

June 16, 2006

Mr. J. A. Stall
Senior Vice President, Nuclear and
Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT PLANT, UNIT NO. 3 - REVIEW OF THE STEAM
GENERATOR TUBE INSPECTION REPORTS FOR THE FALL 2004
OUTAGE (TAC NO. MC8112)

Dear Mr. Stall:

By letters dated October 18, 2004 (Agencywide Documents Access and Management System Accession Number ML060040111), February 28, 2005 (ML050670045 and ML050670057), June 17, 2005 (ML051810242), and February 27, 2006 (ML060760370), Florida Power & Light Company (the licensee) submitted information related to the steam generator (SG) tube inspections at the Turkey Point Nuclear Plant, Unit 3, for the fall 2004 outage. The results of the U.S. Nuclear Regulatory Commission (NRC) staff review of these documents is summarized below.

Turkey Point Unit 3 has three Westinghouse Model 44F SGs. The SGs utilize a hydraulic tubesheet expansion method and have 405 stainless steel tube support plates with broached quatrefoil holes. These SGs began operation in 1982, following SG replacement, and each has thermally treated Alloy 600 tubing.

The licensee provided the scope, extent, methods and results of their SG tube inspections in the documents referenced above. In addition, the licensee described corrective actions (i.e., tube plugging) taken in response to the inspection findings.

The licensee indicated that a wear indication in SG3B at the second broached tube support location is due to a probable loose part. The licensee clarified that this wear indication and suspected loose part were first detected in the October 2004 inspection, but an analysis of previous data dating back 14 years showed that the indication was present at lower levels. The licensee concluded that the wear growth rate is low based on the historical data. Given the shallow measured depth, long service history without significant change in voltage, and lack of physical evidence of wear to adjacent tubes, the licensee determined that the loose part is most likely small and immobile. The tube was left in service and will be tracked during future inspections.

Based on a review of the information provided, the NRC staff concludes that the licensee provided the information required by their technical specifications. In addition, the staff concludes that there are no technical issues that warrant follow-up action at this time, since the

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inspections appear to be consistent with the objective of detecting potential tube degradation and the inspection results appear to be consistent with industry operating experience at similarly designed and operated units.

This completes the NRC staff's efforts under TAC No. MC8112. If you have any questions regarding this matter, please contact me at 301-415-3974.

Sincerely,

/RA/

Brendan T. Moroney, Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-250

cc: See next page

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Mr. J. A. Stall
Florida Power and Light Company

cc:

Mr. William E. Webster
Vice President, Nuclear Operations
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

M. S. Ross, Managing Attorney
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

Marjan Mashhadi, Senior Attorney
Florida Power & Light Company
801 Pennsylvania Avenue, NW.
Suite 220
Washington, DC 20004

T. O. Jones, Site Vice President
Turkey Point Nuclear Plant
Florida Power and Light Company
9760 SW. 344th Street
Florida City, FL 33035

County Manager
Miami-Dade County
111 Northwest 1 Street, 29th Floor
Miami, Florida 33128

Senior Resident Inspector
Turkey Point Nuclear Plant
U.S. Nuclear Regulatory Commission
9762 SW. 344th Street
Florida City, Florida 33035

Mr. William A. Passetti, Chief
Department of Health
Bureau of Radiation Control
2020 Capital Circle, SE, Bin #C21
Tallahassee, Florida 32399-1741

Mr. Craig Fugate, Director

TURKEY POINT PLANT

Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Michael O. Pearce
Plant General Manager
Turkey Point Nuclear Plant
Florida Power and Light Company
9760 SW. 344th Street
Florida City, FL 33035

Walter Parker
Licensing Manager
Turkey Point Nuclear Plant
9760 SW 344th Street
Florida City, FL 33035

Mark Warner, Vice President
Nuclear Operations Support
Florida Power and Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

Mr. Rajiv S. Kundalkar
Vice President - Nuclear Engineering
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420