UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
801 WARRENVILLE ROAD
LISLE, ILLINOIS 60532-4351

DEC 13 2002

E. Lynn McGuire
Director, National Health Physics Program (115HP/NLR)
Department of Veterans Affairs
Veterans Health Administration
2200 Fort Roots Drive
North Little Rock, AR 72114

Dear Mr. McGuire:

Enclosed is Amendment No. 92 to NRC Material License No. 21-04234-01 in accordance with your request. Please note that the changes made to your license are printed in **bold** font.

Please review the enclosed document carefully and be sure that you understand all conditions. If there are any errors or questions, please notify the U.S. Nuclear Regulatory Commission, Region III office at (630) 829-9887 so that we can provide appropriate corrections and answers.

Sincerely,

Kevin G. Null
Materials Licensing Branch

License No. 21-04234-01
Docket No. 030-02050

Enclosure: Amendment No. 92

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Amendment No. 92

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below. PL 02110 311 12

Licensee	In accordance with letter dated August 16, 2002,
1. V.A. Medical Center ATTN: Radiation Safety Officer (001F-S) 2. 4646 John R. Detroit, MI 48201	3. License number 21-04234-01 is amended in its entirety to read as follows: 4. Expiration date March 31, 2012 5. Docket No. 030-02050 Reference No.

6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
A.		
B.		
C.		
D.		
E.		
F. Any byproduct material identified in 10 CFR 31.11	F. Prepackaged Kits	F. 20 millicuries
G. Any byproduct material with Atomic Numbers 3-83, inclusive, except as noted below	G. Any	G. 50 millicuries of each radionuclide. Total possession not to exceed 2 curies
H. Hydrogen-3	H. Any	H. 250 millicuries
I. Carbon-14	I. Any	I. 100 millicuries
J. Iodine-125	J. Any	J. 100 millicuries

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6. Byproduct, source, and/or special nuclear material	7. Chemical and/or physical form	8. Maximum amount that licensee may possess at any one time under this license
K. Sulfur-35	K. Any	K. 600 millicuries
L. Technetium-99m	L. Any	L. 150 millicuries

9. Authorized Use:

A. Medical use described in 10 CFR 35.100.

B. Medical use described in 10 CFR 35.200.

C. Medical use described in 10 CFR 35.300.

D. Medical use described in 10 CFR 35.400.

E. Medical use described in 10 CFR 35.500 in devices which have been registered with the NRC pursuant to Section 32.210 of 10 CFR Part 32 or an Agreement State.

F. In vitro studies.

G. through L. Medical diagnosis, therapy, and research in humans (excluding sealed sources). Research and development as defined in Section 30.4 of 10 CFR Part 30, including animal studies and the licensee's instrument calibration only.

CONDITIONS

10. Licensed material shall be used only at the licensee's facilities approved by the licensee's Radiation Safety Committee located at the VA Medical Center, Detroit, Michigan. Ex.2

11. The Radiation Protection Officer for the activities authorized by this license is Steven D. Conatser.

12. A. The use of licensed material in or on humans shall be by a physician as defined in 10 CFR 35.2.

B. Physicians designated to use licensed material in or on humans shall meet the training criteria established in 10 CFR 35, Subpart J, and shall be designated by the licensee's Radiation Safety Committee, Ramesh Rao, M.D., Chairperson. The licensee shall maintain records of physicians designated as users.

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C. Licensed material for other than human use shall be used by, or under the supervision of, individuals designated by the Radiation Safety Committee, Ramesh Rao, M.D., Chairperson. The licensee shall maintain records of individuals designated as users.

13. In addition to the possession limits in Item 8., the licensee shall further restrict the possession of licensed material to quantities below the limits specified in 10 CFR 30.72 which require consideration of the need for an emergency plan for responding to a release of licensed material, and shall further restrict the possession of unsealed licensed material to quantities less than 105 times the applicable limits in Appendix C of 10 CFR 20, as specified in 10 CFR 30.35(d).

14. The licensee shall possess and use byproduct material for human use in accordance with the prescriptive and performance criteria in all sections of 10 CFR Part 35 except Sections 35.49(a) and (b), 35.100, 35.200, and 35.300.

15. Notwithstanding the requirements of 10 CFR 35.49(a) and (b), 10 CFR 35.100, 10 CFR 35.200, and 10 CFR 35.300, the licensee may use for medical use any byproduct material or reagent kit. The licensee shall possess and use byproduct material for medical use in accordance with the prescriptive and performance criteria in the other sections of 10 CFR 35. This does not relieve the licensee from complying with applicable Food and Drug Administration (FDA) and other federal and State requirements.

16. A. Sealed sources and detector cells shall be tested for leakage and/or contamination at intervals not to exceed 6 months or at such other intervals as specified by the certificate of registration referred to in 10 CFR 32.210.

B. Notwithstanding Paragraph A of this condition, sealed sources designed to emit alpha particles shall be tested for leakage and/or contamination at intervals not to exceed 3 months.

C. In the absence of a certificate from a transferor indicating that a test has been made, a sealed source or detector cell received from another person shall not be put into use until tested.

D. Sealed sources need not be leak tested if:

(i) they contain only hydrogen-3; or

(ii) they contain only krypton-85; or

(iii) the half-life of the isotope is 30 days or less; or

(iv) they contain not more than 100 microcuries of beta and/or gamma emitting material or not more than 10 microcuries of alpha emitting materials; or

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(v) they are not designed to emit alpha particles, are in storage, and are not being used. However, when they are removed from storage for use of transferred to another person, and have not been tested within the required leak test interval, they shall be tested before use or transfer. No sealed source or detector cell shall be stored for a period of more than 10 years without being tested for leakage and/or contamination.

E. The leak test shall be capable of detecting the presence of 0.005 microcurie of radioactive material on the test sample. Records of leak test results shall be kept in units of microcuries and shall be maintained for inspection by the Commission. If the test reveals the presence of 0.005 microcurie or more of removable contamination, a report shall be filed with the U.S. Nuclear Regulatory Commission and the source shall be removed from service and decontaminated, repaired, or disposed of in accordance with Commission regulations. The report shall be filed within 5 days of the date the leak test result is known with the U.S. Nuclear Regulatory Commission, Region III, 801 Warrenville Road, Lisle, Illinois 60532-4351, ATTN: Chief, Nuclear Materials Safety Branch. The report shall specify the source involved, the test results, and corrective action taken. Records of leak test results shall be kept in units of microcuries and shall be maintained for inspection by the Commission. Records may be disposed of following Commission inspection.

F. Tests for leakage and/or contamination shall be performed by the licensee or by other persons specifically licensed by the Commission or an Agreement State to perform such services.

17. Pursuant to Title 10, Chapter 1, Code of Federal Regulations, Part 40, "Domestic Licensing of Source Material," the licensee is authorized to possess, use, transfer, and import up to 999 kilograms of depleted uranium contained as shielding material.

18. Sealed sources or detector cells containing licensed material shall not be opened or sources removed from source holders by the licensee.

19. The licensee is authorized to hold licensed material with a physical half-life of less than 90 days for decay-in-storage before disposal in ordinary trash provided:

A. Radioactive waste to be disposed of in this manner shall be held for decay a minimum of 10 half-lives.

B. Before disposal as ordinary trash, byproduct material shall be surveyed at the container surface with the appropriate meter set on its most sensitive scale and with no interposed shielding to determine that its radioactivity cannot be distinguished from background. All radiation labels shall be removed or obliterated.

C. Generator columns shall be segregated so that they may be monitored separately to ensure decay to background levels prior to disposal.

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D. A record of each disposal permitted under this license condition shall be retained for three years. The record must include the date of disposal, the date on which the byproduct material was placed in storage, the radionuclides measured at the surface of each waste container, and the name of the individual who performed the disposal.

20. The licensee shall maintain records of information important to safe and effective decommissioning at the licensee's facilities listed in Condition 10. per the provisions of 10 CFR 30.35(g) until this license is terminated by the Commission.

21. Pursuant to 10 CFR 20.2002, the licensee may dispose of emptied scintillation vials, which previously contained sulfur-35, as normal waste, subject to the survey and sampling commitments in letter dated February 17, 1995 (excluding Item (1) under the heading "Disposal of Contaminated Vial Batches."

22. Except as specifically provided otherwise in this license, the licensee shall conduct its program in accordance with the statements, representations, and procedures contained in the documents, including any enclosures, listed below. The U.S. Nuclear Regulatory Commission's regulations shall govern unless the statements, representations, and procedures in the licensee's application and correspondence are more restrictive than the regulations.

A. Applications dated March 18, 1991 and September 4, 2001; and

B. Letters dated September 26, 1991 (with attachments), April 9, 1992, October 15, 1992 (excluding request to replace paragraph 4 of Item 11 (Waste Disposal)), December 18, 1992, February 10, 1993, March 8, 1993, March 29, 1994, October 18, 1994, June 2, 1995, February 17, 1995 (excluding Item (1) under "Disposal of Contaminated Vial Batches"), July 26, 1995 (with attachments), April 19, 1996 (with attachments), May 28, 1998 (with attachments), August 27, 1998 (with attachments), May 26, 2000, July 27, 2000 (with attachments), August 11, 2000 (with attachment), September 28, 2000, October 5, 2000, October 12, 2000, October 23, 2000 (with attachments), November 22, 2000 (with attachments), December 28, 2001 (with enclosure) and **December 4, 2002 (with enclosure).**

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Date DEC 18 2002

By _____
Kevin G. Null
Materials Licensing Branch
Region III