

May 16, 2006

Mrs. Mary G. Korsnick  
Vice President R.E. Ginna Nuclear Power Plant  
R.E. Ginna Nuclear Power Plant, LLC  
1503 Lake Road  
Ontario, NY 14519

SUBJECT: R.E. GINNA NUCLEAR POWER PLANT - REQUEST FOR ADDITIONAL  
INFORMATION REGARDING STEAM GENERATOR TUBE INSPECTION  
SUMMARY REPORT FOR THE SPRING 2005 OUTAGE (TAC NO. MD1207)

Dear Mrs. Korsnick:

By letter dated July 1, 2005, R.E. Ginna Nuclear Power Plant, LLC (the licensee), submitted its steam generator tube inspection summary report for the spring 2005 outage at R.E. Ginna Nuclear Power Plant.

The Nuclear Regulatory Commission staff has reviewed the information provided in the summary report and has determined that additional information is needed. The specific information is described in the enclosed request for additional information (RAI). During a telephone discussion with your staff on May 11, 2006, it was agreed that a response would be provided within 60 days from the date of this letter.

If you have any questions, please contact me at 301-415-1457.

Sincerely,

*/RA/*

Patrick D. Milano, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure:  
RAI

cc w/encl: See next page

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Vice President R.E. Ginna Nuclear Power Plant  
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The Nuclear Regulatory Commission staff has reviewed the information provided in the summary report and has determined that additional information is needed. The specific information is described in the enclosed request for additional information (RAI). During a telephone discussion with your staff on May 11, 2006, it was agreed that a response would be provided within 45 days from the date of this letter.

If you have any questions, please contact me at 301-415-1457.

Sincerely,

*/RA/*

Patrick D. Milano, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

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Accession Number: ML061310405

OFFICE	LPLI-1\PM	LPLI-1\LA	CSGB\A)BC	LPLI-1\BC	
NAME	PMilano	SLittle	TBloomer	RLaufer	
DATE	05/11/06	05/14/06	05/09/06	05/16/06	

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REQUEST FOR ADDITIONAL INFORMATION RE: STEAM GENERATOR TUBE  
INSPECTION SUMMARY REPORT FOR THE SPRING 2005 OUTAGE AT R.E. GINNA  
NUCLEAR POWER PLANT

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T. Bloomer

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K. Karwoski

Y. Díaz-Castillo

OGC

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ACRS

RidsAcrsAcnwMailCenter

B. McDermott, R-I

RidsRgn1MailCenter

B. Singal

RidsNrrDorlLdpr

cc: Plant Service list

R.E. Ginna Nuclear Power Plant

cc:

Mr. Michael J. Wallace  
President  
R.E. Ginna Nuclear Power Plant, LLC  
c/o Constellation Energy  
750 East Pratt Street  
Baltimore, MD 21202

Mr. John M. Heffley  
Senior Vice President and  
Chief Nuclear Officer  
Constellation Generation Group  
1997 Annapolis Exchange Parkway  
Suite 500  
Annapolis, MD 21401

Kenneth Kolaczyk, Sr. Resident Inspector  
R.E. Ginna Nuclear Power Plant  
U.S. Nuclear Regulatory Commission  
1503 Lake Road  
Ontario, NY 14519

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, PA 19406

Mr. Peter R. Smith, President  
New York State Energy, Research,  
and Development Authority  
17 Columbia Circle  
Albany, NY 12203-6399

Mr. Carey W. Fleming, Esquire  
Senior Counsel - Nuclear Generation  
Constellation Generation Group, LLC  
750 East Pratt Street, 17th Floor  
Baltimore, MD 21202

Mr. Charles Donaldson, Esquire  
Assistant Attorney General  
New York Department of Law  
120 Broadway  
New York, NY 10271

Ms. Thelma Wideman, Director  
Wayne County Emergency Management  
Office  
Wayne County Emergency Operations  
Center  
7336 Route 31  
Lyons, NY 14489

Ms. Mary Louise Meisenzahl  
Administrator, Monroe County  
Office of Emergency Preparedness  
1190 Scottsville Road, Suite 200  
Rochester, NY 14624

Mr. Paul Eddy  
New York State Department of  
Public Service  
3 Empire State Plaza, 10th Floor  
Albany, NY 12223

REQUEST FOR ADDITIONAL INFORMATION  
REGARDING STEAM GENERATOR TUBE INSPECTION REPORT  
R.E. GINNA NUCLEAR POWER PLANT, LLC  
R.E. GINNA NUCLEAR POWER PLANT  
DOCKET NO. 50-244

By letter dated July 1, 2005 (Agencywide Documents Access and Management System Accession No. ML051930115), R.E. Ginna Nuclear Power Plant, LLC (the licensee) submitted its steam generator (SG) tube inspection summary report for the spring 2005 refueling outage. The SG tube inspection summary was included with the transmittal of inservice inspection (ISI) report for the fourth ISI interval (2000-2009), second period, second outage (2005). In order to complete its review of the information in the report, the Nuclear Regulatory Commission (NRC) staff requests the following information:

1. In order to better understand the design of the replacement SGs, provide the following information: tube manufacturer, tubesheet thickness (with and without clad), smallest U-bend radius (including the row of tubes with this radius), and tube pitch.
2. In Section 2.0 of the July 1, 2005, report, the licensee indicated that one tube in SG "B" was inspected with a rotating probe because it had reported non-conformance. Describe the condition of this tube (i.e., what prompted the non-conformance report).
3. In Section 2.0, the licensee stated that, as part of its supplemental inspection, it would inspect a 20% sample of reported dings/dents with a voltage greater than or equal to 2 volts. Then, in Section 5.2.1 of the report, the licensee stated that the reporting thresholds for dings/dents in the freespan was 2.5 volts and for dings/dents in the U-bend was 5 volts. Clarify this apparent discrepancy.
4. In Section 5.2.1, the licensee stated that a restricted tube was noted with a 0.620-inch probe diameter. Describe the nature of the restriction. Include a discussion of whether the obstruction was service-induced and the extent and location of the restriction.
5. In Section 5.0, the licensee stated that a comparison was made between all the bobbin signals meeting the current reporting criteria with the 1997 inspection results to determine if the signal was present and if it had changed in characteristics. Discuss the results of this comparison.
6. For SGs "A" and "B," discuss whether a foreign object search and retrieval was performed in each SG and whether the loose parts were removed from the SGs. If the parts were not removed or the locations were not visually inspected, discuss the results of any evaluations performed to ensure these parts (or suspected parts) would not result in a loss of tube integrity for the period of time between inspections. In addition, discuss whether a visual inspection was performed around the tubes in row 94/column 54 and row 93/column 55 in SG "A" to confirm that the eddy current signals were from deposits.

Enclosure

7. Provide a discussion if any wear indication at support structures was identified. If so, discuss the number of tubes affected and the severity of the indications.
8. In Section 4.4, the licensee stated that previously identified U-bend areas having tube to tube proximity signals were examined to verify that no change has occurred. Although no changes to these previously identified proximity signals were identified, discuss whether any new tube to tube proximity signals were identified. If so, discuss the number of tubes affected and the implications.