



Entergy Nuclear South
Entergy Operations, Inc.
17265 River Road
Killona, LA 70057-3093
Tel 504-739-6780
Fax 504-739-6693
tanker@entergy.com

Tom Tankersley
Licensing Manager
Waterford 3

W3F1-2006-0012

May 1, 2006

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

Subject: Report of Facility Changes, Tests and Experiments and Commitment
Changes
Waterford Steam Electric Station, Unit 3 (Waterford 3)
Docket No. 50-382
License No. NPF-38

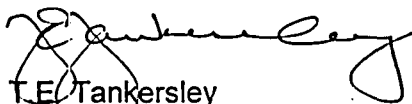
Dear Sir or Madam:

Enclosed is the summary report of facility changes, tests and experiments for Waterford 3, which is submitted pursuant to 10CFR50.59 (d)(2). This report covers the period from June 1, 2004 through April 1, 2006. This submittal also includes a CD-ROM of the 10CFR50.59 Evaluation for each change and a summary report of Commitment Changes for the same time period.

If you have any questions regarding this report, please contact Michael E. Mason at (504) 739-5673.

There are no new commitments contained in this submittal.

Sincerely,


T.E. Tankersley
Licensing Manager

TET/MEM/cbh
Attachment(s)

IE47

cc: (w/Attachment – CD Rom)

Dr. Bruce S. Mallett
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector
Waterford Steam Electric Station Unit 3
P.O. Box 822
Killona, LA 70066-0751

U. S. Nuclear Regulatory Commission
Attn: Mr. Mel B. Fields
Mail Stop O-07D1
Washington, DC 20555-0001

Wise, Carter, Child & Caraway
ATTN: J. Smith
P.O. Box 651
Jackson, MS 39205

Winston & Strawn
ATTN: N.S. Reynolds
1700 K Street, NW
Washington, DC 20006-3817

Morgan, Lewis & Bockius LLP
ATTN: T.C. Poindexter
1111 Pennsylvania Avenue, NW
Washington, DC 20004

Attachment 1

W3F1-2006-0012

Waterford 3 10 CFR 50.59 Summary Report

10CFR50.59 Evaluation Number	Initiating Document	Summary
04-004	Amendment 91	New TRM 3.0/4.0 Section, Remove Shutdown Requirements From TRM
04-005	Amendment 92	Move Pressurizer Heat-up and Cooldown Limits to TRM and Remove Shutdown Requirements
04-007	Calculations EC-M88-021 and EC-E89-016	Station Blackout Water Requirements From Extended Power Up-rate
04-008	ER-W3-2001-1149-005	Extended Power Up-rate Impact To FSAR Chapter 5
04-009	ER-W3-2001-1149-010	Extended Power Up-rate Impact To FSAR Chapter 10
05-001	ER-W3-2004-1129-015	Extended Power Up-rate Impact To FSAR Chapter 15
05-002	ER-W3-2001-1149-009	Extended Power Up-rate Impact To FSAR Chapter 9
05-003	ER-W3-2004-0137-000	In Core Instrumentation Thimble Support Plate Modification
05-004	ER-W3-2004-0276-000	Alternate Source Term Implementation
05-005	ER-W3-2004-0478-000	Steam Generator Sub-compartment Platforms Addition Impact
05-006	ER-W3-2004-0602-000	Steam Generator Feedwater Pump Trip Logic Modification Due To Extended Power Up-rate
05-007	ER-W3-2004-0122-000	Pressurizer Heater Sleeves And Instrument Nozzles Weld Repair
05-008	ER-W3-2005-0019-000	Steam Generator Level Set-points Modification Due To Extended Power Up-rate
05-009	ER-W3-2003-0662-000	480 Volt Safety Buses Limit Change When On Startup Transformers
05-011	ER-W3-2005-0083-000	Health Physics Calculations Implementation From Extended Power Up-rate And Alternate Source Term
05-012	ER-W3-2004-0116-003	Core Next Generation Fuel And Lead Test Assemblies From Reload Analysis Report
05-013	ER-W3-2003-0053-000	High Pressure Turbine Design Upgrade For Extended Power Up-rate
05-014	ER-W3-2002-0595-000	Main Generator Output Breaker Modification From Extended Power Up-rate
05-015	ER-W3-2005-0130-000	Temporary Alteration For Keeping Circulating Water System Operating
05-016	License Amendment Request NPF 38-249	Technical Requirements Manual and Final Safety Analysis Report change due to Extended Power Up-rate
05-017	Core Operating Limits Report and Startup and Setpoint Transmittal for Cycle 14	Changes To Moderator Temperature Coefficient, Linear Heat Rate, Departure From Nuclear Boiling Rate Margin, Axial Shape Index; Revised Core Operating Limit

05-018	ER-W3-2004-0615-000	Supervisory System And Core Protection Calculator System Set-points Ultimate Heat Sink Impact Due To Extended Power Up-rate With Design Bases Tornado Event
05-019	ER-W3-2005-0145-000	Annulus Negative Pressure Impact Due To Extended Power Up-Rate Considering Instrument Uncertainty
05-020	ER-W3-2004-0276-001	Environmental Qualification Impacts Due To Alternate Source Term Methodology
05-021	ER-W3-2004-0276-002	Steam Generator Tube Rupture Impacts Due To Alternate Source Term Methodology With Revised Assumptions
05-022	ER-W3-2004-0122-000	Pressurizer Heater Sleeve And Instrument Nozzle Repair
05-023	ER-W3-2005-0378-000	Outside Containment Leak Reduction Program Test Medium Change
05-024	ER-W3-2005-0396-000	Confirmatory Criticality Calculation Correction For Spent Fuel Racks
05-025	ER-W3-2005-0334-001	Temporary Alteration For Reactor Vessel Gasket Leak-off Pressure High Annunciator
05-026	ER-W3-2004-0331-000	Reactor cavity cooling system temperature clarification
05-027	ER-W3-2000-0334-000	Heating Ventilation And Air Conditioning Equipment Room And Component Cooling Heat Exchanger Room Calculations Correction