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MAY 15 2006

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Stop OP1-17
Washington, DC 20555

**SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 50-387/2006-002-00
LICENSE NO. NPF-14
PLA-6043**

Docket No. 50-387

Attached is Licensee Event Report 50-387/2006-002-00. This event was determined to be reportable under 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications.

On March 23, 2006 during a refuel outage it was discovered that a required pre-modification Local Leak Rate test (LLRT) on a containment penetration had not been performed as required by the station's containment leakage rate program. This constituted a missed surveillance in accordance with the plant's Technical Specification surveillance requirement SR 3.6.1.1.1. Because the original configuration no longer existed, the containment leakage rate prior to the modification could not be quantified.

This event did not result in any actual adverse consequences to the health and safety of the public.

No commitments are associated with this LER.

A handwritten signature in black ink, appearing to read "R. Saccone", is written over the printed name.

Robert Saccone
Vice President – Nuclear Operations

Attachment

JE22

cc: Mr. S. Collins
Regional Administrator
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Alan Blamey
Sr. Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 35
Berwick, PA 18603-0035

Mr. R. Osborne
Allegheny Electric Cooperative
P. O. Box 1266
Harrisburg, PA 17108-1266

Mr. R. R. Janati
Bureau of Radiation Protection
Rachel Carson State Office Building
P. O. Box 8469
Harrisburg, PA 17105-8469

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME Susquehanna Steam Electric Station Unit 1

2. DOCKET NUMBER
05000387

3. PAGE
1 OF 3

4. TITLE Missed Technical Specification surveillance requirement

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
3	23	2006	2006	002	00	5	15	2006	FACILITY NAME	DOCKET NUMBER

9. OPERATING MODE 5	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)			
	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)
10. POWER LEVEL 0%	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME Dayne R. Brophy, Senior Engineer – Nuclear Regulatory Affairs	TELEPHONE NUMBER (Include Area Code) (570) 542-3365
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED

YES (If yes, complete 15. EXPECTED SUBMISSION DATE) NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On March 10, 2006 with Unit 1 in Mode 5 for a refuel outage, Maintenance workers breached a containment electrical penetration to install a cable as part of a planned modification. On March 23, 2006 during review of a 10CFR 50 Appendix J post-modification local leak rate test (LLRT) for the penetration, it was discovered that a pre-modification LLRT had not been performed as required by the station's Containment Leakage Rate Testing Program. This constituted a missed surveillance in accordance with plant Technical Specification surveillance requirement SR 3.6.1.1.1 and is reportable under 10CFR 50.73(a)(2)(i)(B). The apparent causes of the event were a lack of clear understanding of the procedural requirement by the program's engineer and the absence of a peer review. Because the penetration had an excellent history of leak rate results and was on an extended testing interval, it was reasonable to conclude that a pre-modification LLRT would have successfully passed its acceptance criteria. Corrective actions include the establishment of a pre-outage peer review of the outage plan and associated LLRT testing requirements for future outages.

This event did not result in any actual adverse consequences to the health and safety of the public.

LICENSEE EVENT REPORT (LER)

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Susquehanna Steam Electric Station Unit 1	05000387	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
		2006	-- 002	-- 00	

17. NARRATIVE (If more space is required, use additional copies of NRC Form 366A)

PLANT CONDITIONS AT TIME OF EVENT

Unit 1, Mode 5, 0%
Unit 2, Mode 1, 100%

EVENT DESCRIPTION

On March 10, 2006 with Unit 1 in Mode 5 for a refuel outage, Maintenance workers breached a containment electrical penetration (EIS:PEN) to install coax cable as part of a planned modification. The station Containment Leakage Rate Testing Program requires a LLRT prior to performing any maintenance or modification to Appendix J penetrations that may alter the leakage rate of the penetration during an Integrated Leak Rate Test (ILRT) outage. An ILRT was planned as part of the outage. Because the existing conditions no longer existed, the pre-modification leakrate for the penetration could no longer be quantified. In accordance with plant Technical Specification surveillance requirement SR 3.6.1.1.1, this constituted a missed surveillance and is reportable under 10CFR 50.73(a)(2)(i)(B) as a violation of Tech Specs.

This event did not result in any actual adverse consequences to the health and safety of the public.

CAUSE OF THE EVENT

The apparent cause was a mis-application of the Appendix J Program requirements by the program engineer. During a pre-outage review of the modification, the program engineer made an assessment that the leak rate values from the last performed LLRT could be utilized for the pre-modification leak rate. This decision was based on the fact that the penetration was a passive barrier and had an excellent history for leakrate results (it was on an extended test frequency). The program requirements for an ILRT outage clearly mandate a pre-modification LLRT to confirm the integrity of the penetration, even for penetrations on an extended test frequency. This pre-outage review also did not have a peer review to evaluate the scope of outage work activities.

ANALYSIS / SAFETY SIGNIFICANCE

Actual Consequences

There were no actual consequences to this event. Based on previous LLRT results, the penetration has historically met its administrative limits and was on an extended test frequency. The reactor was shutdown in mode 5 when the event occurred. The penetration was not required to be operable at the time of the event.

Potential Consequences

The potential existed that the penetration would have been degraded, resulting in a test failure. Failure to maintain the primary containment leakage limits within the 10CFR 50 Appendix J requirements could result in exceeding the calculated radiological limits under 10CFR 100.

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CORRECTIVE ACTIONS

Completed Actions

- 1) Because the penetration was disturbed during the modification, the periodic testing requirement was re-established at a 30 month interval.

Planned Actions

- 1) Review scheduled work on similar containment penetrations governed by the Appendix J Program for the 2007 Unit 2 Refuel Outage.
- 2) Provide for a peer review of final LLRT/ILRT outage plans and testing requirements prior to future outages.

ADDITIONAL INFORMATION

Past similar events: Docket 50-387 LER 00-001-00