

May 16, 2006

Mrs. Mary G. Korsnick
Vice President R.E. Ginna Nuclear Power Plant
R.E. Ginna Nuclear Power Plant, LLC
1503 Lake Road
Ontario, NY 14519

SUBJECT: R.E. GINNA NUCLEAR POWER PLANT - REQUEST FOR ADDITIONAL
INFORMATION REGARDING STEAM GENERATOR TUBE INSPECTION
SUMMARY REPORT FOR THE SPRING 2005 OUTAGE (TAC NO. MD1207)

Dear Mrs. Korsnick:

By letter dated July 1, 2005, R.E. Ginna Nuclear Power Plant, LLC (the licensee), submitted its steam generator tube inspection summary report for the spring 2005 outage at R.E. Ginna Nuclear Power Plant.

The Nuclear Regulatory Commission staff has reviewed the information provided in the summary report and has determined that additional information is needed. The specific information is described in the enclosed request for additional information (RAI). During a telephone discussion with your staff on May 11, 2006, it was agreed that a response would be provided within 60 days from the date of this letter.

If you have any questions, please contact me at 301-415-1457.

Sincerely,

/RA/

Patrick D. Milano, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure:
RAI

cc w/encl: See next page

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The Nuclear Regulatory Commission staff has reviewed the information provided in the summary report and has determined that additional information is needed. The specific information is described in the enclosed request for additional information (RAI). During a telephone discussion with your staff on May 11, 2006, it was agreed that a response would be provided within 45 days from the date of this letter.

If you have any questions, please contact me at 301-415-1457.

Sincerely,

/RA/

Patrick D. Milano, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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Accession Number: ML061310405

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| OFFICE | LPLI-1\PM | LPLI-1\LA | CSGB\A)BC | LPLI-1\BC | |
| NAME | PMilano | SLittle | TBloomer | RLaufer | |
| DATE | 05/11/06 | 05/14/06 | 05/09/06 | 05/16/06 | |

OFFICIAL RECORD COPY

REQUEST FOR ADDITIONAL INFORMATION RE: STEAM GENERATOR TUBE
INSPECTION SUMMARY REPORT FOR THE SPRING 2005 OUTAGE AT R.E. GINNA
NUCLEAR POWER PLANT

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R. Laufer

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S. Little

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T. Bloomer

RidsNrrDciCsgb

K. Karwoski

Y. Díaz-Castillo

OGC

RidsOgcRp

ACRS

RidsAcrsAcnwMailCenter

B. McDermott, R-I

RidsRgn1MailCenter

B. Singal

RidsNrrDorlLdpr

cc: Plant Service list

R.E. Ginna Nuclear Power Plant

cc:

Mr. Michael J. Wallace
President
R.E. Ginna Nuclear Power Plant, LLC
c/o Constellation Energy
750 East Pratt Street
Baltimore, MD 21202

Mr. John M. Heffley
Senior Vice President and
Chief Nuclear Officer
Constellation Generation Group
1997 Annapolis Exchange Parkway
Suite 500
Annapolis, MD 21401

Kenneth Kolaczyk, Sr. Resident Inspector
R.E. Ginna Nuclear Power Plant
U.S. Nuclear Regulatory Commission
1503 Lake Road
Ontario, NY 14519

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Peter R. Smith, President
New York State Energy, Research,
and Development Authority
17 Columbia Circle
Albany, NY 12203-6399

Mr. Carey W. Fleming, Esquire
Senior Counsel - Nuclear Generation
Constellation Generation Group, LLC
750 East Pratt Street, 17th Floor
Baltimore, MD 21202

Mr. Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, NY 10271

Ms. Thelma Wideman, Director
Wayne County Emergency Management
Office
Wayne County Emergency Operations
Center
7336 Route 31
Lyons, NY 14489

Ms. Mary Louise Meisenzahl
Administrator, Monroe County
Office of Emergency Preparedness
1190 Scottsville Road, Suite 200
Rochester, NY 14624

Mr. Paul Eddy
New York State Department of
Public Service
3 Empire State Plaza, 10th Floor
Albany, NY 12223

REQUEST FOR ADDITIONAL INFORMATION
REGARDING STEAM GENERATOR TUBE INSPECTION REPORT

R.E. GINNA NUCLEAR POWER PLANT, LLC

R.E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

By letter dated July 1, 2005 (Agencywide Documents Access and Management System Accession No. ML051930115), R.E. Ginna Nuclear Power Plant, LLC (the licensee) submitted its steam generator (SG) tube inspection summary report for the spring 2005 refueling outage. The SG tube inspection summary was included with the transmittal of inservice inspection (ISI) report for the fourth ISI interval (2000-2009), second period, second outage (2005). In order to complete its review of the information in the report, the Nuclear Regulatory Commission (NRC) staff requests the following information:

1. In order to better understand the design of the replacement SGs, provide the following information: tube manufacturer, tubesheet thickness (with and without clad), smallest U-bend radius (including the row of tubes with this radius), and tube pitch.
2. In Section 2.0 of the July 1, 2005, report, the licensee indicated that one tube in SG "B" was inspected with a rotating probe because it had reported non-conformance. Describe the condition of this tube (i.e., what prompted the non-conformance report).
3. In Section 2.0, the licensee stated that, as part of its supplemental inspection, it would inspect a 20% sample of reported dings/dents with a voltage greater than or equal to 2 volts. Then, in Section 5.2.1 of the report, the licensee stated that the reporting thresholds for dings/dents in the freespan was 2.5 volts and for dings/dents in the U-bend was 5 volts. Clarify this apparent discrepancy.
4. In Section 5.2.1, the licensee stated that a restricted tube was noted with a 0.620-inch probe diameter. Describe the nature of the restriction. Include a discussion of whether the obstruction was service-induced and the extent and location of the restriction.
5. In Section 5.0, the licensee stated that a comparison was made between all the bobbin signals meeting the current reporting criteria with the 1997 inspection results to determine if the signal was present and if it had changed in characteristics. Discuss the results of this comparison.
6. For SGs "A" and "B," discuss whether a foreign object search and retrieval was performed in each SG and whether the loose parts were removed from the SGs. If the parts were not removed or the locations were not visually inspected, discuss the results of any evaluations performed to ensure these parts (or suspected parts) would not result in a loss of tube integrity for the period of time between inspections. In addition, discuss whether a visual inspection was performed around the tubes in row 94/column 54 and row 93/column 55 in SG "A" to confirm that the eddy current signals were from deposits.

Enclosure

7. Provide a discussion if any wear indication at support structures was identified. If so, discuss the number of tubes affected and the severity of the indications.
8. In Section 4.4, the licensee stated that previously identified U-bend areas having tube to tube proximity signals were examined to verify that no change has occurred. Although no changes to these previously identified proximity signals were identified, discuss whether any new tube to tube proximity signals were identified. If so, discuss the number of tubes affected and the implications.