

July 5, 2006

Mr. Mano K. Nazar
Senior Vice President and
Chief Nuclear Officer
Indiana Michigan Power Company
Nuclear Generation Group
One Cook Place
Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 (DCCNP-1 AND
DCCNP-2) - ISSUANCE OF AMENDMENTS REGARDING LEAKAGE
MONITORING PROGRAM (TAC NO. MD0494 AND MD0495)

Dear Mr. Nazar:

The Commission has issued the enclosed Amendment No. 294 to Renewed Facility Operating License No. DPR-58 for DCCNP-1 and Amendment No. 277 to Renewed Facility Operating License No. DPR-74 for DCCNP-2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated March 7, 2006.

The amendments revised Section 5.5.2, "Leakage Monitoring Program," of the units' TS, adding the Liquid Waste Disposal System, Waste Gas System, and Post-Accident Containment Hydrogen Monitoring System to the list of systems. The listing of these systems was inadvertently omitted from Section 5.5.2.

A copy of our related safety evaluation is enclosed. A Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Peter S. Tam, Senior Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures:

1. Amendment No. 294 to DPR-58
2. Amendment No. 277 to DPR-74
3. Safety Evaluation

cc w/encls: See next page

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Donald C. Cook Nuclear Plant, Units 1 and 2

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INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 294
License No. DPR-58

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated March 7, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to Renewed Facility Operating License DPR-58 and Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of its date of issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

L. Raghavan, Chief
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Renewed Operating License

Date of Issuance: July 5, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 294
RENEWED FACILITY OPERATING LICENSE NO. DPR-58
DOCKET NO. 50-315

Replace the following page of Renewed Facility Operating License No. DPR-58 with the attached revised page. The changed area is identified by a marginal line.

REMOVE

INSERT

3

3

Replace the following page of Appendix A, Technical Specifications, with the attached revised page. The changed area is identified by a marginal line.

REMOVE

INSERT

5.5-2

5.5-2

and radiation monitoring equipment calibration, and as fission detectors in amounts as required.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not to exceed 3304 megawatts thermal in accordance with the conditions specified therein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 294, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Less Than Four Loop Operation

The licensee shall not operate the reactor at power levels above P-7 (as defined in Table 3.3.1-1 of Specification 3.3.1 of Appendix A to this renewed operating license) with less than four reactor coolant loops in operation until (a) safety analyses for less than four loop operation have been submitted, and (b) approval for less than four loop operation at power levels above P-7 has been granted by the Commission by amendment of this license.

- (4) Indiana Michigan Power Company shall implement and maintain, in effect, all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated December 12, 1977, July 31, 1979, January 10, 1981, February 7, 1983, November 22, 1983, December 23, 1983, March 16, 1984, August 27, 1985

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 277
License No. DPR-74

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated March 7, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to Renewed Facility Operating License DPR-74 and Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of its date of issuance and shall be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

L. Raghavan, Chief
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Operating License

Date of Issuance: July 5, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 277
RENEWED FACILITY OPERATING LICENSE NO. DPR-74
DOCKET NO. 50-316

Replace the following page of Renewed Facility Operating License No. DPR-74 with the attached revised page. The changed area is identified by a marginal line.

REMOVE

3

INSERT

3

Replace the following page of Appendix A, Technical Specifications, with the attached revised page. The changed area is identified by a marginal line.

REMOVE

5.5-2

INSERT

5.5-2

radiation monitoring equipment calibration, and as fission detectors in amounts as required.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not to exceed 3468 megawatts thermal in accordance with the conditions specified therein and in Attachment 1 to the renewed operating license. The preoperational tests, startup and other items identified in Attachment 1 to this renewed operating license shall be completed. Attachment 1 is an integral part of this renewed operating license.

(2) Technical Specifications

The Technical Specifications contained in Appendix A and Appendix B, as revised through Amendment No. 277, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

- (a) Deleted by Amendment No. 76
- (b) Deleted by Amendment No. 2

(c) Leak Testing of Emergency Core Cooling System Valves

Indiana Michigan Power Company shall prior to completion of the first inservice testing interval leak test each of the two valves in series in the

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
AMENDMENT NO. 294 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-58
AND AMENDMENT NO. 277 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-74
INDIANA MICHIGAN POWER COMPANY
DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-315 AND 50-316

1.0 INTRODUCTION

By application to the U.S. Nuclear Regulatory Commission (NRC) dated March 7, 2006, Indiana Michigan Power Company (I&M, or the licensee) requested an amendment to the Renewed Facility Operating Licenses for Donald C. Cook Nuclear Plant, Units 1 and 2 (DCCNP-1 and DCCNP-2). The proposed amendment would revise Section 5.5.2, "Leakage Monitoring Program," of the units' Technical Specifications (TS), by adding the Liquid Waste Disposal System, Waste Gas System, and Post-Accident Containment Hydrogen Monitoring System to the list of systems. These systems were inadvertently omitted from Section 5.5.2.

The NRC staff's no significant hazards consideration analysis was published on April 11, 2006 (71 FR 18374).

2.0 REGULATORY EVALUATION

This amendment corrects an omission of text in the units' TSs whose intent was to list all of the systems included within the Leakage Monitoring System.

The applicable regulations are elements of 10 CFR Part 50, Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants." The applicable criteria are discussed below with quotations of text from Appendix B and underlining added to emphasize key wording:

- (1) Criterion III, "Design Control," provides that structures, systems, and components included in the design basis "are correctly translated into specifications, drawings, procedures, and instructions." In this regard, the Leakage Monitoring System was not correctly translated into Section 5.5.2 of the TS. The deficiency was that of incompleteness.
- (2) Criterion XVI, "Corrective Action," provides that "conditions adverse to quality, such as failures, malfunctions, deficiencies, deviations, defective material and equipment, and nonconformances are promptly identified and corrected." This proposed amendments would correct the deficient documentation.

ENCLOSURE

3.0 TECHNICAL EVALUATION

The licensee performed a technical analysis of the proposed amendments. The licensee stated that a license condition originally existed in the units' operating licenses, requiring the implementation of a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This license condition, however, did not specifically list the systems included in the program.

Amendments 287 (for Unit 1) and 269 (for Unit 2), dated June 1, 2005, (ADAMS Accession No. ML050620034) approved the conversion to Improved Technical Specifications (ITS). The conversion included the relocation of requirements from the "System Integrity" program to the "Leakage Monitoring Program" in ITS Section 5.5.2 (see Safety Evaluation for the Amendments, Section 5.0, "RELOCATED CNP UNITS 1 AND 2 LICENSE CONDITIONS," item 2 on page 112). Additionally, Table M - More Restrictive Changes, the row pertaining to ITS Requirement 5.5.2, on Page 67 of 67, stated that:

The program [referring to the System Integrity program] is not explicit as to which systems outside containment must be monitored. ITS 5.5.2 includes the requirements for the Leakage Monitoring Program and provides a list of systems that should be monitored because [emphasis added] they could contain highly radioactive fluids during a serious transient or accident.

Thus the intent of ITS 5.5.2 is to identify all systems that fit the criteria for inclusion in the Leakage Monitoring Program.

When the units' TSs were converted to the Improved Technical Specifications format on June 1, 2005, the license condition was deleted and Section 5.2.2 was added to each unit's TS, listing the systems that are to be monitored. In this process, the three systems were inadvertently omitted. Accordingly, the licensee proposed to add the three missing systems to Section 5.5.2 to correct this inadvertent error. The licensee stated that, even though the three systems are not currently listed in Section 5.2.2, they are nevertheless monitored per the requirements of Section 5.2.2. Thus, the proposed correction of Section 5.2.2 would produce no substantive change in plant operations.

The NRC staff reviewed the licensee's analysis and agrees that the proposed amendments do not alter the design or the operation of DCCNP-1 and DCCNP-2. The NRC staff agrees with the licensee's explanation summarized above and documented in more detail in the March 7, 2006, application. Therefore, the NRC staff finds the proposed amendments acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Michigan State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10) and no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The NRC staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: P. Tam
K. Feintuch

Date: July 5, 2006