82 for withdrawal of the final capsule of a four capsule withdrawal program. This is acceptable because 10 CFR Part 50, Appendix H, permits the licensee's to meet the RVMSP withdrawal criteria of more recent versions of ASTM E185, inclusive of E185-82. Therefore, the NRC staff concludes that the adjustments to the withdrawal time and projected neutron fluence for Capsule B will still be in compliance with 10 CFR Part 50, Appendix H.

The ASTM E185 version of record for DCP-2 is ASTM E185-73. The most recent RVMSP withdrawal schedule for DCP-2 was requested in PG&E Serial Letter No. DCL-97-178, dated

October 22, 1997 (Reference 13). This RVMSP withdrawal schedule was approved in an SE to PG&E dated February 10, 1998 (Reference 14). In the NRC staff's SE, the NRC staff concluded the RVMSP withdrawal schedule provided in PG&E's letter of October 22, 1997, constituted an acceptable RVMSP withdrawal schedule for DCP-2 that met the withdrawal schedule criteria of ASTM E185-73 and met the intent of the more recent withdrawal schedule criteria of ASTM E185-82. Under this program, four capsules, Capsule U, X, Y, and V were designated for removal from the DCP-2 RV. PG&E has removed all of these capsules in accordance with the withdrawal schedule criteria that was approved in the NRC staff's SE of February 10, 1998.

PG&E is not required to withdraw any further surveillance capsules from DCP-2 for the current operating term and the proposed license amendment will not impact continued compliance of the DCP-2 RVMSP for the current operating period, as assessed against the requirements of 10 CFR Part 50, Appendix H.

3.4 Impact of Reactor Vessel P-T Limits

On May 13, 2004 (Reference 16), License Amendment No. 170 for DCP-1 Operating License No. DPR-80 and License Amendment No. 171 for DCP-2 Operating License No. DPR-82 were approved for DCP-1/2, respectively. These license amendments permitted PG&E to relocate the P-T limits for DCP-1/2 into a PTLR and to make administrative changes of the P-T limits in accordance with the PTLR process governed by TS 5.9.6. Thus, the licensee is authorized to make any changes of the P-T limits that are necessary to account for the change in the neutron fluence values resulting from the recovery of the power ascension time in accordance with the PTLR process (i.e., in accordance with TS 5.9.6).

3.5 Recovery of the Low-Power Testing Time Period

The NRC staff has reviewed PG&E's license amendment request to recover the low-power testing time that was performed during the initial startups of the DCP-1/2 reactors. The NRC staff has determined that authorization of the requested license may be granted based on the following conclusions:

- (1) The DCP-1/2 RV beltline materials will remain in compliance with the acceptance criteria of 10 CFR Part 50, Appendix G, for the remaining USE safety margins.
- (2) The revised RT_{PTS} values for the DCP-1/2 RV beltline materials are below the RT_{PTS} screening criteria. Therefore, the DCP-1/2 RV beltline materials remain in compliance with the requirements of 10 CFR 50.61 for demonstrating adequate protection against PTS events.
- (3) The RV surveillance capsule withdrawal schedules for DCP-1/2 remain in compliance with the requirements of 10 CFR Part 50, Appendix H, and the ASTM E185 versions of record for the units.
- (4) The licensee will make any changes that may be necessary to the DCP-1/2 P-T limits in accordance with the PTLR process that is administratively mandated and controlled under TS 5.9.6.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the California State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding on October 11, 2005 (70 FR 59087). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

1. PG&E Serial Letter No. DCL-05-098, "License Amendment Request 05-03, Request for Amendment to Recapture Low-Power Testing Time," August 23, 2005.
2. Letter from A. B. Wang (NRC) to J. S. Keenan, "Diablo Canyon Power Plant, Units 1 and 2 - Request for Additional Information on License Amendment Request for Recovery of Low-Power Testing Time - Impact on Reactor Vessel Integrity Assessments (TAC NOS. MC8206 and MC8207)," January 27, 2006.
3. PG&E Serial Letter No. DCL-06-045, "Response to Request for Additional Information Regarding License Amendment Request 05-03, 'Request for Amendment to Recapture Low-Power Testing Time,'" April 6, 2006.
4. Appendix G to Title 10, *Code of Federal Regulations*, "Fracture Toughness Requirements."
5. Paragraph §50.36 of Part 50 to Title 10, *Code of Federal Regulations* (10 CFR 50.36), "Technical specifications."
6. Generic Letter No. 96-03, "Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits."

7. Paragraph §50.61 of Part 50 to Title 10, *Code of Federal Regulations* (10 CFR 50.61), "Fracture toughness requirements for protection against pressurized thermal shock events."
8. Appendix H to Title 10, *Code of Federal Regulations*, "Reactor Vessel Material Surveillance Program Requirements."
9. American Society for Testing and Materials Designation E185, "Standard Practice for Conducting Surveillance Tests for Light-Water Cooled Nuclear Power Reactor Vessels": 1982 Edition (ASTM E185-82), July 1, 1982; 1973 Edition (ASTM E185-73), March 1, 1973; 1970 Edition (ASTM E185-70),
10. PG&E Serial Letter No. DCL-92-072, "Diablo Canyon Unit 1, Supplemental Reactor Vessel Radiation Surveillance Program," March 31, 1992..
11. Letter from H. Rood (NRC) to G. M. Rueger (PG&E), "Evaluation of Diablo Canyon Unit 1 Supplemental Reactor Vessel Radiation Surveillance Program (TAC No. M83285)," September 4, 1992.
12. WCAP-15858, Revision 0, "Analysis of Capsule V from Pacific Gas and Electric Company Diablo Canyon Unit 1 Reactor Vessel Radiation Surveillance Program," January 2003.
13. PG&E Serial Letter No. DCL-97-178, "Diablo Canyon Unit 2, Revision to Reactor Vessel Surveillance Capsule Withdrawal Schedule," October 22, 1997.
14. Letter from S. D. Bloom (NRC) to G. M. Rueger (PG&E), "Pacific Gas & Electric Company's Revision to the Reactor Vessel Surveillance Capsule Withdrawal Schedule for Diablo Canyon Unit No. 2 (TAC No. M99917)," February 10, 1998.
15. WCAP-15423, Revision 0, "Analysis of Capsule V from Pacific Gas and Electric Company Diablo Canyon Unit 2 Reactor Vessel Radiation Surveillance Program," September 2000.
16. Letter from D. Holland (NRC) to G. M. Rueger (PG&E), "Diablo Canyon Power Plant, Unit 1 (TAC No. MB5796) and Unit 2 (TAC No. MB5797) - Issuance of Amendment Revising Technical Specification 5.6.6 - Reactor Coolant System Pressure Temperature Limits Report," May 13, 2004.

Principal Contributor: James Medhoff

Date: July 17, 2006

Diablo Canyon Power Plant, Units 1 and 2

cc:

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