

June 13, 2006

L-PI-06-041
10 CFR 50.73

U S Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant Unit 1
Docket 50-282
License No. DPR-42

LER 1-06-01, Unit 1 Reactor Trip

The Licensee Event Report (LER) for this occurrence is attached. Nuclear Management Company, LLC (NMC) notified the NRC of this event, as required by 10 CFR 50.72(b)(2)(iv)(B) and 10 CFR 50.72(b)(3)(iv)(A), on April 14, 2006. Please contact us if you require additional information related to this event.

Summary of Commitments

This letter contains one new commitment and no revisions to existing commitments:

NMC will submit a supplement to this LER after the associated Root Cause Evaluation has been completed.



Thomas J. Palmisano
Site Vice President, Prairie Island Nuclear Generating Plant
Nuclear Management Company, LLC

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Prairie Island, USNRC
Resident Inspector, Prairie Island, USNRC
Glenn Wilson, State of Minnesota

ENCLOSURE

LICENSEE EVENT REPORT 1-06-01

3 pages follow

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0066), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1) Prairie Island Nuclear Generating Plant Unit 1	DOCKET NUMBER (2) 05000 282	PAGE (3) 1 of 3
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TITLE (4)
Unit 1 Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
4	14	06	06	-- 01 --	0	6	13	06	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply) (11)							
POWER LEVEL (10)		100	20.2201(b)			20.2203(a)(3)(ii)			50.73(a)(2)(ii)(B)	50.73(a)(2)(ix)(A)
			20.2201(d)			20.2203(a)(4)			50.73(a)(2)(iii)	50.73(a)(2)(x)
			20.2203(a)(1)			50.36(c)(1)(i)(A)		X	50.73(a)(2)(iv)(A)	73.71(a)(4)
			20.2203(a)(2)(i)			50.36(c)(1)(ii)(A)			50.73(a)(2)(v)(A)	73.71(a)(5)
			20.2203(a)(2)(ii)			50.36(c)(2)			50.73(a)(2)(v)(B)	OTHER Specify in Abstract below or in NRC Form 366A
			20.2203(a)(2)(iii)			50.46(a)(3)(ii)			50.73(a)(2)(v)(C)	
			20.2203(a)(2)(iv)			50.73(a)(2)(i)(A)			50.73(a)(2)(v)(D)	
			20.2203(a)(2)(v)			50.73(a)(2)(i)(B)			50.73(a)(2)(vii)	
			20.2203(a)(2)(vi)			50.73(a)(2)(i)(C)			50.73(a)(2)(viii)(A)	
			20.2203(a)(3)(i)			50.73(a)(2)(ii)(A)			50.73(a)(2)(viii)(B)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Jeff Kivi	TELEPHONE NUMBER (Include Area Code) 651.388.1121
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		
X	YES (If yes, complete EXPECTED SUBMISSION DATE).	NO		MONTH	DAY	YEAR
				7	31	06

ABSTRACT

On April 14, 2006, Prairie Island Nuclear Generating Plant (PINGP) Unit 1 was operating at 100% power. At approximately 1425 CDT, an 11 FEEDWATER PUMP LOCKED OUT annunciator was received in the control room. Unit 1 operators responded per the alarm response procedure and manually tripped the reactor. The trip was uncomplicated and all systems operated as expected and operator response and recovery actions were as expected.

Initial investigation of the cause of the 11 Feedwater Pump trip determined the feedwater pump tripped because the 11 Condensate Pump had tripped. Troubleshooting identified the cause of the condensate pump trip to be a ground between the 11 Condensate Pump and Bus 13. The motor was shipped off site for repairs and the stator windings were found to have grounded. A root cause evaluation of this event is ongoing – a supplement to this LER will be submitted after the root cause evaluation is completed.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
Prairie Island Nuclear Generating Plant Unit 1	05000282	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 of 3
		06	-- 01	-- 0	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION

On April 14, 2006, Prairie Island Nuclear Generating Plant (PINGP) Unit 1 was operating at 100% power. At approximately 1425 CDT, an 11 FEEDWATER¹ PUMP² LOCKED OUT annunciator was received in the control room. Unit 1 operators responded per the alarm response procedure and manually tripped the reactor. The auxiliary feedwater³ pumps started automatically in response to the low narrow range steam generator level, which was caused by shrink resulting from the reactor trip. The trip was uncomplicated and all systems operated as expected and operator response and recovery actions were as expected.

EVENT ANALYSIS

The trip of the Unit 1 reactor and the actuation of the auxiliary feedwater system are required to be reported per 10 CFR 50.73(a)(2)(iv)(A).

Impact on Safety System Functional Failure Performance Indicator

The affected condensate and feedwater pumps have no active safety function. All other systems performed as expected during the event. Therefore, this event does not represent a loss of safety function. Consequently, this event is not reportable per 10CFR 50.73(a)(2)(v).

SAFETY SIGNIFICANCE

The plant was stabilized in Mode 3 after the trip with all systems performing as expected in response to the reactor trip. Therefore, this event did not affect the health and safety of the public.

CAUSE

Initial investigation of the cause of the 11 Feedwater Pump trip determined the feedwater pump tripped because the 11 Condensate Pump had tripped. Troubleshooting identified the cause of the condensate pump trip to be a ground between the 11 Condensate Pump and Bus 13. The condensate pump motor was determined and a zero ohm ground was measured on the motor. The motor was shipped off site for repairs and the stator windings were found to have grounded. NMC is

¹ EIS System Code: SJ
² EIS Component Identifier: P
³ EIS System Code: BA

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

conducting a Root Cause Evaluation (RCE) to determine the cause of the condensate pump motor ground; after the RCE is completed, a supplement to this LER will be submitted.

CORRECTIVE ACTION

Corrective actions associated with this event will be identified as part of the RCE (currently in progress). After the RCE is completed, a supplement to this LER will be submitted that includes the corrective actions identified by the RCE.

PREVIOUS SIMILAR EVENTS

Both Unit 1 and Unit 2 have experienced unplanned reactor trips in the past, however, the most recent unplanned reactor trip of either unit occurred in 2002.