

June 15, 2006

Mr. Dennis Koehl  
Site Vice President  
Point Beach Nuclear Plant  
Nuclear Management Company, LLC  
6610 Nuclear Road  
Two Rivers, WI 54241-9516

SUBJECT: POINT BEACH NUCLEAR PLANT, UNIT 1 - REQUEST FOR ADDITIONAL  
INFORMATION RELATED TO THE FALL 2005 STEAM GENERATOR TUBE  
INSPECTION REPORT (TAC NOS. MD0242)

Dear Mr. Koehl:

By letter to the Nuclear Regulatory Commission (NRC) dated February 21, 2006, Nuclear Management Company, LLC submitted its inspection report for the fall 2005 refueling outage (U1R29) for the steam generator tube inspections at Point Beach Nuclear Plant, Unit 1.

The NRC staff is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested is addressed in the enclosure to this letter. During a discussion with Mr. J. Gadzala of your staff on June 12, 2006, it was agreed that you would provide a response within 30 days from the date of this letter.

The NRC staff considers that timely responses to requests for additional information help ensure sufficient time is available for staff review and contribute toward the NRC's goal of efficient and effective use of staff resources. If circumstances result in the need to revise the requested response date, please contact me at (301) 415-2296.

Sincerely,

**/RA/**

Carl F. Lyon, Project Manager  
Plant Licensing Branch III-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-266

Enclosure:  
Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

STEAM GENERATOR TUBE INSERVICE INSPECTION REPORT FOR U1R29

POINT BEACH NUCLEAR PLANT, UNIT 1

DOCKET NUMBER 50-266

In reviewing Nuclear Management Company, LLC's submittal regarding the Point Beach Nuclear Plant (Point Beach), Unit 1, fall 2005 (U1R29) steam generator tube inspection report dated February 21, 2006, the NRC staff has determined that the following information is needed in order to complete its review:

1. Indicate if the wear indications identified in steam generator (SG) A were inspected with both bobbin and +Point™ probes. If the wear indications were not inspected with a +Point™ probe, discuss how you concluded the indications were wear.
2. In Table 1, the historical comparison of wear indications in the anti-vibration bar area of SG A were provided. The percent through-wall (TW) was given for Tube Row 40 Column 42 for 1995, 2001, 2004, and 2005. Discuss why the percent TW for this tube was not given for 1998.
3. Confirm that a visual inspection was performed (either during Point Beach, Unit 1 Refueling Outage 28 or Point Beach, Unit 1 Refueling Outage 29) at the locations where wear indications were reported at or near the top of the tubesheet and that these inspections indicated that no loose parts or foreign objects were present at these locations.
4. Regarding the "signature tubes," discuss whether both low and high row tubes were screened for potentially having elevated susceptibility to cracking. In addition, discuss the extent to which these "signature tubes" were also inspected with a rotating probe in the tubesheet region.
5. Several dents and dings were reported for SG A. Clarify the difference between a dent and a ding. If any of the dents and dings are service induced, discuss the cause. In addition, discuss whether the size of the dents and dings are changing with time.
6. Confirm that no known loose parts or foreign objects were left in SG A. If any were, discuss the basis for leaving these parts or objects in service (i.e., with regards to tube integrity).
7. It was indicated that no significant degradation of the swirl vanes, moisture separator, or feeding J-nozzles were found. Discuss whether any degradation was found (other than the minor weld burn noted in two closely spaced J-nozzles).

Enclosure

8. Discuss the nature of the SG wrapper modification that led to the plugging of two tubes in 1988.
9. Discuss the nature (chemical constituents) of the sludge removed from the SG and the scale on the tubes. If copper (or other species that can affect eddy current data quality) is present, discuss the quality of the eddy current data and whether techniques have been qualified given the levels of interference that exist in your data.

Point Beach Nuclear Plant, Units 1 and 2

cc:

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