

May 30, 2006

Mr. Britt T. McKinney
Sr. Vice President and
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Berwick, PA 18603-0467

SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2 - ISSUANCE
OF AMENDMENTS RE: REVISION TO TECHNICAL SPECIFICATION
SURVEILLANCE REQUIREMENTS 3.8.4.7 AND 3.8.4.8 (TAC NOS. MD0254
AND MD0255)

Dear Mr. McKinney:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 235 to Facility Operating License No. NPF-14 and Amendment No. 212 to Facility Operating License No. NPF-22 for the Susquehanna Steam Electric Station, Units 1 and 2 (SSES 1 and 2). These amendments are in response to your application dated February 28, 2006, as supplemented on April 7, 2006.

The amendments revise the SSES 1 and 2 Technical Specification Surveillance Requirements 3.8.4.7 and 3.8.4.8 to clarify that Diesel Generator "E" (DG E) electrical power subsystem testing does not require a mode restriction when the DG E diesel is not aligned to the Class 1E distribution system.

A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's Biweekly *Federal Register* Notice.

Sincerely,

/RA/

Richard V. Guzman, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-387 and 50-388

Enclosures: 1. Amendment No. 235 to
License No. NPF-14
2. Amendment No. 212 to
License No. NPF-22
3. Safety Evaluation

cc w/encls: See next page

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Dear Mr. McKinney:

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cc w/encls: See next page

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LPLI-1 RF	RidsOGC Rp	RidsAcrsAcnwMailCenter	RidsRgn1MailCenter
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RidsNrrDpr	RidsNrrDeEeeb		

* SE inputs provided by memo. No substantive changes made.

Accession No.: ML061290048

Package No.: ML061290061

TSs: ML06

OFFICE	LPLI-1/PM	LPLI-1/LA	IROB/SC	EEEEB/SC	OGC	LPLI-1/BC
NAME	RGuzman	SLittle	TBoyce*	GWilson*	SHamrick	RLaufer
DATE	5/30/06	5/30/06	4/12/06	5/4/06	5/18/06	5/30/06

Susquehanna Steam Electric Station, Unit Nos. 1 and 2

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Susquehanna Steam Electric Station, Unit Nos. 1 and 2

cc:

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PPL SUSQUEHANNA, LLC
ALLEGHENY ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-387
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 235
License No. NPF-14

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by PPL Susquehanna, LLC, dated February 28, 2006, as supplemented on April 7, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-14 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 235 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PPL Susquehanna, LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and License

Date of Issuance: May 30, 2006

ATTACHMENT TO LICENSE AMENDMENT NO.235

FACILITY OPERATING LICENSE NO. NPF-14

DOCKET NO. 50-387

Replace the following page of the Facility Operating License No. NPF-14 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

REMOVE

3

INSERT

3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3.8-26

3.8-27

INSERT

TS/3.8-26

TS/3.8-27

PPL SUSQUEHANNA, LLC
ALLEGHENY ELECTRIC COOPERATIVE, INC.
DOCKET NO. 50-388
SUSQUEHANNA STEAM ELECTRIC STATION, UNIT 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 212
License No. NPF-22

1. The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for the amendment filed by PPL Susquehanna, LLC, dated February 28, 2006, as supplemented on April 7, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-22 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 212 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. PPL Susquehanna, LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Richard J. Laufer, Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and License

Date of Issuance: May 30, 2006

ATTACHMENT TO LICENSE AMENDMENT NO. 212

FACILITY OPERATING LICENSE NO. NPF-22

DOCKET NO. 50-388

Replace the following page of the Facility Operating License No. NPF-22 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

REMOVE

3

INSERT

3

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3.8-30

3.8-31

INSERT

TS/3.8-30

TS/3.8-31

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 235 TO FACILITY OPERATING LICENSE NO. NPF-14
AND AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. NPF-22
PPL SUSQUEHANNA, LLC
ALLEGHENY ELECTRIC COOPERATIVE, INC.
SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2
DOCKET NOS. 50-387 AND 50-388

1.0 INTRODUCTION

By application dated February 28, 2006 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML060620067), as supplemented by letter dated April 7, 2006 (ML061080583), PPL Susquehanna, LLC (PPL, the licensee), requested changes to the Technical Specifications (TSs) for Susquehanna Steam Electric Station, Units 1 and 2 (SSES 1 and 2).

The proposed changes would revise the SSES 1 and 2 TS Surveillance Requirements (SRs) 3.8.4.7 and 3.8.4.8 to clarify that Diesel Generator "E" (DG E) electrical power subsystem testing does not require a mode restriction when the DG E diesel is not aligned to the Class 1E distribution system.

The supplement dated April 7, 2006, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on March 28, 2006 (71 FR 15485).

2.0 REGULATORY EVALUATION

The Nuclear Regulatory Commission (NRC) finds that PPL, in its February 28, 2006, submittal, identified the applicable regulatory requirements. The regulatory requirements and guidance which the NRC staff considered in its review of the application are as follows:

1. Title 10 of the *Code of Federal Regulations* (10 CFR) establishes the fundamental regulatory requirements with respect to the electric power distribution systems. Specifically, General Design Criterion (GDC) 17, "Electrical power systems," in Appendix A to Part 50, "General Design Criteria for Nuclear Power Plants," states, in part, that nuclear power plants must have onsite and offsite electric power systems to permit the functioning of structures, systems, and components that are important to safety. The onsite system is required to have sufficient independence, redundancy, and testability to

perform its safety function, assuming a single failure. The offsite power system must be supplied by two physically independent circuits that are designed and located so as to minimize, to the extent practical, the likelihood of their simultaneous failure under operating and postulated accident and environmental conditions. In addition, this criterion requires provisions to minimize the probability of losing electric power from the remaining electric power supplies as a result of loss of power from the unit, the offsite transmission network, or the onsite power supplies.

2. GDC 18, "Inspection and testing of electric power systems," requires that electric power systems that are important to safety be designed to permit appropriate periodic inspection and testing.
3. 10 CFR 50.36(c)(3), "Technical specifications," requires a licensee's TSs to have SRs relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operations are within safety limits, and that the limiting conditions for operation (LCOs) will be met.
4. 10 CFR 50.63, "Loss of all alternating current power," requires that each light-water cooled nuclear power plant licensed to operate must be able to withstand for a specified duration and recover from a station blackout.
5. 10 CFR 50.65, "Requirements for monitoring the effectiveness of maintenance at nuclear power plants," requires that preventive maintenance activities must not reduce the overall availability of the systems, structures, and components.
6. Regulatory Guide (RG) 1.32, "Criteria For Safety-Related Electric Power Systems For Nuclear Power Plants," provides guidance for complying with GDC 17 and 18 with respect to design, operation, and testing of safety-related electric power systems of all types of nuclear power plants.
7. RG 1.129, "Maintenance, Testing, and Replacement of Large Lead Storage Batteries for Nuclear Power Plants," describes a method acceptable to the NRC staff for performing the maintenance, testing, and replacement of large lead storage batteries for all types of nuclear power plants.

3.0 TECHNICAL EVALUATION

3.1 Design of SSES 1 and 2, Electrical Power System

SSES 1 and 2 has a total of five DGs. The DGs are shared by the two SSES units. DG A, B, C, and D are normally assigned to the safety-related load groups. DG E is capable of being substituted for any of the DGs A, B, C, or D without violating the independence of the redundant safety-related load groups. Only four DGs can be aligned to the safety-related load groups. When a DG is aligned, it is connected to the 4.16 kilo-Volt (kV) bus of the assigned load group per unit. The capacity of the aligned DGs (assuming one of the aligned diesel fails) is sufficient to operate the engineered safety features loads of one unit and those systems required for concurrent safe shutdown of the second unit. Therefore, with one DG unavailable, the remaining three in-service DGs have the capacity and capability to supply the required

engineered safety features loads in one unit; the equipment needed to safely shutdown the second unit; and any more engineered safety features loads manually switched onto the DGs as described in the SSES 1 and 2 Final Safety Analysis Report (FSAR).

There are no provisions for parallel operation of the aligned DG of one load group with the aligned DG of the redundant load group. The DG circuit breaker and the offsite power incoming circuit breakers are interlocked to prevent feedback into the offsite power system. These interlocks are bypassed during load testing of the aligned DG; however, only one unit is tested at any one time. When not aligned, DG E can be tested through a 13.8 - 4.16 kV test facility transformer which permits DG E to be synchronized to the offsite power system. DGs A, B, C, or D do not have the capability of being tested when not aligned.

A separate and independent Class 1E 125 V DC battery is installed as a dedicated DC power supply for DG E Class 1E loads and the four motor-operated valves used to align emergency service water to DG E. This 125 V DC electrical power subsystem contains a battery bank, a battery charger, a motor control center, and a distribution panel to distribute power to connected Class 1E loads. Each 125 V DC battery has adequate storage capacity to continuously supply power to the required loads for at least 4 hours.

3.2 Evaluation of Proposed Changes

In its letter dated February 28, 2006, as supplemented by letter dated April 7, 2006, PPL proposed changes to the SSES 1 and 2 TS SRs 3.8.4.7 and 3.8.4.8 to clarify that DG E DC electrical power subsystem testing does not require a mode restriction when DG E is not aligned to the Class 1E distribution system.

PPL provided the following technical basis for their proposed TS changes:

The proposed revised Note currently precludes performance of the SRs when in Mode 1, 2 or 3. The TS Bases identify the reason for the Note is that performing the Surveillance would remove a required DC electrical power subsystem from service, perturb the Electrical Distribution System, and challenge safety systems.

For the subsystem other than the DG E subsystem identified in TS Table 3.8.4-1, the SRs are performed when the respective subsystem's unit is in Mode 4 and 5. This is done to comply with the SR Note for the reasons described in the SR 3.8.4.7 and the SR 3.4.8.8 TS Bases.

Because the DG E subsystem does not support loads other than the DG E required loads as described previously (no unit common or unit specific loads are supplied by the subsystem), the Mode Restriction Note need not be applied to the DG E subsystem unless it is substituted for one of the LCO 3.8.1 required DGs. When the DG E is not substituted, it is not one of the four LCO 3.8.1 required DGs.

However, the Note does have applicability to the DG E and the associated subsystem when the DG E is substituted for one of the other DGs. When the DG E is substituted for one of the four LCO 3.8.1 required DGs, the DG E subsystem is required to support operability of the DG E. Thus, when in this configuration, the note needs to be applicable, since as stated in the TS bases, performing the surveillance would remove the DG E DC electrical power subsystem from service when it is required.

The NRC staff reviewed and evaluated each of the proposed changes to the SSES 1 and 2 TSs as follows:

3.2.1 TS SR 3.8.4.7

Currently, the Note in TS SR 3.8.4.7 states:

This Surveillance shall not be performed in MODE 1, 2, or 3.

The Note is changed to state the following:

This Surveillance shall not be performed in MODE 1, 2 or 3 except for the Diesel Generator E DC electrical power subsystem. This Surveillance can be performed on the Diesel Generator E DC electrical power subsystem when the Diesel Generator E is not aligned to the Class 1E distribution system.

SR 3.8.4.7 requires verification of the battery capacity to be adequate to supply and maintain in operable status the required emergency loads for the design duty cycle by performance of a battery service test every 24 months. The licensee proposed revising this SR to remove the MODE restrictions for performing a battery service test or modified performance test on the DG E battery when DG E is not aligned to the Class 1E distribution system.

A battery service test is a special test of the battery's capability, as found, to satisfy the design requirements (battery duty cycle) of the DC electrical power system. The test can be conducted using actual or simulated loads. The discharge rate and test length corresponds to the design duty cycle requirements as specified in the SSES 1 and 2 FSAR.

The frequency of 24 months is consistent with the recommendations of RG 1.32 and RG 1.129, which state that the battery service test should be performed during refueling operations or at some other outage, with intervals between tests not to exceed 24 months.

This SR is modified by two Notes. One of the Notes allows the performance of a modified performance discharge test in lieu of a service test once per 60 months.

A modified performance discharge test is a test of the battery capacity and its ability to provide a high rate, short duration load (usually the highest rate of the duty cycle). This will confirm the battery's ability to meet the critical period of the load duty cycle, in addition to determining its percentage of rated capacity. Initial conditions for the modified performance discharge test should be identical to those specified for a service test.

The modified performance discharge test is a test of simulated duty cycle consisting of two different discharge rates. The first discharge rate consists of the 1 minute published rate for the battery or the largest current loads of the duty cycle followed by a second discharge rate which employs the test rate for the performance discharge test. These discharge rates envelope the duty cycle of the service test. Since the ampere-hours removed by a published 1 minute discharge rate represent a very small portion of the battery capacity, the test rate can be changed to that for the performance discharge test without compromising the results of the performance discharge test. The battery terminal voltage for the modified performance discharge test should remain above the minimum battery terminal voltage specified in the service test.

The other Note restricts the licensee from performing this Surveillance during certain MODES of operation, specifically, MODES 1, 2, and 3. The reason for the Note is that performing the Surveillance would remove a required DC electrical power subsystem from service, perturb the Electrical Distribution System, and challenge safety systems.

The proposed revision will prevent the performance of this Surveillance when in MODES 1, 2, or 3 for DC electrical subsystems other than the DG E DC electrical subsystem. This change is justified by the fact that the DG E DC electrical subsystem does not support loads other than the DG E required loads and because the MODE restrictions will be applicable when DG E is substituted for one of the four LCO 3.8.1 required DGs. Based on this information, the NRC staff finds the proposed change to revise the Note in SR 3.8.4.7 acceptable.

3.2.2 TS SR 3.8.4.8

Currently, the Note in TS SR 3.8.4.7 states:

This Surveillance shall not be performed in MODE 1, 2 or 3.

The Note is changed to state the following:

This Surveillance shall not be performed in MODE 1, 2 or 3 except for the Diesel Generator E DC electrical power subsystem. This Surveillance can be performed on the Diesel Generator E DC electrical power subsystem when the Diesel Generator E is not aligned to the Class 1E distribution system.

SR 3.8.4.8 requires verification that the battery capacity is greater than or equal to 80% of the manufacturer's rating when the battery is subjected to a performance discharge test or a modified performance discharge test. A battery performance discharge test is a test of constant current capacity of a battery, normally done in the as found condition, after having been in service, to detect any change in the capacity determined by the acceptance test. The test is intended to determine overall battery degradation due to age and usage.

This test must be conducted every 60 months and every 12 months when the battery shows signs of degradation or has reached 85% of expected service life with capacity less than 100% of the manufacturer's rating, and every 24 months when the battery has reached 85% of the expected service life with capacity greater than or equal to 100% of the manufacturer's rating.

This SR is modified by a Note. The Note restricts the licensee from performing this Surveillance during certain MODES of operation, specifically, MODES 1, 2, and 3. The reason for the Note is that performing the Surveillance would remove a required DC electrical power subsystem from service, perturb the Electrical Distribution System, and challenge safety systems.

The proposed revision will prevent the performance of this Surveillance when in MODES 1, 2, or 3 for DC electrical subsystems other than the DG E DC electrical subsystem. This change is justified by the fact that the DG E DC electrical subsystem does not support loads other than the DG E required loads and because the MODE restrictions will be applicable when DG E is substituted for one of the four LCO 3.8.1 required DGs. Based on this information, the NRC staff finds the proposed change to revise the Note in SR 3.8.4.8 acceptable.

3.3 NRC Staff Conclusion

Based on the review of the above changes, the NRC staff concludes that the proposed changes to SR 3.8.4.7 and 3.8.4.8 are acceptable since the changes allow performance of the SRs only when the DG E is not aligned to the Class 1E distribution system. The proposed changes are consistent with PPL's compliance with the applicable regulations noted in Section 2.0 above. The NRC staff concludes that the proposed changes do not affect the safe and reliable operation of the plant, maintains compliance with requirements governing the design and operation of the electrical power system and, therefore, are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State Official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change SRs. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (71 FR 15485). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded based on the considerations discussed above that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: R. Guzman
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M. McConnell

Date: May 30, 2006