



June 8, 2006
NRC:06:025

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Clarification of Labeling Conventions in BAW-2251A, "Demonstration of the Management of Aging Effects for the Reactor Vessel"

Reference: 1. Letter, Christopher I. Grimes (NRC) to David J. Firth (B&W Owners Group), "Acceptance for Referencing of Generic License Renewal Program Topical Report Entitled 'Demonstration of the Management of Aging Effects for the Reactor Vessel,' BAW-2251, June 1996," April 26, 1999.

In Reference 1, the NRC issued a safety evaluation for BAW-2251A, "Demonstration of the Management of Aging Effects for the Reactor Vessel." AREVA NP Inc. has been made aware of inconsistent labeling conventions of the reactor vessel plates and forgings used in some of the figures in the report as compared to the labeling convention used in Appendix A of the report.

Attachment A provides a summary of the necessary labeling clarifications.

If you have any questions, please do not hesitate to contact me.

Sincerely,

A handwritten signature in cursive script that reads "Ronnie L. Gardner".

Ronnie L. Gardner, Manager
Site Operations and Regulatory Affairs
AREVA NP

Enclosures

cc: M.A. Mitchell
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D045

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Attachment A

Summary of Labeling Clarifications

1. The Nozzle Belt Forging referred to in Appendix A Tables A-1 through A-5 corresponds to the Upper Shell Forging in Figures 2-2 through 2-6.
2. The Upper Shell Forging or Upper Shell Plate referred to in Appendix A Tables A-1 through A-5 corresponds to the Intermediate Shell in Figures 2-2 through 2-6.
3. The Intermediate Shell Plate referred to in Appendix A Table A-2 corresponds to the unlabeled short plates shown in Figure 2-3 which are between the Upper Shell Forging and the Intermediate Shell. Both plates at this elevation are of the same heat.
4. The "LS to Dutch. Circ. Weld" referred to in Appendix A Tables A-1 through A-5 corresponds to the weld between the Lower Shell and the Transition Forging in Figures 2-2 through 2-6.