

August 2, 2006

Mr. Karl W. Singer
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2 — ISSUANCE OF
AMENDMENTS REGARDING TECHNICAL SPECIFICATION CHANGES TO
CYCLIC AND TRANSIENT LIMITS WITH DESIGN FEATURES REVISION
(TAC NOS. MC8532 AND MC8533) (TS 05-02)

Dear Mr. Singer:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 309 to Facility Operating License No. DPR-77 and Amendment No. 298 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated September 30, 2005 (TVA-SQN-TS-05-02).

The amendments modify the Technical Specification (TS) Section 5.0, "Design Features," to be more consistent with the content of NUREG-1431, Revision 3, "Improved Standard TS for Westinghouse Plants," and do not diminish the level of safety found in the current TSs.

A copy of the staff's Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

Enclosures: 1. Amendment No. 309 to
License No. DPR-77
2. Amendment No. 298 to
License No. DPR-79
3. Safety Evaluation

cc w/enclosures: See next page

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Mr. Karl W. Singer
Tennessee Valley Authority

SEQUOYAH NUCLEAR PLANT

cc:

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TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 309
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated September 30, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

- (1) Technical Specifications

- The Technical Specifications contained in Appendices A and B, as revised through Amendment No .309, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented no later than 45 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/original signed by L.Raghavan for/

Michael L. Marshall, Jr., Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 2, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 309

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Replace page 3 of Operating License No. DPR-77 with the attached page 3.

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the area of change.

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Index Page XVI

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- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis, instrument calibration or associated with radioactive apparatus or components; and
 - (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the Sequoyah and Watts Bar Unit 1 Nuclear Plants.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The Tennessee Valley Authority is authorized to operate the facility at reactor core power levels not in excess of 3455 megawatts thermal.
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 309, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Initial Test Program

The Tennessee Valley Authority shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Tennessee Valley Authority's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:
 - a. Elimination of any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;
 - b. Modification of test objectives, methods or acceptance criteria for any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;
 - c. Performance of any test at power level different from there described; and

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.298
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated September 30, 2005, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 298, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented no later than 45 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

original signed by L.Raghavan for/

Michael L. Marshall, Jr., Chief
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 2, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 298

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Replace page 3 of Operating License No. DPR-79 with the attached page 3.

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the area of change.

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- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the Sequoyah and Watts Bar Unit 1 Nuclear Plants.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

The Tennessee Valley Authority is authorized to operate the facility at reactor core power levels not in excess of 3455 megawatts thermal.
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 298, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Initial Test Program

The Tennessee Valley Authority shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Tennessee Valley Authority's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

 - a. Elimination of any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;
 - b. Modification of test objectives, methods or acceptance criteria for any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;
 - c. Performance of any test at power level different from there described; and

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 309 TO FACILITY OPERATING LICENSE NO. DPR-77
AND AMENDMENT NO. 298 TO FACILITY OPERATING LICENSE NO. DPR-79
TENNESSEE VALLEY AUTHORITY
SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-327 AND 50-328

1.0 INTRODUCTION

By application dated September 30, 2005 (Agencywide Documents Access and Management System Accession No. ML052870040), the Tennessee Valley Authority (TVA, the licensee) proposed amendments to the Technical Specifications (TSs) for Sequoyah Nuclear Plant (SQN), Units 1 and 2. The requested changes would modify TS Section 5.0 "Design Features," to be more consistent with the content of NUREG-1431, Revision 3, "Standard Technical Specifications for Westinghouse Plants," and do not diminish the level of safety found in the current SQN TSs.

The proposed changes would revise the specific sections of the SQN TSs "Design Features." The changes will remove figures identifying the site exclusion area, low population zone, site boundary gaseous and liquid effluents, and meteorological tower. A description of the site location will be added to the newly titled, "Site Location" section. The "Site Boundary" definition will be revised for consistency. SQN also proposes deletion of Containment Configuration and Design Pressure and Temperature Sections, as well as the Reactor Coolant System Design Pressure, and Temperature and Volume Sections. Lastly, SQN proposes moving the component cyclic or transient limit requirements to SQN TS 6.0, "Administrative Controls." Each of the proposed changes is conforming to NUREG-1431, Revision 3, "Standard Technical Specifications for Westinghouse Plants."

2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The licensee provides TSs to maintain the operational capability of structures, systems, and components required to protect the health and safety of the public. The staff of the Nuclear Regulatory Commission (NRC) used the regulatory requirements for TS changes set forth in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50.36 for this evaluation. The proposed changes would affect the "Design Features" section of the SQN TSs, which is regulated under 10 CFR 50.36(c)(4), *Design features*:

"Design features to be included are those features of the facility such as materials of construction and geometric arrangements, which, if altered or modified, would have a significant effect on safety and are not covered in categories described in paragraphs (c) (1), (2), and (3) of this section."

Originally, the requirements of 10 CFR 50.36 established the categories of items for inclusion in the TSs, but not the particular requirements for the TSs of each individual plant. The NRC provided guidance for the specific contents of the TSs in the "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" (Final Policy Statement), 58 FR 39132 (July 22, 1993). In particular, the Final Policy Statement allows items that are not required by 10 CFR 50.36 to be moved from the TSs to licensee-controlled documents. The licensee may further change the licensee-controlled document according to 10 CFR 50.59. The licensee-controlled Final Safety Analysis Report (FSAR) meets the NRC expectations for a licensee-controlled document. It is an acceptable document to which specifications that do not meet the 10 CFR 50.36 applicability requirements can be moved. The proposed revision to the SQN TSs for Units 1 and 2 will either delete or relocate several design features that are not required in the TSs to the SQN FSAR.

There are two classes of TSs changes: (1) Changes needed to reflect modifications to the design basis (TSs are derived from the design basis), and (2) voluntary changes that take advantage of updates in NRC policy and guidance with respect to TSs over time. This amendment deals with only the second class of changes. For Westinghouse plants, the NRC staff uses NUREG-1431 "Standard Technical Specifications Westinghouse Plants," Revision 3, to determine the acceptability of TSs changes. This NUREG is an accumulation of generically approved guidance on TSs called the improved standard TS (ISTS).

Within this general framework, licensees may remove material from their TSs on two conditions: (1) the material is not required to be in the TSs based on the staff interpretation of 10 CFR 50.36, including judgments about how much detail is required in the TSs, and (2) there exist suitable alternative regulatory controls for the material. Licensees may revise the remaining TSs to adopt current ISTS format and content if plant-specific review supports a finding of continued adequate safety because of one of the following reasons: (1) the change is editorial, administrative or provides clarification (i.e., no requirements are materially altered), (2) the change is more restrictive than the licensee's current requirement, or (3) the change is less restrictive than the licensee's current requirement, but nonetheless still affords adequate assurance of safety when judged against current regulatory standards. The detailed application of this general framework and additional specialized guidance are discussed in Section 3.0 in the context of specific proposed changes.

The described changes proposed for the amendment include removing current site information regarding exclusion area, low population zone, and site boundary for gaseous and liquid effluents from the TSs "Design Features" section. The title of Section 5.1 "Site," will be renamed, "Site Location," and descriptive language relating to the site location will be added for consistency with NUREG-1431, Revision 3. Information regarding the design features for containment, reactor coolant system (RCS), and meteorological tower location will be deleted. The "Component Cyclic or Transient Limit" section is moved to the TS "Administrative Control" section. A referral in the site boundary definition is removed and the TS index is modified for consistency with the revised design features.

The bulk of the information changed by the deletion or revision of the above TSs will continue to be retained in various sections of the SQN FSAR. The SQN FSAR is a document controlled by 10 CFR 50.59 that provides an appropriate level of review and approval for the revision of requirements that are important to safety, but do not satisfy the 10 CFR 50.36 TS requirements.

The proposed revision will also maintain an improved level of consistency with NUREG-1431, which does not contain requirements for the TSs proposed for revision.

3.0 TECHNICAL EVALUATION

The proposed changes will relocate the identified TSs to the licensee-controlled FSAR consistent with the requirements of 10 CFR 50.36. The NRC staff has reviewed the licensee's submittal and finds that relocation of these requirements to a licensee-controlled document is acceptable in that the information slated for relocation is not within the scope of the criteria contained in 10 CFR 50.36(c)(4), and changes to licensee-controlled documents will be adequately controlled by 10 CFR 50.59, as applicable.

3.1 Proposed TS Changes to Section 5.1 "Site"

The licensee proposes to retitle the TSs Design Features section titled, "Site," as "Site Location," and to add a description of the site location that is consistent with the text in the SQN FSAR. In addition, the licensee proposes deleting the information concerning the SQN site exclusion area, low population zone, and site boundary for gaseous and liquid effluents. This entails elimination of both the exclusion area figure and low population zone figure. This change also requires removal of a figure referenced within the site boundary definition. These changes are considered administrative in nature and conforming to NUREG-1431, which does not include site or plant maps. The information contained in the current TSs is discussed in Chapter 2 of the SQN FSAR. The staff finds that reliance on the FSAR content is acceptable because the 10 CFR 50.59 program would control a change to this information. In addition, the inclusion of site and plant area maps and associated text are not requirements of 10 CFR 50.36(c)(4), Design Features of TSs.

3.1.1 Site

To be consistent with NUREG-1431, the title of Section 5.1 "Site," is renamed, "Site Location," and the following site location description is added:

"The Sequoyah Nuclear Plant is located on a site near the geographical center of Hamilton County, Tennessee, on a peninsula on the western shore of Chickamauga Lake at Tennessee River mile (TRM) 484.5. The Sequoyah site is approximately 7.5 miles northeast of the nearest city limit of Chattanooga, Tennessee, 14 miles west-northwest of Cleveland, Tennessee, and approximately 31 miles south-southwest of TVA's Watts Bar Nuclear Plant."

No physical modifications to the facility are required to carry out this proposed change. Thus, this change would not affect the materials of construction or geometric arrangements of features of the facility and are not requirements of 10 CFR 50.36(c)(4).

3.1.2 Exclusion Area

The subsection entitled, "Exclusion Area," will retain its title, however, the text of the subsection and the associated figure, Figure 5.1-1 "Exclusion Area," will be removed and replaced with text that indicates their deletion. A description of the site exclusion area is contained in Chapter 2 of the SQN FSAR and controlled by 10 CFR 50.59. This change is consistent with the guidance

of NUREG-1431. No physical modifications to the facility are required to implement this proposed change. Thus, this change would not affect the materials of construction or geometric arrangements of features of the facility and are not requirements of 10 CFR 50.36(c)(4).

3.1.3 Low Population Zone

The subsection entitled, "Low Population Zone," will retain its title, however, the text of the subsection and the associated figure, Figure 5.1-2 "Low Population Zone," will be removed and replaced with text that indicates their deletion. Provisions for protection of this area were considered during the development of the site emergency plan. Similar information regarding the site low population zone is in SQN FSAR, Chapter 2 and is controlled by the 10 CFR 50.59 program. This change is consistent with NUREG-1431 and no physical modifications to the facility are required to implement this proposed change. Thus, this change would not affect the materials of construction or geometric arrangements of features of the facility and are not requirements of 10 CFR 50.36(c)(4).

3.1.4 Site Boundary for Gaseous Effluents

The subsection entitled, "Site Boundary for Gaseous Effluents," will retain its title, however, the text of the subsection will be removed and replaced with text that indicates its deletion. A discussion of the gaseous effluents' site boundary is found in SQN FSAR, Chapter 2 and is controlled by the 10 CFR 50.59 program. This change is consistent with NUREG-1431. The proposed change does not modify system, structure, component, or program related to control of gaseous effluents. In addition, it does not increase limits regarding the release of, or monitoring of, gaseous effluents. Thus, this change would not affect the materials of construction or geometric arrangements of features of the facility and are not required by 10 CFR 50.36(c)(4).

3.1.5 Site Boundary for Liquid Effluents

The subsection entitled, "Site Boundary for Liquid Effluents," will retain its title, however, the text of the subsection will be removed and replaced with text that indicates its deletion. A discussion of the liquid effluents' site boundary is found in SQN FSAR, Chapter 2 and is controlled by the 10 CFR 50.59 program. This change is consistent with NUREG-1431. The proposed change does not modify any system, structure, component, or program related to control of liquid effluents. It does not increase limits regarding the release of, or monitoring of, liquid effluents. Thus, this change is not required by 10 CFR 50.36(c)(4).

3.2 Proposed TS Changes to Section 5.2 "Containment"

The licensee proposes deleting the containment configuration, and design pressure and temperature information. This change is considered administrative in nature and conforming to NUREG-1431. The SQN FSAR, Chapter 3 contains similar information and reliance on the FSAR content is acceptable because the 10 CFR 50.59 program would control a change to this information. Some detailed information is not described in the FSAR and will require relocation to ensure fidelity.

3.2.1 Configuration

The subsection entitled, "Configuration," will retain its title, however, the text of the subsection will be removed and replaced with text that indicates its deletion. A discussion of the containment configuration and nominal dimensions is found in the SQN FSAR, Chapter 3 and is controlled by the 10 CFR 50.59 program. This proposed change does not physically modify the containment structure or assisting systems, structure, and components. It does not change calculations related to the containment design. Additionally, the TSs limiting conditions for operations (LCOs) are maintained for the containment systems with no proposed change. This change would not affect the materials of construction or geometric arrangements of features of the facility and is not required by 10 CFR 50.36(c)(4).

3.2.2 Design Pressure and Temperature

The subsection entitled, "Design Pressure and Temperature," will retain its title. However, the text of the subsection will be removed and replaced with text that indicates its deletion. This TS section ensures the steel containment vessel is designed and maintained for a maximum internal pressure and temperature. The containment vessel is designed to accommodate the maximum internal pressure and temperature calculated to occur following a loss-of-coolant accident (LOCA) (i.e., design basis accident).

TS Section 5.2.2 states "The steel containment is designed and shall be maintained for a maximum internal pressure of 12 psig and a temperature of 250EF." The containment design temperature and pressure are not being changed and the limiting values will be retained in the SQN FSAR. Thus, its elimination is considered administrative in nature and does not result in a change of the margin of safety to the containment design.

The staff has previously determined that the containment design temperature and pressure are not needed in the TSs, as found in NUREG-1431, and therefore do not meet the inclusion requirements of 10 CFR 50.36(c)(4). Furthermore, this proposed change does not physically modify the containment structure or assisting systems, structure, and components, nor does it change calculations related to the containment design. Additionally, the TS LCOs are maintained for the containment systems with no proposed change. This change would not affect the materials of construction or geometric arrangements of features of the facility and the containment design pressure and temperature are not required by 10 CFR 50.36(c)(4).

3.3 Proposed TS Changes to Section 5.4 "Reactor Coolant System"

The licensee proposes deleting the RCS design pressure and temperature, and volume information from the TS's Design Features. This change conforms with NUREG-1431, which does not contain this information. The SQN FSAR contains similar information, as well as the code requirements in FSAR, Section 5.2. Reliance on the FSAR content is acceptable because the 10 CFR 50.59 program would control a change to this information. As described previously, some detailed information is not described in the FSAR and will require relocation to ensure fidelity. Additionally, SQN TS LCO, Section 3.4 provides requirements to ensure safe reactor coolant system operation.

3.3.1 Design Pressure and Temperature

The subsection entitled, "Design Pressure and Temperature," will retain its title, however, the text of the subsection will be removed and replaced with text that indicates its deletion. This information is retained in the SQN FSAR, Section 5.2. Removing the section regarding RCS design pressure and temperature does not result in a loss of margin for normal operation or anticipated operational occurrences. Moreover, it does not physically modify the reactor coolant pressure boundary (RCPB) or calculations related to the RCPB design. Additionally, the SQN TS LCOs for the RCS are maintained to ensure the safe continued operation. The RCS design pressure and temperature are not required by 10 CFR 50.36(c)(4).

3.3.2 Reactor Coolant System Volume

The subsection entitled, "Reactor Coolant System Volume," will retain its title. The text of the subsection will be removed and replaced with text that indicates its deletion. This information is retained in the SQN FSAR, Chapter 5. Removing the section regarding RCS volume does not result in a loss of margin for normal operation or anticipated operational occurrences. Moreover, it does not physically modify the RCPB or calculations related to the RCPB design. The RCS volume is not required by 10 CFR 50.36(c)(4).

3.4 Proposed TS Changes to Section 5.5 "Meteorological Tower Location"

The subsection entitled, "Meteorological Tower Location," will retain its title. The text of the subsection and the associated figure will be deleted and replaced with text showing the deletion. The associated figure, used to show the location of the meteorological tower, is the same as that for exclusion area and effluents boundaries. This change is considered administrative in nature and conforming to NUREG-1431. This information is retained in the SQN FSAR, Chapter 2. Reliance on the FSAR content is acceptable because the 10 CFR 50.59 program would control a change to this information. In addition, a figure showing the location of the meteorological tower does not meet the requirements of 10 CFR 50.35(c)(4).

3.4 Proposed TS Changes to Section 5.7 "Component Cyclic or Transient Limit"

The subsection entitled, "Component Cyclic or Transient Limit," will be relocated to a new Administrative Control Section 6.8.4.I "Component Cyclic or Transient Limit," and the language will be revised as follows to conform with NUREG-1431:

"This program provides control to track the FSAR, Section 5.2.1, cyclic and transient occurrences to ensure that components are maintained within the design limits."

This change will also move Table 5.7.1 "Component Cyclic or Transient Limits," to the SQN FSAR, Chapter 5. This change is considered administrative in nature since the requirements to track and maintain these limits remain in the SQN TSs. This change also is conforming to NUREG-1431, which is based upon an accumulation of NRC staff guidance pertaining to TSs.

3.5 Summary

The NRC staff has reviewed the licensee's proposal to delete or relocate the above TSs. The staff finds these changes consistent with 10 CFR 50.36 criteria and precedent established in

NUREG-1431, Revision 3. In addition, any changes made to information referenced in the SQN FSAR will be subject to the change control process of 10 CFR 50.59. On this basis, the NRC staff concludes that the proposed changes do not diminish the level of safety ensured by operation in accordance with the SQN TSs and are, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Nuclear Regulatory Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Nuclear Regulatory Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (70 FR 67752). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Nuclear Regulatory Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

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