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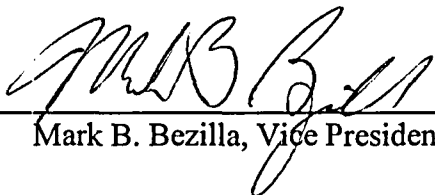
Subject: Davis-Besse Nuclear Power Station
Core Operating Limits Report for Cycle 15

Ladies and Gentlemen:

This letter transmits Revision 0 of the Davis-Besse Nuclear Power Station, Unit 1 (DBNPS) Core Operating Limits Report (COLR) for Cycle 15 operation (Enclosure 1). Submittal of this revision is in accordance with DBNPS Technical Specification 6.9.1.7.

Attachment 1, Commitment List, identifies that there are no commitments contained in this letter. If there are any questions or if additional information is required, please contact Mr. Gregory A. Dunn, Manager – FENOC Fleet Licensing, at (330) 315-7243.

Sincerely,



Mark B. Bezilla, Vice President-Nuclear

MKL

Enclosures

cc: Regional Administrator, NRC Region III
NRC/NRR Project Manager
DB-1 Senior Resident Inspector, NRC Region III
Utility Radiological Safety Board

A001

Docket Number 50-346
License Number NPF-3
Serial Number 3252
Enclosure 1

FIRSTENERGY NUCLEAR OPERATING COMPANY

DAVIS-BESSE UNIT 1

CYCLE 15

CORE OPERATING LIMITS REPORT

REVISION 0

(31 pages follow)

FIRSTENERGY NUCLEAR OPERATING COMPANY

DAVIS-BESSE UNIT 1

CYCLE 15

CORE OPERATING LIMITS REPORT

Prepared by *D. B. Kelley 4/15/06*
D. B. Kelley

Reviewed by *S. M. Hopper*
S. M. Hopper

Approved by *A.R. Burger per telecall D. B. Kelley 4/18/06*
A.R. Burger

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3.1.3.6 and 3.1.3.8	Figure 1c	Regulating Group Position Operating Limits, 0 to 300 ±10 EFPD, Three RC Pumps, 2772 MWth
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FIRSTENERGY NUCLEAR OPERATING COMPANY

DAVIS-BESSE UNIT 1

CYCLE 15

CORE OPERATING LIMITS REPORT

1.0 Core Operating Limits

This CORE OPERATING LIMITS REPORT for DB-1 Cycle 15 has been prepared in accordance with the requirements of Technical Specification 6.9.1.7. The Core Operating Limits have been developed using the methodology provided in reference 2.0 (1). The licensed length of Cycle 15 is 711.36 EFPDs (based on a reactor thermal rating of 2772 MWt).

The following cycle-specific core Operating Limits, Protective Limit and Flux - Δ Flux/Flow Reactor Protection System Allowable Values are included in this report:

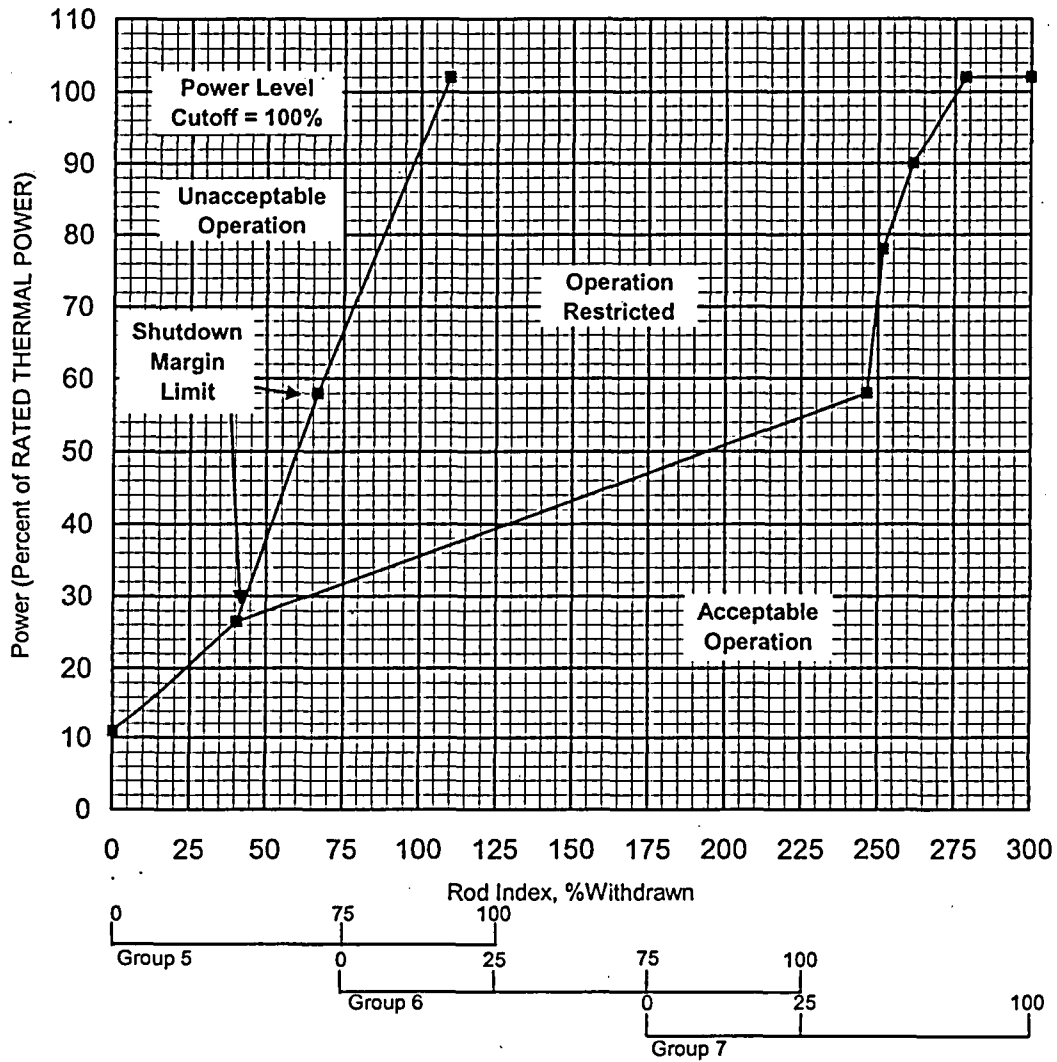
- 1) Regulating Group Position Alarm Setpoints (error adjusted Operating Limits) and Xenon reactivity "power level cutoff"
- 2) Rod program group positions (Control Rod Core locations and group assignments)
- 3) Axial Power Shaping Rod Alarm Setpoints (error adjusted Operating Limits)
- 4) AXIAL POWER IMBALANCE Alarm Setpoints (error adjusted Operating Limits)
- 5) AXIAL POWER IMBALANCE Protective Limits
- 6) Flux- Δ Flux/Flow (or Power/Imbalance/Flow) Allowable Values
- 7) QUADRANT POWER TILT limits
- 8) Negative Moderator Temperature Coefficient limit
- 9) Nuclear Heat Flux Hot Channel Factor, F_Q and
- 10) Nuclear Enthalpy Rise Hot Channel Factor, $F_{\Delta H}^N$

2.0 References

- 1) BAW-10179P-A, Rev. 6, "Safety Criteria and Methodology For Acceptable Cycle Reload Analyses", August, 2001.

Figure 1a Regulating Group Position Operating Limits
 0 to 300 ±10 EFPD, Four RC Pumps--2772 MWt RTP
 Davis-Besse 1, Cycle 15

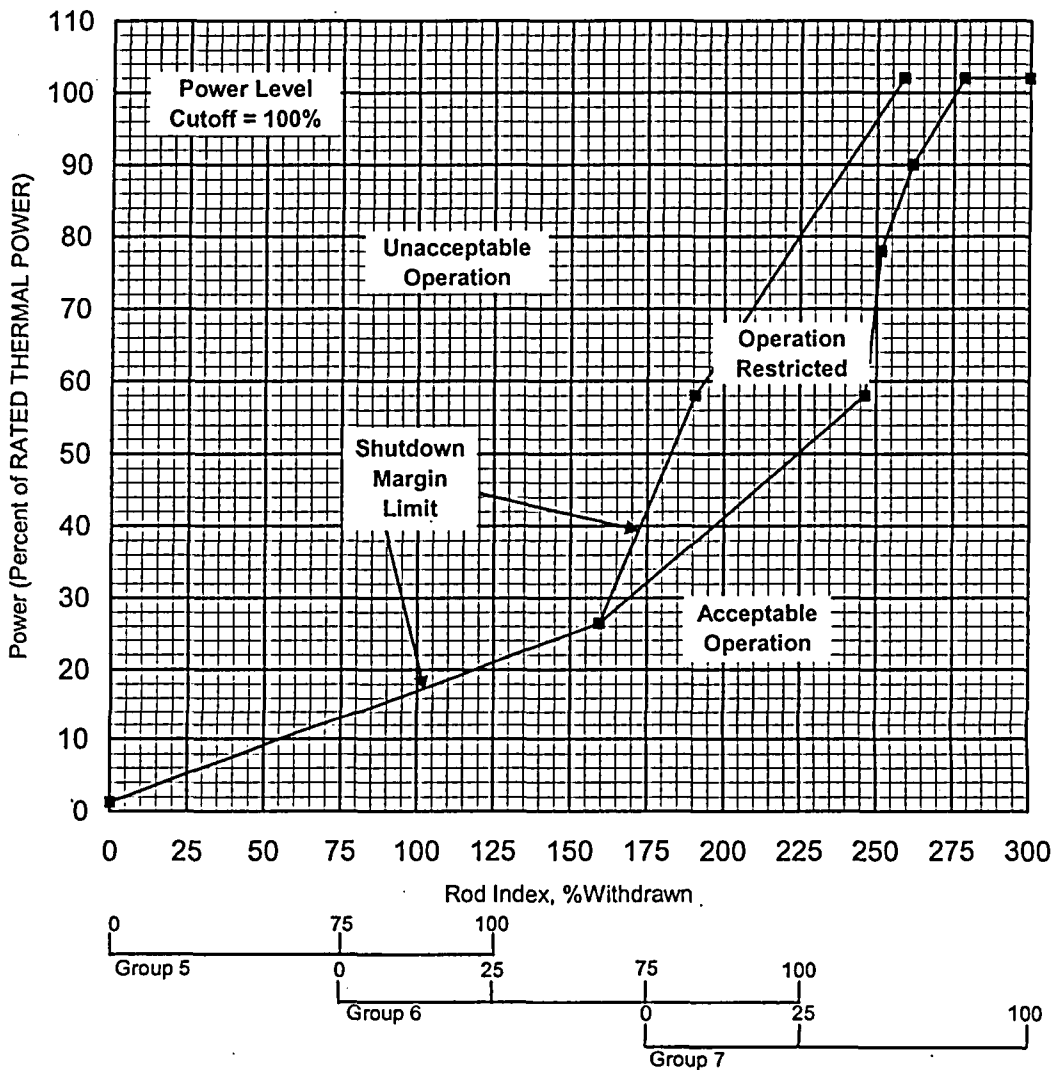
This Figure is referred to by Technical Specifications 3.1.3.6 and 3.1.3.8



Note 1: A Rod Group overlap of 25 +/- 5% between sequential withdrawn groups 5 and 6, and 6 and 7, shall be maintained.
 Note 2: Instrument error is accounted for in these Operating Limits.

Figure 1b Regulating Group Position Operating Limits
 After 300 ±10 EFPD, Four RC Pumps -2772 MWt RTP
 Davis-Besse 1, Cycle 15

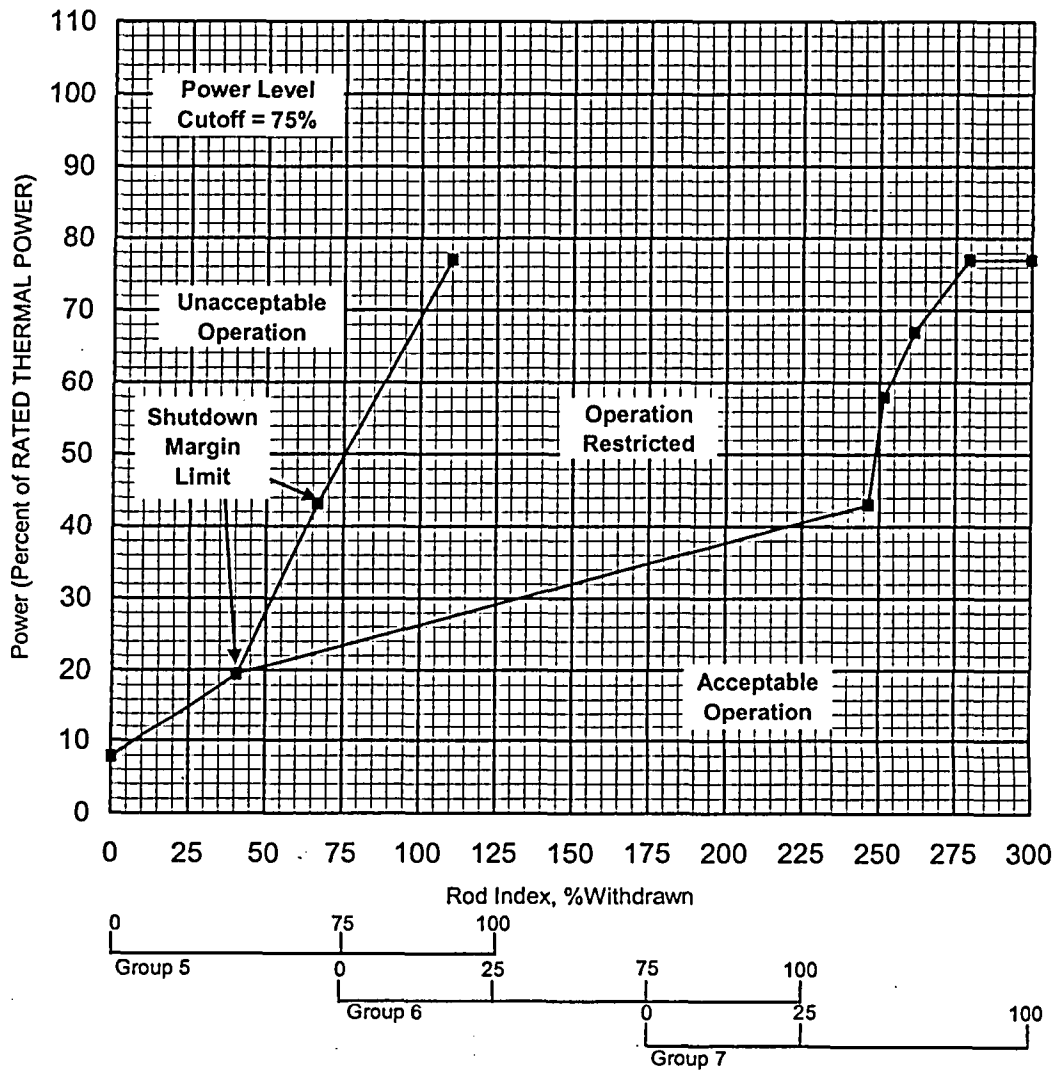
This Figure is referred to by Technical Specifications 3.1.3.6 and 3.1.3.8



Note 1: A Rod Group overlap of 25 ±5% between sequential withdrawn groups 5 and 6, and 6 and 7, shall be maintained.
 Note 2: Instrument error is accounted for in these Operating Limits.

Figure 1c Regulating Group Position Operating Limits
 0 to 300 ±10 EFPD, Three RC Pumps—2772 MWt RTP
 Davis-Besse 1, Cycle 15

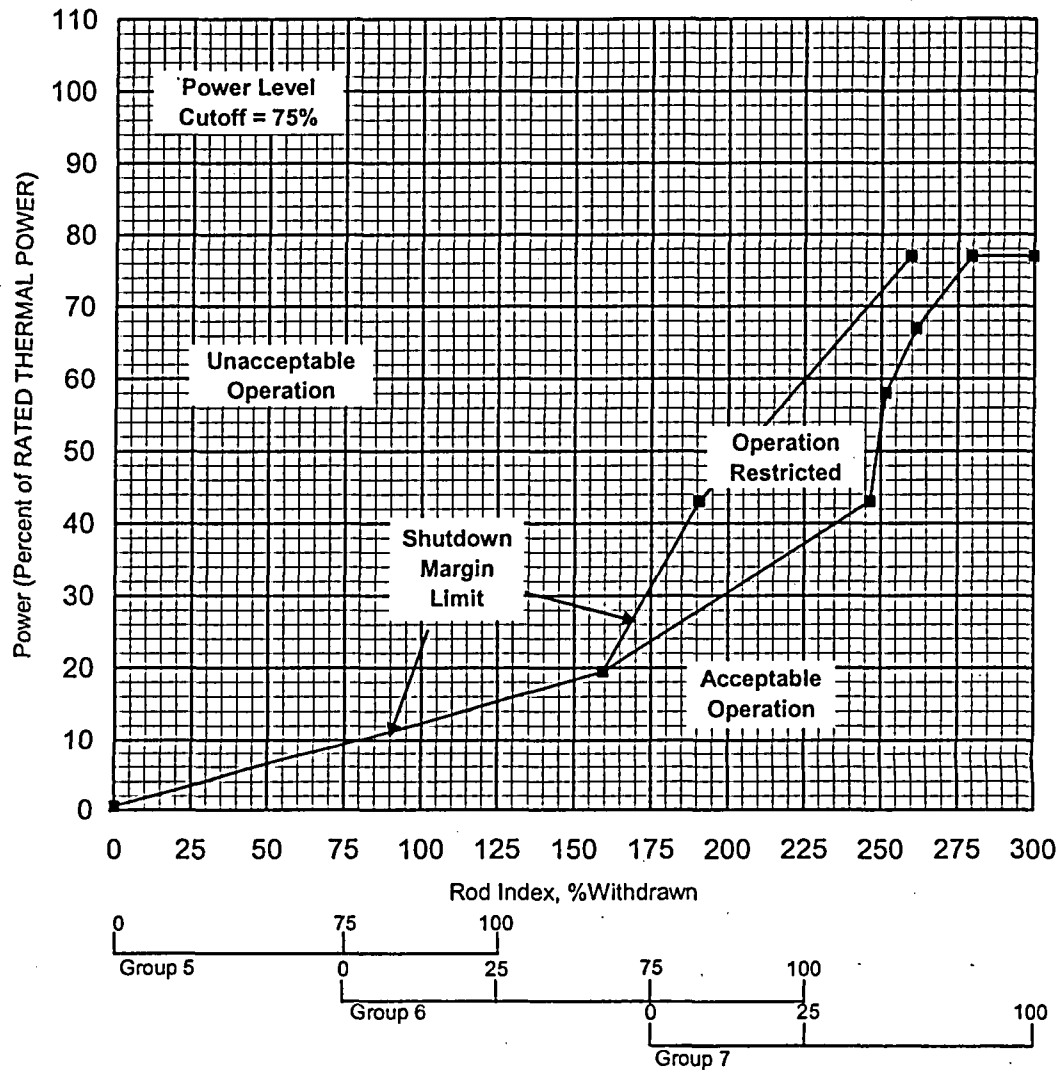
This Figure is referred to by Technical Specifications 3.1.3.6 and 3.1.3.8



Note 1: A Rod Group overlap of 25 +/- 5% between sequential withdrawn groups 5 and 6, and 6 and 7, shall be maintained.
 Note 2: Instrument error is accounted for in these Operating Limits.

Figure 1d Regulating Group Position Operating Limits
 After 300 ±10 EFPD, Three RC Pumps –2772 MWt RTP
 Davis-Besse 1, Cycle 15

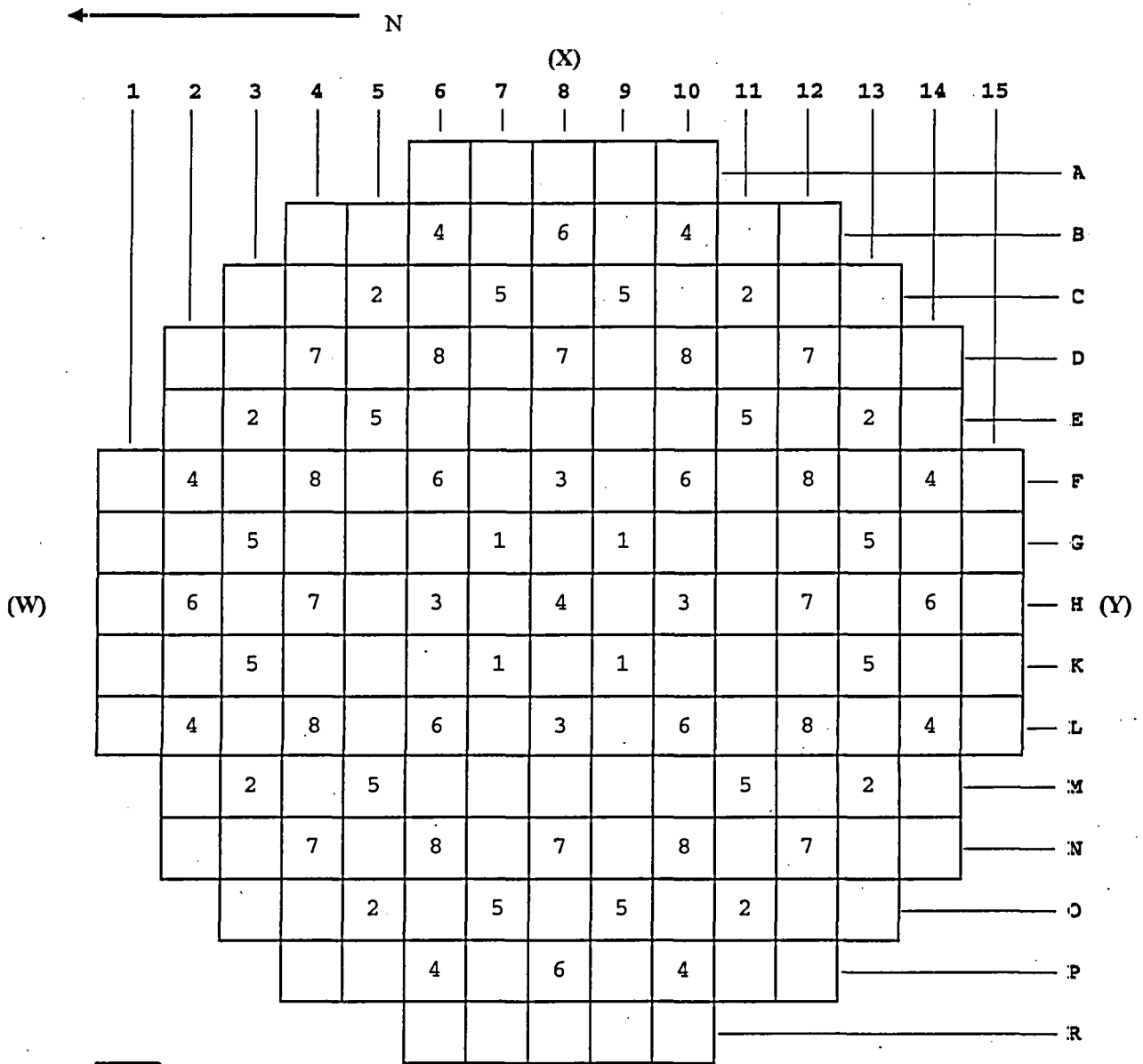
This Figure is referred to by Technical Specifications 3.1.3.6 and 3.1.3.8



Note 1: A Rod Group overlap of 25 ±5% between sequential withdrawn groups 5 and 6, and 6 and 7, shall be maintained.
 Note 2: Instrument error is accounted for in these Operating Limits.

Figure 2 Control Rod Core Locations
and Group Assignments
Davis-Besse 1, Cycle 15

This Figure is referred to by Technical
Specification 3.1.3.7



X Group Number			(Z)		
Group	No. of Rods	Function	Group	No. of Rods	Function
1	4	Safety	5	12	Control
2	8	Safety	6	8	Control
3	4	Safety	7	8	Control
4	9	Safety	8	8	APSRs
			Total	61	

Figure 3 APSR Position Operating Limits—2772 MWt RTP

This Figure is referred to by Technical
Specification 3.1.3.9

**Before APSR Pull: 0 EFPD to 620 ± 10 EFPD,
Three or Four RC pumps operation***

Lower Limit: 0 %WD

Upper Limit: 100 %WD

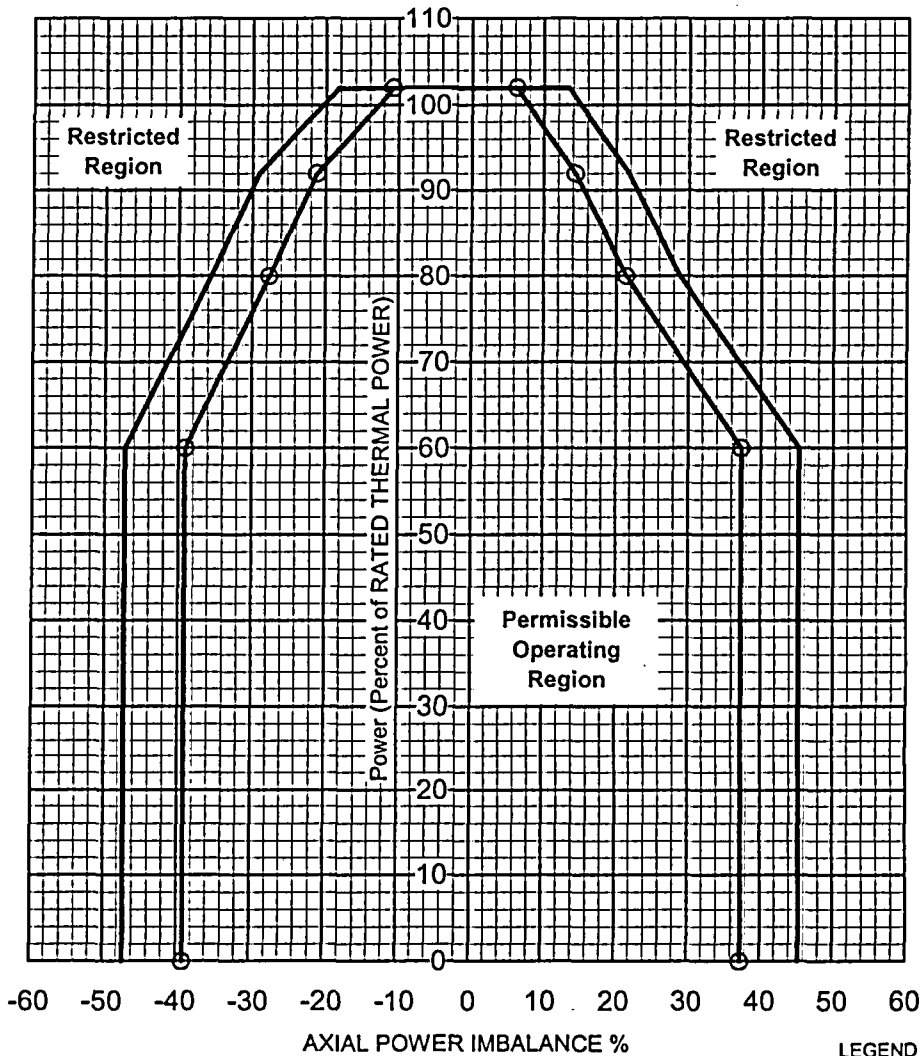
**After APSR Pull: 620 ± 10 EFPD to End-of-Cycle
Three or Four RC pumps operation***

Insertion Prohibited (maintain $\geq 99\%$ WD)

*** Power restricted to 77% for 3-pump operation.**

Figure 4a AXIAL POWER IMBALANCE Operating Limits
 0 to 300 ±10 EFPD, Four RC Pumps--2772 MWt RTP
 Davis-Besse 1, Cycle 15

This Figure is referred to by
 Technical Specification 3.2.1

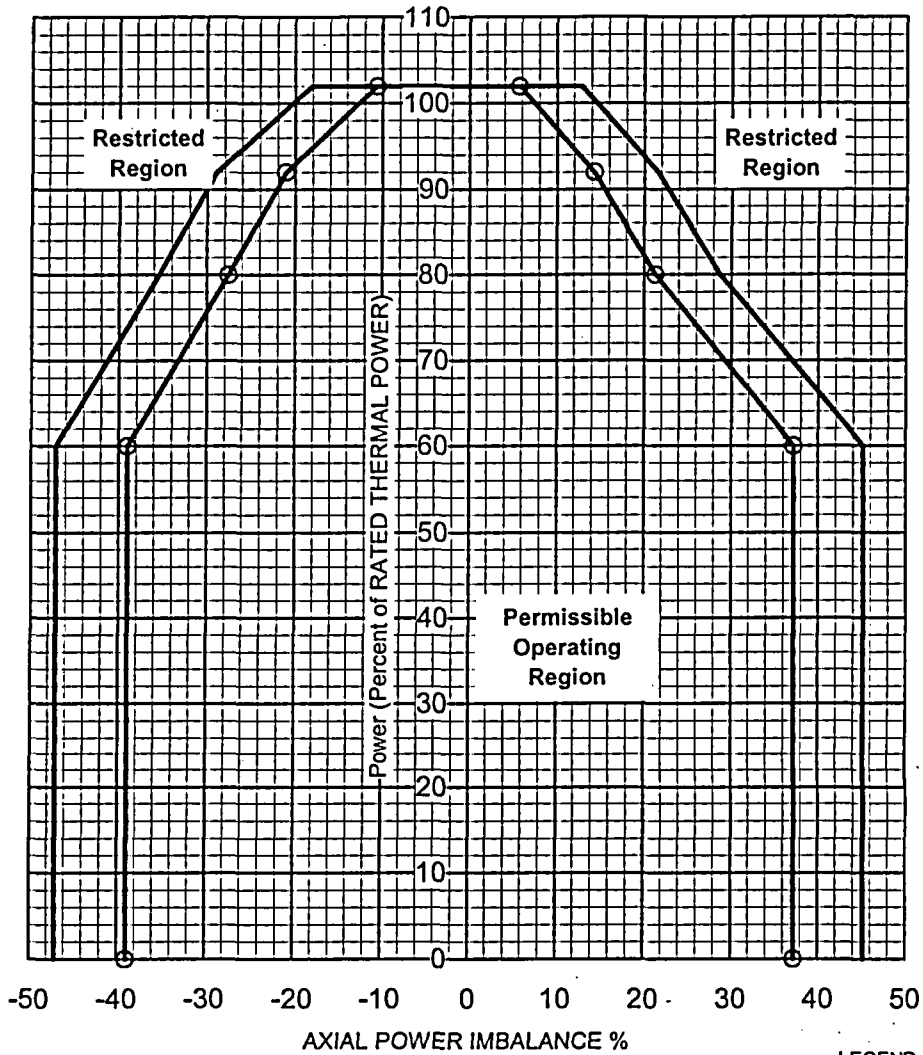


Note 1: Instrument error is accounted for in these Operating Limits.

LEGEND
FULL INCORE
 EXCORE

Figure 4b AXIAL POWER IMBALANCE Operating Limits
 300 ±10 to 600 ±10 EFPD, Four RC Pumps-2772 MWt RTP
 Davis-Besse 1, Cycle 15

This Figure is referred to by
 Technical Specification 3.2.1

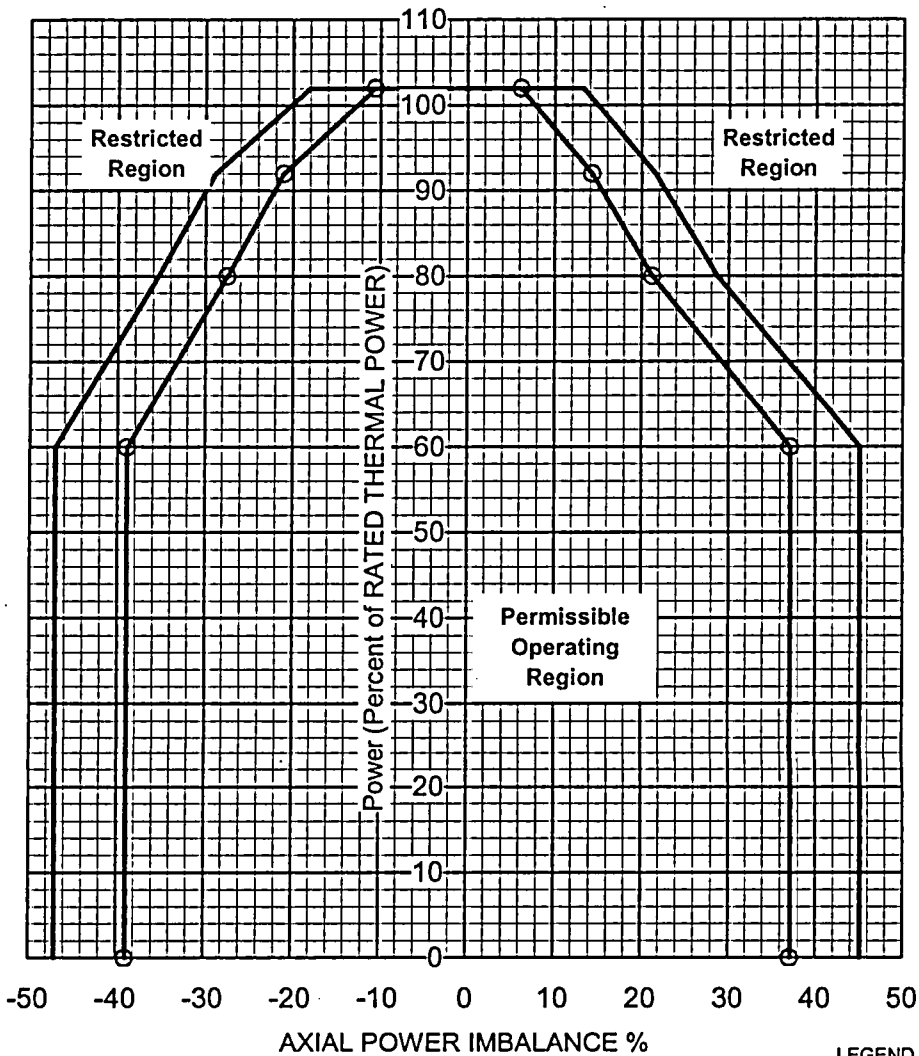


Note 1: Instrument error is accounted for in these Operating Limits.

LEGEND
 FULL INCORE
 EXCORE

Figure 4c AXIAL POWER IMBALANCE Operating Limits
 After 600 ±10 EFPD, Four RC Pumps--2772 MWt RTP
 Davis-Besse 1, Cycle 15

This Figure is referred to by
 Technical Specification 3.2.1

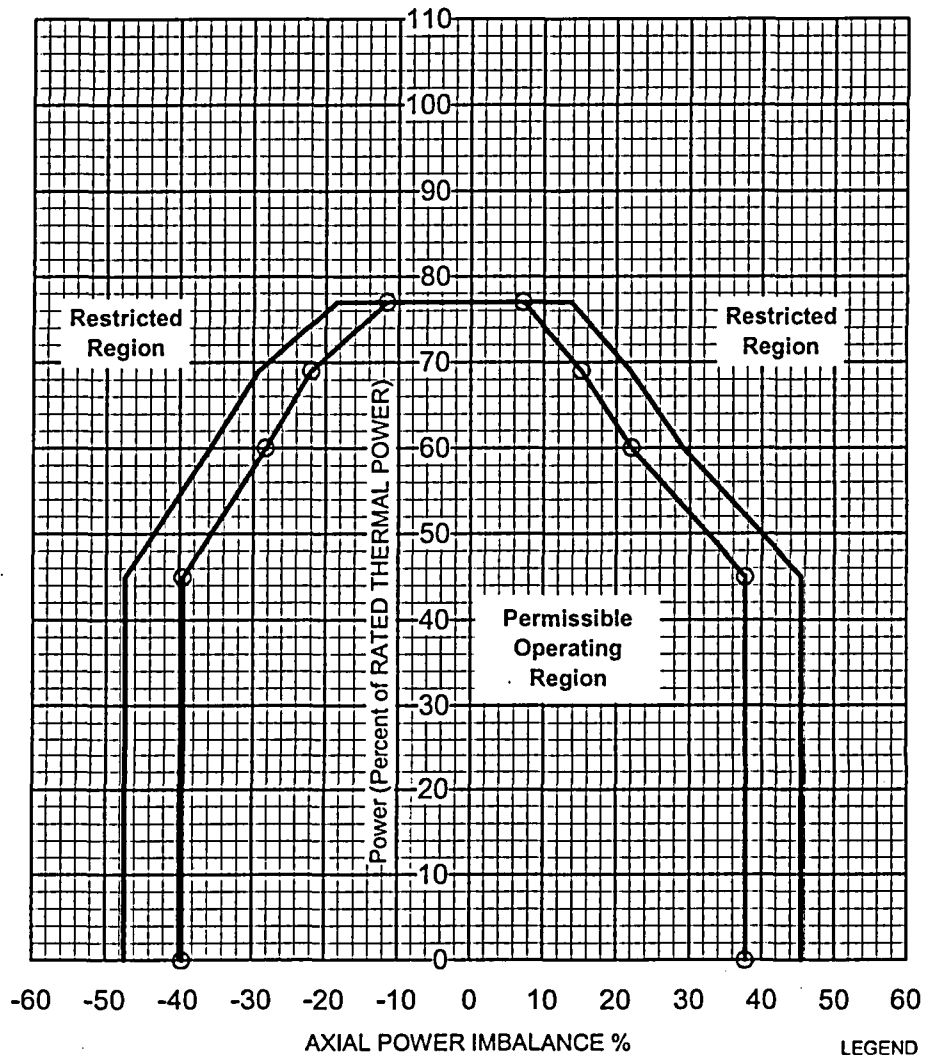


Note 1: Instrument error is accounted for in these Operating Limits.

LEGEND
FULL INCORE
 EXCORE ○

Figure 4d AXIAL POWER IMBALANCE Operating Limits
 0 to 300 ±10 EFPD, Three RC Pumps--2772 MWt RTP
 Davis-Besse 1, Cycle 15

This Figure is referred to by
 Technical Specification 3.2.1

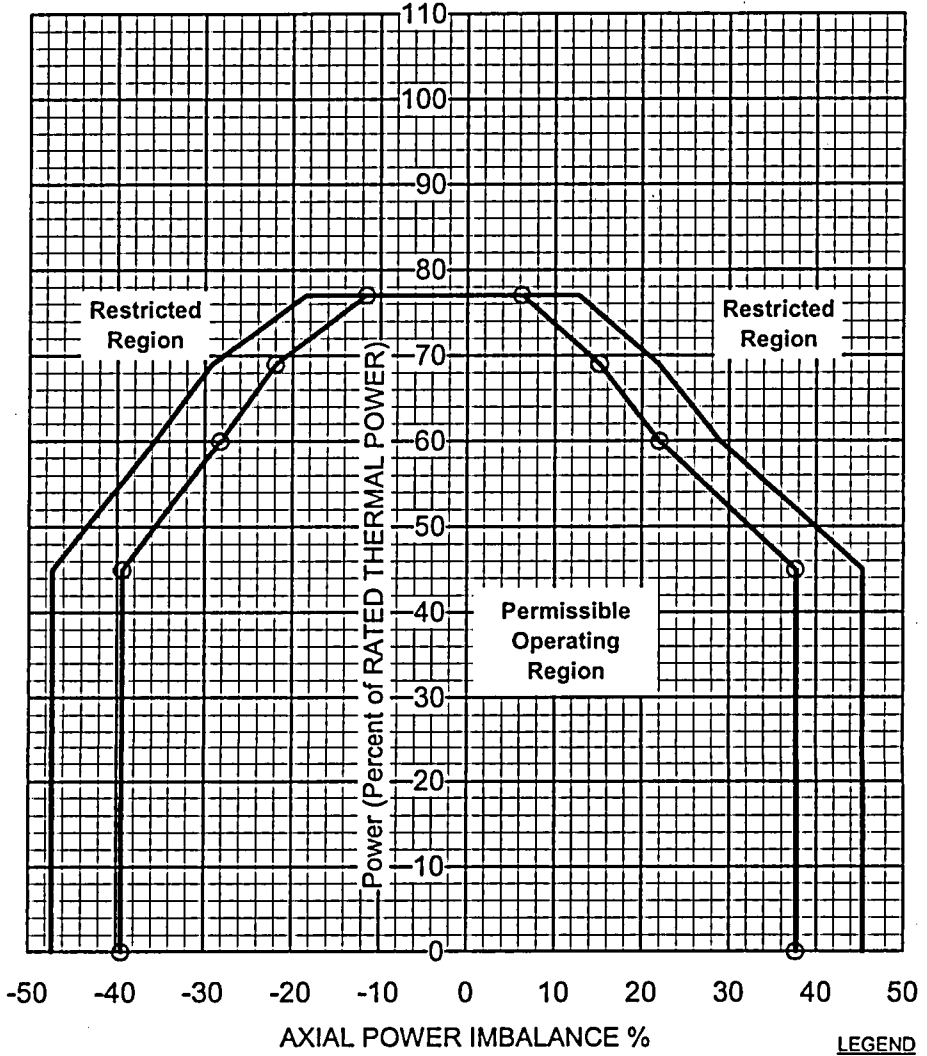


Note 1: Instrument error is accounted for in these Operating Limits.

LEGEND
FULL INCORE
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Figure 4e AXIAL POWER IMBALANCE Operating Limits
 300 ±10 to 600 ±10 EFPD, Three RC Pumps--2772 MWt RTP
 Davis-Besse 1, Cycle 15

This Figure is referred to by
 Technical Specification 3.2.1



Note 1: Instrument error is accounted for in these Operating Limits.

LEGEND
FULL INCORE
 EXCORE

Figure 4f AXIAL POWER IMBALANCE Operating Limits
 After 600 ±10 EFPD, Three RC Pumps--2772 MWt RTP
 Davis-Besse 1, Cycle 15

This Figure is referred to by
 Technical Specification 3.2.1

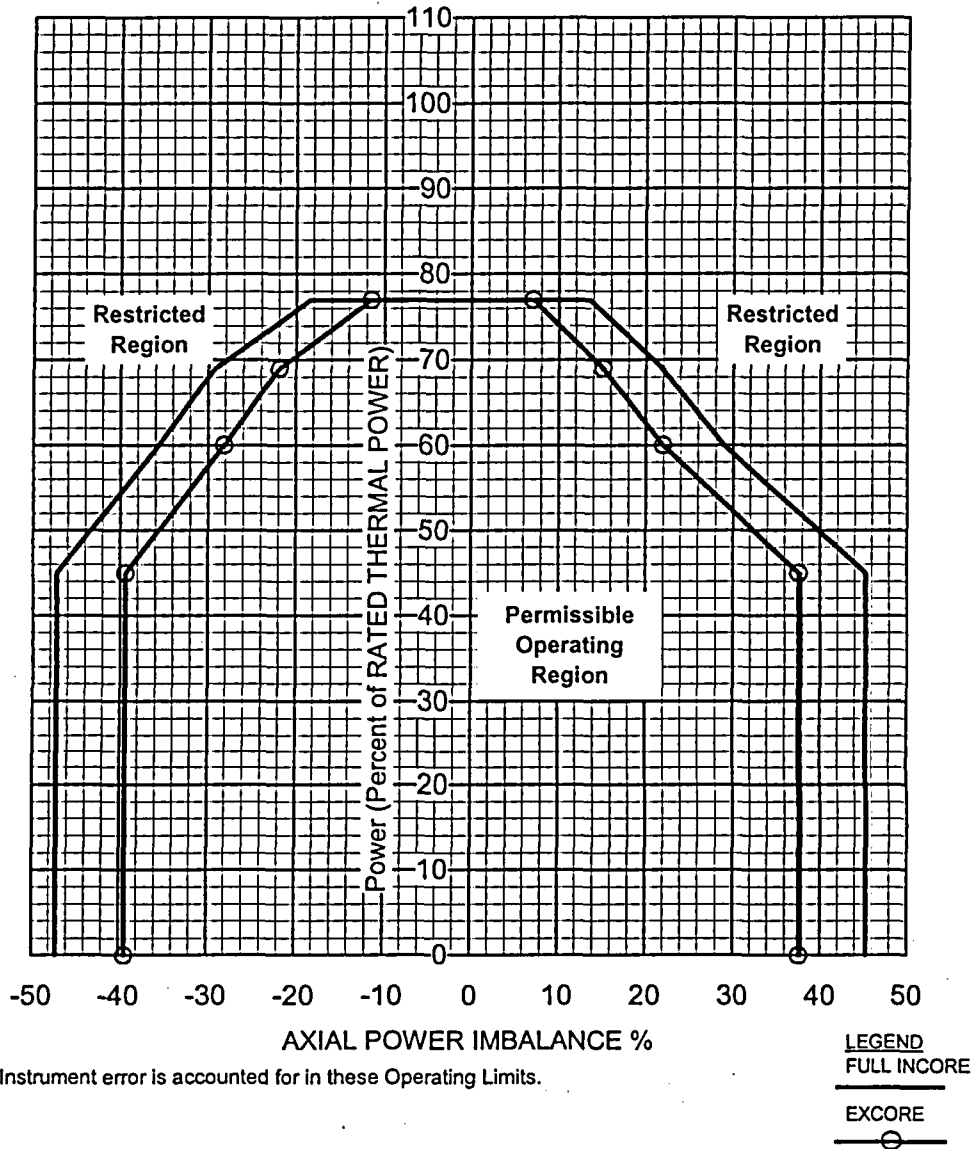
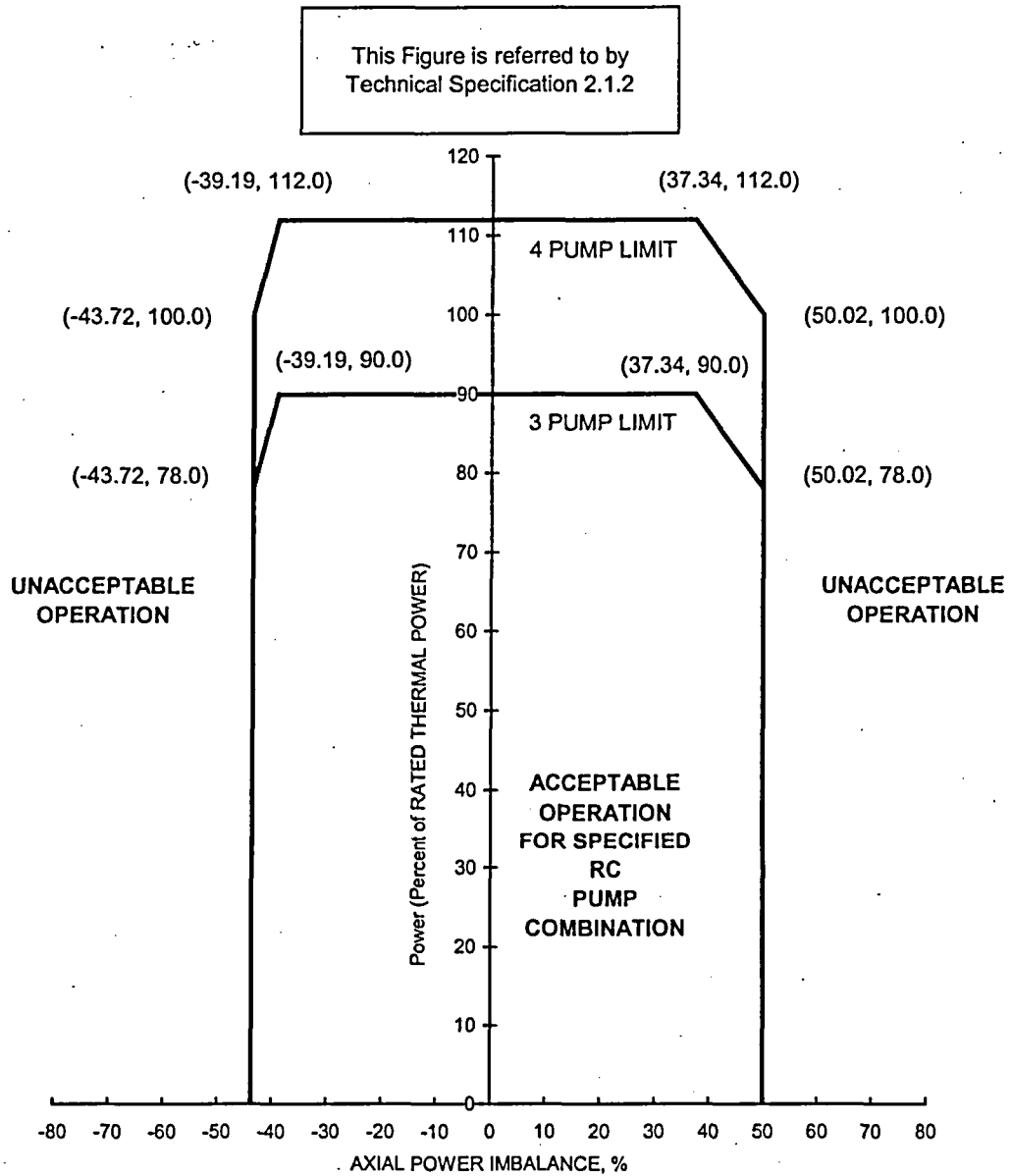


Figure 5 AXIAL POWER IMBALANCE Protective Limits
 2772 MWt RTP



<u>Pumps Operating</u>	<u>Reactor Coolant Flow, gpm.</u>	<u>Required Measured Flow to Ensure Compliance, gpm</u>
4	380,000	389,500
3	283,860	290,957

Figure 6 Flux- Δ Flux/Flow
 (or Power/Imbalance/Flow)
 Allowable Values

This Figure is referred to by
 Technical Specification 2.2.1

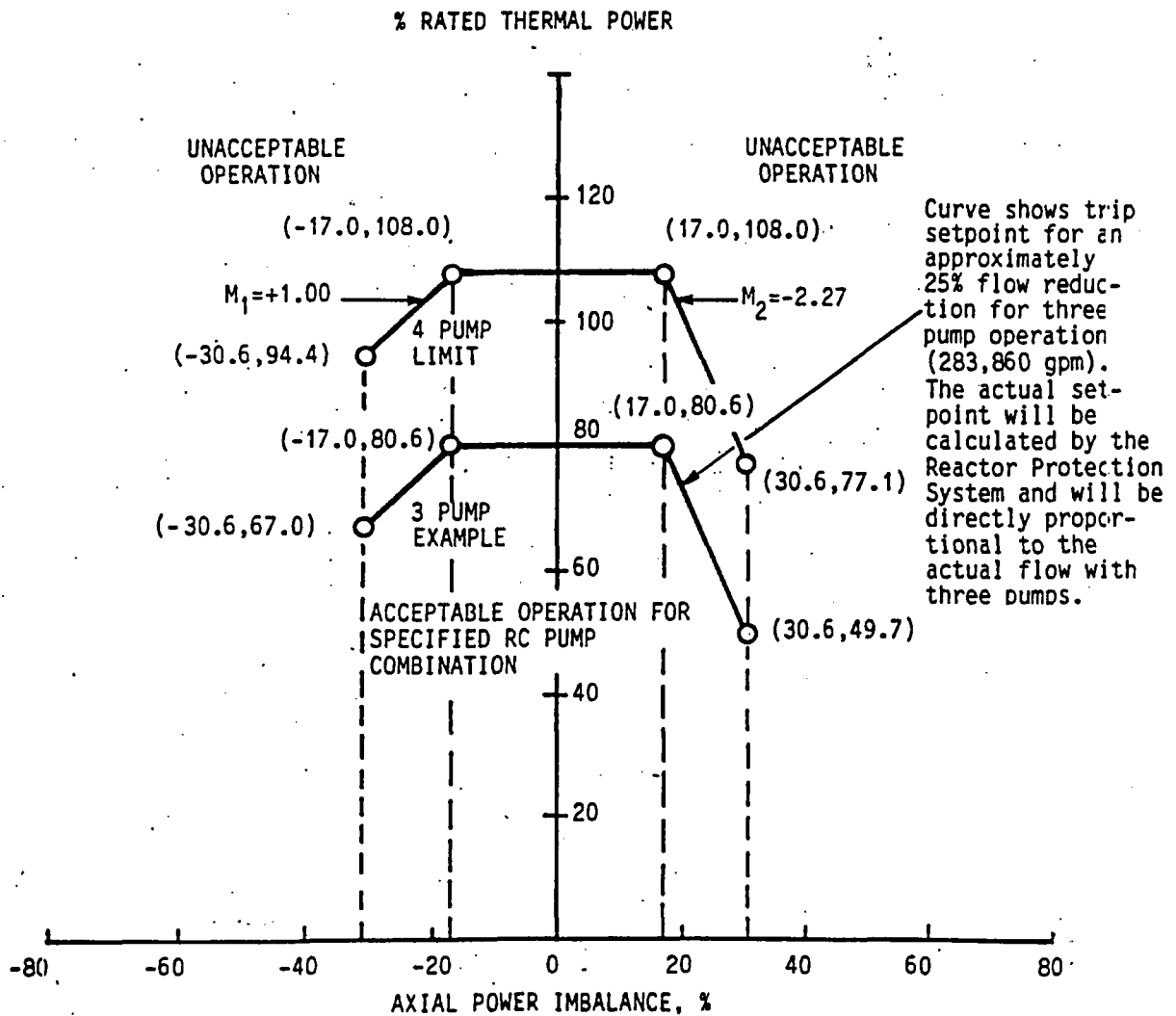


Table 1 QUADRANT POWER TILT Limits

2772 MWt RTP

This Table is referred to by Technical Specification 3.2.4

QUADRANT POWER TILT as measured by:	From 0 EFPD to EOC-15			
	Steady-state Limit for THERMAL POWER $\leq 60\%$ (%)	Steady-state Limit for THERMAL POWER $> 60\%$ (%)	Transient Limit (%)	Maximum Limit (%)
Symmetric Incore Detector System	7.90	4.31	10.03	20.00

Table 2 Negative Moderator Temperature Coefficient Limit

This Table is referred to by Technical Specification 3.1.1.3c

Negative Moderator Temperature Coefficient Limit
(@ RATED THERMAL POWER)

$-3.71 \times 10^{-4} \Delta k/k/^{\circ}F$

Table 3 Power To Melt Limits

This Table is referred to by Technical Specification Bases B2.1

	<u>Batch 9J</u>	<u>Batch 14A1</u>	<u>Batch 15</u>	<u>Batch 16</u>	<u>Batch 17</u>
Fuel Assembly Type	Mark-B8A	Mark-B10M	Mark-B10K	Mark-B12	Mark-B-HTP
Minimum linear heat rate to melt, kW/ft	20.5	22.3 (20.8) ^(a)	22.1 (21.1) ^(b) (19.3) ^(c)	22.1 (20.3) ^(d) (19.3) ^(e)	22.1 (20.1) ^(f) (19.1) ^(g)

- (a) Limit for 3 wt% Gd₂O₃ rods – Batch 14A1
- (b) Limit for 2 wt% Gd₂O₃ rods – Batch 15
- (c) Limit for 8 wt% Gd₂O₃ rods – Batch 15
- (d) Limit for 4 wt% Gd₂O₃ rods – Batch 16
- (e) Limit for 8 wt% Gd₂O₃ rods – Batch 16
- (f) Limit for 4 wt% Gd₂O₃ rods – Batch 17
- (g) Limit for 8 wt% Gd₂O₃ rods – Batch 17

Table 4a Nuclear Heat Flux Hot Channel Factor - F_Q (NAS)

2772 MWt RTP

This Table is referred
to by Technical Specification 3.2.2

Heat Flux Hot Channel Factor F_Q

F_Q shall be limited by the following relationships:

$$F_Q \leq \text{LHR}^{\text{ALLOW}}(\text{Bu}) / [\text{LHR}^{\text{AVG}} * P] \quad (\text{for } P \leq 1.0)$$

$\text{LHR}^{\text{ALLOW}}(\text{Bu})$: See Tables below

$\text{LHR}^{\text{AVG}} = 6.3095$ kW/ft for Mark-B8A fuel

$\text{LHR}^{\text{AVG}} = 6.4201$ kW/ft for Mark-B10M fuel

$\text{LHR}^{\text{AVG}} = 6.3183$ kW/ft for Mark-B10K fuel

$\text{LHR}^{\text{AVG}} = 6.3183$ kW/ft for Mark-B12 fuel

$\text{LHR}^{\text{AVG}} = 6.3294$ kW/ft for Mark-B-HTP fuel

P = ratio of THERMAL POWER / RATED THERMAL POWER

Bu = Fuel Burnup (MWd/mtU)

Batch 9J (Mark-B8A) – UO_2 Fuel $\text{LHR}^{\text{ALLOW}}$ kW/ft ^(a)

Axial Segment	0 MWd/mtU	24,500 MWd/mtU	52,000 MWd/mtU	60,000 MWd/mtU
1	16.1	16.1	12.0	10.2
2	15.8	15.8	12.0	10.2
3	15.0	15.0	12.0	10.2
4	15.0	15.0	12.0	10.2
5	15.4	15.4	12.0	10.2
6	15.4	15.4	12.0	10.2
7	14.6	14.6	12.0	10.2
8	14.3	14.3	12.0	10.2

Batch 14A1 (Mark-B10M) – UO_2 Fuel $\text{LHR}^{\text{ALLOW}}$ kW/ft ^(a)

Axial Segment	0 MWd/mtU	35,000 MWd/mtU	62,000 MWd/mtU
1	17.6	16.8	12.8
2	17.5	16.7	12.8
3	17.0	15.6	12.8
4	16.6	15.3	12.8
5	16.0	15.3	12.8
6	15.3	15.3	12.8
7	14.7	14.7	12.8
8	14.5	14.5	12.8

^(a) Linear interpolation for allowable LHR between specified burnup points is valid for these tables.

Table 4a, (Continued)

Batch 15A,B,C2 (Mark-B10K) – UO₂ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Axial Segment</u>	<u>0</u> <u>MWd/mtU</u>	<u>35,000</u> <u>MWd/mtU</u>	<u>58,000</u> <u>MWd/mtU</u>	<u>59,000</u> <u>MWd/mtU</u>	<u>60,000</u> <u>MWd/mtU</u>	<u>62,000</u> <u>MWd/mtU</u>
1	17.6	16.8	14.7	14.4	14.1	13.5
2	17.5	16.7	14.7	14.4	14.1	13.5
3	17.0	15.6	14.6	14.4	14.1	13.5
4	16.6	15.3	14.4	14.4	14.1	13.5
5	16.0	15.3	14.2	14.1	14.1	13.5
6	15.3	15.3	13.7	13.7	13.6	13.5
7	14.7	14.7	13.2	13.1	13.1	13.0
8	14.5	14.5	13.0	12.9	12.9	12.8

Batch 16A-E (Mark-B12) – UO₂ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Axial Segment</u>	<u>0</u> <u>MWd/mtU</u>	<u>35,000</u> <u>MWd/mtU</u>	<u>58,000</u> <u>MWd/mtU</u>	<u>59,000</u> <u>MWd/mtU</u>	<u>60,000</u> <u>MWd/mtU</u>	<u>62,000</u> <u>MWd/mtU</u>
1	17.6	16.8	14.7	14.4	14.1	13.5
2	17.5	16.7	14.7	14.4	14.1	13.5
3	17.0	15.6	14.6	14.4	14.1	13.5
4	16.6	15.3	14.4	14.4	14.1	13.5
5	16.0	15.3	14.2	14.1	14.1	13.5
6	15.3	15.3	13.7	13.7	13.6	13.5
7	14.7	14.7	13.2	13.1	13.1	13.0
8	14.5	14.5	13.0	12.9	12.9	12.8

Batch 17A-E (Mark-B-HTP) – UO₂ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Axial Segment</u>	<u>0</u> <u>MWd/mtU</u>	<u>40,000</u> <u>MWd/mtU</u>
1	17.6	17.2
2	17.5	17.1
3	17.1	16.7
4	17.0	16.7
5	16.5	16.2
6	15.9	15.8
7	15.3	15.2
8	15.1	15.0

^(a) Linear interpolation for allowable LHR between specified burnup points is valid for these tables.

Table 4b Nuclear Heat Flux Hot Channel Factor - F_Q (FIDMS)

2772 MWt RTP

This Table is referred
to by Technical Specification 3.2.2

Heat Flux Hot Channel Factor F_Q

F_Q shall be limited by the following relationships:

$$F_Q \leq \text{LHR}^{\text{ALLOW}} (\text{Bu}) / [\text{LHR}^{\text{AVG}} * P] \quad (\text{for } P \leq 1.0)$$

$\text{LHR}^{\text{ALLOW}} (\text{Bu})$: See the Tables below

$\text{LHR}^{\text{AVG}} = 6.3095 \text{ kW/ft}$ for Mark-B8A fuel

$\text{LHR}^{\text{AVG}} = 6.4201 \text{ kW/ft}$ for Mark-B10M fuel

$\text{LHR}^{\text{AVG}} = 6.3183 \text{ kW/ft}$ for Mark-B10K fuel

$\text{LHR}^{\text{AVG}} = 6.3183 \text{ kW/ft}$ for Mark-B12 fuel

$\text{LHR}^{\text{AVG}} = 6.3294 \text{ kW/ft}$ for Mark-B-HTP fuel

$P = \text{ratio of THERMAL POWER / RATED THERMAL POWER}$

$\text{Bu} = \text{Fuel Burnup (MWd/mtU)}$

Batch 9J (Mark-B8A) – UO_2 Fuel $\text{LHR}^{\text{ALLOW}}$ kW/ft ^(a)

Core Elevation (ft)	0 MWd/mtU	24,500 MWd/mtU	52,000 MWd/mtU	60,000 MWd/mtU
0.000	16.3	16.3	12.0	10.2
2.506	15.9	15.9	12.0	10.2
4.264	15.1	15.1	12.0	10.2
6.021	15.5	15.5	12.0	10.2
7.779	16.0	16.0	12.0	10.2
9.536	15.4	15.4	12.0	10.2
12.000	14.3	14.3	12.0	10.2

Batch 14A1 (Mark-B10M) – UO_2 Fuel $\text{LHR}^{\text{ALLOW}}$ kW/ft ^(a)

Core Elevation (ft)	0 MWd/mtU	35,000 MWd/mtU	62,000 MWd/mtU
0.000	17.6	16.8	12.8
2.506	17.6	16.8	12.8
4.264	17.1	15.7	12.8
6.021	16.6	15.3	12.8
7.779	16.0	15.8	12.8
9.536	15.3	15.3	12.8
12.000	14.5	14.5	12.8

^(a) Linear interpolation for allowable LHR between specified burnup points is valid for these tables.

Table 4b, (Continued)

Batch 15A,B,C2 (Mark-B10K) – UO₂ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>35,000 MWd/mtU</u>	<u>58,000 MWd/mtU</u>	<u>59,000 MWd/mtU</u>	<u>60,000 MWd/mtU</u>	<u>62,000 MWd/mtU</u>
0.000	17.6	16.8	14.7	14.4	14.1	13.5
2.506	17.6	16.8	14.7	14.4	14.1	13.5
4.264	17.1	15.7	14.7	14.4	14.1	13.5
6.021	16.6	15.3	14.4	14.4	14.1	13.5
7.779	16.0	15.8	14.2	14.1	14.1	13.5
9.536	15.3	15.3	13.7	13.7	13.6	13.5
12.000	14.5	14.5	13.0	12.9	12.9	12.8

Batch 16A-E (Mark-B12) – UO₂ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>35,000 MWd/mtU</u>	<u>58,000 MWd/mtU</u>	<u>59,000 MWd/mtU</u>	<u>60,000 MWd/mtU</u>	<u>62,000 MWd/mtU</u>
0.000	17.6	16.8	14.7	14.4	14.1	13.5
2.506	17.6	16.8	14.7	14.4	14.1	13.5
4.264	17.1	15.7	14.7	14.4	14.1	13.5
6.021	16.6	15.3	14.4	14.4	14.1	13.5
7.779	16.0	15.8	14.2	14.1	14.1	13.5
9.536	15.3	15.3	13.7	13.7	13.6	13.5
12.000	14.5	14.5	13.0	12.9	12.9	12.8

Batch 17A-E (Mark-B-HTP) – UO₂ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Core Elevation</u>	<u>0 MWd/mtU</u>	<u>40,000 MWd/mtU</u>
0.000	17.6	17.2
2.506	17.6	17.2
4.264	17.2	16.8
6.021	17.0	16.7
7.779	16.5	16.2
9.536	15.9	15.8
12.000	15.1	15.0

^(a) Linear interpolation for allowable LHR between specified burnup points is valid for these tables.

Table 4b, (Continued)

Batch 14A1 (Mark B10M)– 3 wt% Gd₂O₃ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>35,000 MWd/mtU</u>	<u>62,000 MWd/mtU</u>
0.000	16.1	15.4	11.8
2.506	16.1	15.4	11.8
4.264	15.7	14.4	11.8
6.021	15.2	14.0	11.8
7.779	14.7	14.5	11.8
9.536	14.0	14.0	11.8
12.000	13.3	13.3	11.8

Batch 15 (Mark-B10K) – 2 wt% Gd₂O₃ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>35,000 MWd/mtU</u>	<u>58,000 MWd/mtU</u>	<u>59,000 MWd/mtU</u>	<u>60,000 MWd/mtU</u>	<u>62,000 MWd/mtU</u>
0.000	16.5	15.7	13.9	13.6	13.3	12.8
2.506	16.5	15.7	13.9	13.6	13.3	12.8
4.264	16.0	14.7	13.9	13.6	13.3	12.8
6.021	15.5	14.3	13.6	13.6	13.3	12.8
7.779	15.0	14.8	13.4	13.3	13.3	12.8
9.536	14.3	14.3	13.0	12.9	12.9	12.8
12.000	13.6	13.6	12.4	12.3	12.3	12.2

Batch 15 (Mark-B10K)– 8 wt% Gd₂O₃ Fuel LHR^{ALLOW} kW/ft^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>35,000 MWd/mtU</u>	<u>58,000 MWd/mtU</u>	<u>59,000 MWd/mtU</u>	<u>60,000 MWd/mtU</u>	<u>62,000 MWd/mtU</u>
0.000	14.9	14.2	12.4	12.1	11.9	11.4
2.506	14.9	14.2	12.4	12.1	11.9	11.4
4.264	14.5	13.3	12.4	12.1	11.9	11.4
6.021	14.0	12.9	12.2	12.2	11.9	11.4
7.779	13.5	13.4	12.0	11.9	11.9	11.4
9.536	12.9	12.9	11.6	11.5	11.5	11.4
12.000	12.2	12.2	11.0	10.9	10.9	10.8

^(a) Linear interpolation for allowable LHR between specified burnup points is valid for these tables.

Table 4b. (Continued)

Batch 16 (Mark-B12)– 4 wt% Gd₂O₃ Fuel LHR^{ALLOW} kW/ft ^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>35,000 MWd/mtU</u>	<u>58,000 MWd/mtU</u>	<u>59,000 MWd/mtU</u>	<u>60,000 MWd/mtU</u>	<u>62,000 MWd/mtU</u>
0.000	15.8	15.1	13.3	13.0	12.7	12.2
2.506	15.8	15.1	13.3	13.0	12.7	12.2
4.264	15.3	14.1	13.3	13.0	12.7	12.2
6.021	14.9	13.7	13.0	13.0	12.7	12.2
7.779	14.3	14.2	12.9	12.8	12.8	12.2
9.536	13.7	13.7	12.4	12.3	12.3	12.2
12.000	13.0	13.0	11.8	11.7	11.7	11.6

Batch 16 (Mark-B12)– 8 wt% Gd₂O₃ Fuel LHR^{ALLOW} kW/ft ^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>35,000 MWd/mtU</u>	<u>58,000 MWd/mtU</u>	<u>59,000 MWd/mtU</u>	<u>60,000 MWd/mtU</u>	<u>62,000 MWd/mtU</u>
0.000	14.9	14.2	12.4	12.1	11.9	11.4
2.506	14.9	14.2	12.4	12.1	11.9	11.4
4.264	14.5	13.3	12.4	12.1	11.9	11.4
6.021	14.0	12.9	12.2	12.2	11.9	11.4
7.779	13.5	13.4	12.0	11.9	11.9	11.4
9.536	12.9	12.9	11.6	11.5	11.5	11.4
12.000	12.2	12.2	11.0	10.9	10.9	10.8

Batch 17 (Mark-B-HTP)– 4 wt% Gd₂O₃ Fuel LHR^{ALLOW} kW/ft ^(a)

<u>Core Elevation</u>	<u>0 MWd/mtU</u>	<u>40,000 MWd/mtU</u>
0.000	15.8	15.7
2.506	15.8	15.7
4.264	15.4	15.2
6.021	15.2	15.1
7.779	14.8	14.7
9.536	14.2	14.3
12.000	13.5	13.6

^(a) Linear interpolation for allowable LHR between specified burnup points is valid for these tables.

Table 4b, (Continued)

Batch 17 (Mark-B-HTP)– 8 wt% Gd₂O₃, Fuel LHR^{ALLOW} kW/ft ^(a)

<u>Core Elevation (ft)</u>	<u>0 MWd/mtU</u>	<u>40,000 MWd/mtU</u>
0.000	14.9	14.9
2.506	14.9	14.9
4.264	14.5	14.6
6.021	14.4	14.5
7.779	13.9	14.0
9.536	13.4	13.7
12.000	12.7	13.0

^(a) Linear interpolation for allowable LHR between specified burnup points is valid for these tables.

Table 5 Nuclear Enthalpy Rise Hot Channel Factor - $F_{\Delta H}^N$

This Table is referred
 to by Technical Specification 3.2.3

Enthalpy Rise Hot Channel Factor $F_{\Delta H}^N$

$$F_{\Delta H}^N \leq \text{MARP} [1 + 0.3 (1 - P/P_m)]$$

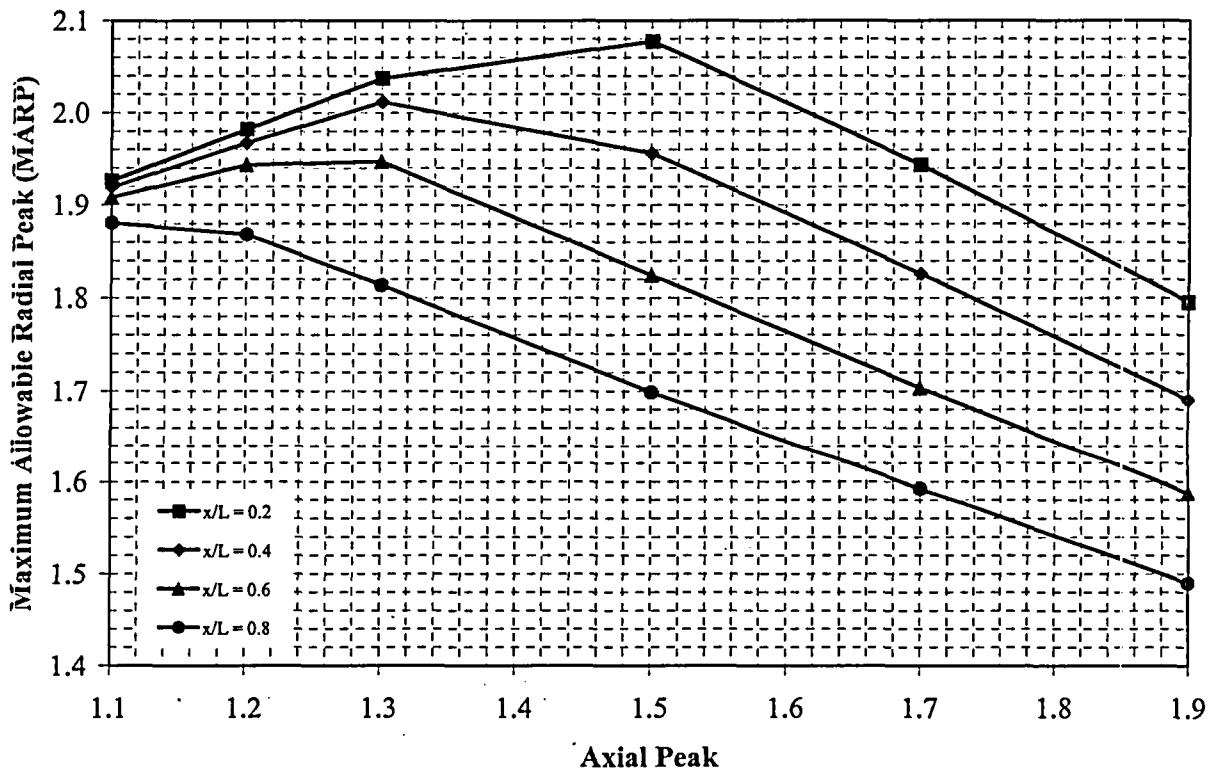
MARP = Maximum Allowable Radial Peak, see MARP Figures and data Tables

P = THERMAL POWER / RATED THERMAL POWER and $P \leq 1.0$

$P_m = 1.0$ for 4-RCP operation

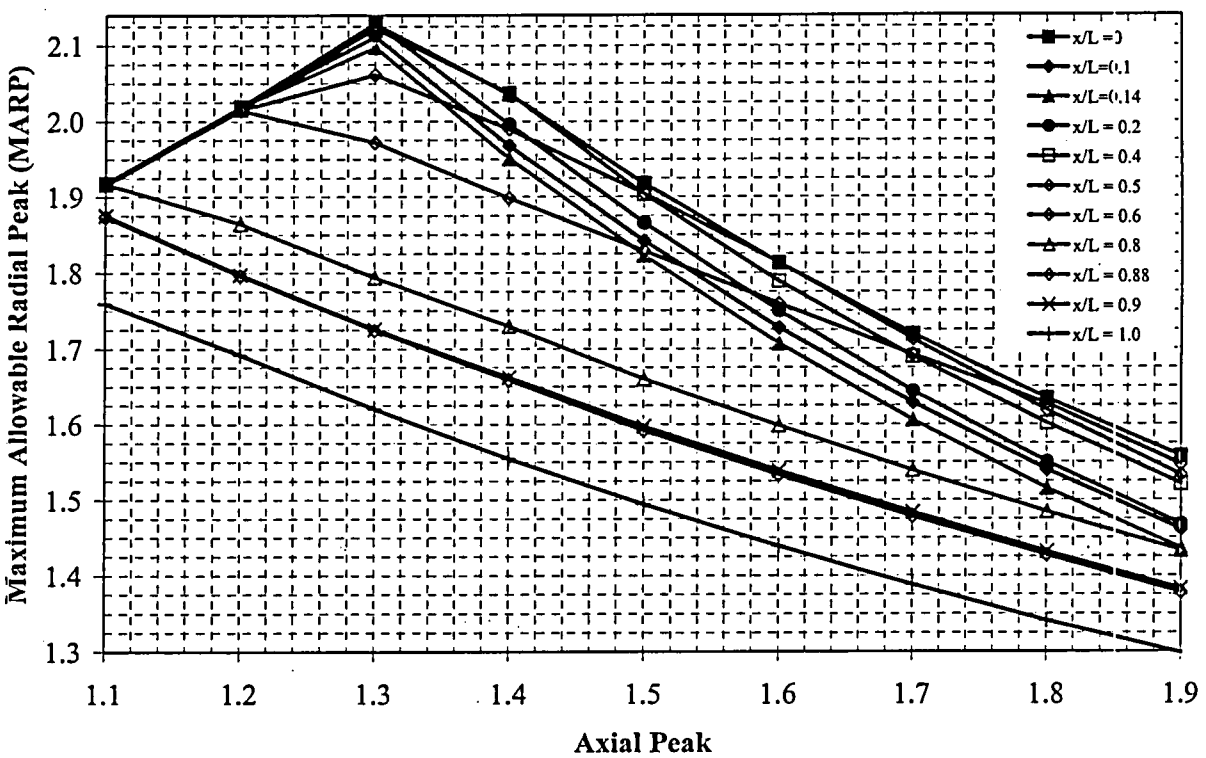
$P_m = 0.75$ for 3-RCP operation

Figure 7 Maximum Allowable Radial Peak for $F_{\Delta H}^N$ in Mark-B Fuel Assemblies



* This figure is applicable to all Mark-B fuel in the core. Linear interpolation and extrapolation above x/L of 0.8 are acceptable. For axial heights $< x/L$ of 0.2, the value at x/L 0.2 will be used.

Figure 8 Maximum Allowable Radial Peak for $F_{\Delta H}^N$ in Mark-BHTP Fuel Assemblies



* This figure is applicable to all Mark-B-HTP fuel in the core. Linear interpolation is acceptable. The MARP values are based on an active fuel height of 142.75 inches.

Table 6 Maximum Allowable Radial Peak for $F_{\Delta H}^N$ in Mark-B Fuel Assemblies

<u>Axial Peak</u>	<u>x/L</u>	<u>Axial Height (inches)</u>	<u>MAP Limit</u>	<u>MARP Limit</u>
1.1	0.2	28.12	2.0415	1.9264
	0.4	56.24	2.0345	1.9198
	0.6	84.36	2.0221	1.9081
	0.8	112.48	1.9934	1.8810
1.2	0.2	28.12	2.2918	1.9824
	0.4	56.24	2.2749	1.9678
	0.6	84.36	2.2470	1.9437
	0.8	112.48	2.1601	1.8685
1.3	0.2	28.12	2.5521	2.0378
	0.4	56.24	2.5197	2.0119
	0.6	84.36	2.4387	1.9472
	0.8	112.48	2.2719	1.8140
1.5	0.2	28.12	3.0022	2.0775
	0.4	56.24	2.8272	1.9564
	0.6	84.36	2.6367	1.8246
	0.8	112.48	2.4553	1.6991
1.7	0.2	28.12	3.1839	1.9441
	0.4	56.24	2.9911	1.8263
	0.6	84.36	2.7902	1.7037
	0.8	112.48	2.6084	1.5927
1.9	0.2	28.12	3.2865	1.7955
	0.4	56.24	3.0945	1.6906
	0.6	84.36	2.9062	1.5877
	0.8	112.48	2.7279	1.4903

Table 7 Maximum Allowable Radial Peak for $F_{\Delta H}^N$ in Mark-BHTP Fuel Assemblies

Axial Peak	x/L	Axial Height (inches)	MAP Limit	MARP Limit	Axial Peak	x/L	Axial Height (inches)	MAP Limit	MARP Limit
	0.0	0.000	2.03286	1.91828		0.0	0.000	2.79510	1.81332
	0.1	14.275	2.03375	1.91912		0.1	14.275	2.66285	1.72752
	0.14	19.985	2.03379	1.91916		0.14	19.985	2.63106	1.70690
	0.2	28.550	2.03355	1.91893		0.2	28.550	2.69787	1.75024
	0.4	57.100	2.03179	1.91727		0.4	57.100	2.75779	1.78912
1.1	0.5	71.375	2.03107	1.91659	1.6	0.5	71.375	2.79658	1.81428
	0.6	85.650	2.03099	1.91652		0.6	85.650	2.71189	1.75934
	0.8	114.200	2.03178	1.91726		0.8	114.200	2.46394	1.59848
	0.88	125.620	1.98623	1.87428		0.88	125.620	2.36299	1.53299
	0.9	128.475	1.98774	1.87570		0.9	128.475	2.37234	1.53906
	1.0	142.750	1.86518	1.76005		1.0	142.750	2.21845	1.43922
	0.0	0.000	2.33414	2.01903		0.0	0.000	2.81571	1.71924
	0.1	14.275	2.33416	2.01905		0.1	14.275	2.66718	1.62855
	0.14	19.985	2.33413	2.01902		0.14	19.985	2.63022	1.60598
	0.2	28.550	2.33248	2.01760		0.2	28.550	2.69195	1.64367
	0.4	57.100	2.33024	2.01566		0.4	57.100	2.76735	1.68971
1.2	0.5	71.375	2.32955	2.01506	1.7	0.5	71.375	2.80468	1.71250
	0.6	85.650	2.32916	2.01472		0.6	85.650	2.77165	1.69234
	0.8	114.200	2.15618	1.86510		0.8	114.200	2.52074	1.53913
	0.88	125.620	2.07666	1.79631		0.88	125.620	2.42051	1.47793
	0.9	128.475	2.07869	1.79807		0.9	128.475	2.43041	1.48398
	1.0	142.750	1.95649	1.69236		1.0	142.750	2.27373	1.38831
	0.0	0.000	2.66591	2.12863		0.0	0.000	2.83401	1.63428
	0.1	14.275	2.64485	2.11181		0.1	14.275	2.66992	1.53965
	0.14	19.985	2.62777	2.09817		0.14	19.985	2.62789	1.51542
	0.2	28.550	2.66761	2.12998		0.2	28.550	2.68772	1.54992
	0.4	57.100	2.65903	2.12313		0.4	57.100	2.7774	1.60163
1.3	0.5	71.375	2.58224	2.06182	1.8	0.5	71.375	2.80694	1.61867
	0.6	85.650	2.46978	1.97202		0.6	85.650	2.82218	1.62746
	0.8	114.200	2.24712	1.79424		0.8	114.200	2.57398	1.48433
	0.88	125.620	2.15963	1.72438		0.88	125.620	2.47403	1.42669
	0.9	128.475	2.16184	1.72615		0.9	128.475	2.48254	1.43160
	1.0	142.750	2.02976	1.62069		1.0	142.750	2.32589	1.34126
	0.0	0.000	2.74599	2.03596		0.0	0.000	2.85122	1.55767
	0.1	14.275	2.65413	1.96785		0.1	14.275	2.67423	1.46097
	0.14	19.985	2.63176	1.95126		0.14	19.985	2.62778	1.43560
	0.2	28.550	2.69213	1.99602		0.2	28.550	2.68609	1.46745
	0.4	57.100	2.74771	2.03723		0.4	57.100	2.78418	1.52104
1.4	0.5	71.375	2.68391	1.98993	1.9	0.5	71.375	2.80567	1.53278
	0.6	85.650	2.56032	1.89829		0.6	85.650	2.83113	1.54669
	0.8	114.200	2.33307	1.72980		0.8	114.200	2.62225	1.43258
	0.88	125.620	2.23716	1.65869		0.88	125.620	2.52166	1.37762
	0.9	128.475	2.24237	1.66256		0.9	128.475	2.53147	1.38298
	1.0	142.750	2.09699	1.55477		1.0	142.750	2.37605	1.29807
	0.0	0.000	2.77184	1.91811		0.0	0.000	2.85122	1.55767
	0.1	14.275	2.66225	1.84228		0.1	14.275	2.67423	1.46097
	0.14	19.985	2.63504	1.82345		0.14	19.985	2.62778	1.43560
	0.2	28.550	2.69706	1.86637		0.2	28.550	2.68609	1.46745
	0.4	57.100	2.75139	1.90396		0.4	57.100	2.78418	1.52104
1.5	0.5	71.375	2.75241	1.90467		0.5	71.375	2.80567	1.53278
	0.6	85.650	2.64282	1.82883		0.6	85.650	2.83113	1.54669
	0.8	114.200	2.39919	1.66024		0.8	114.200	2.62225	1.43258
	0.88	125.620	2.30027	1.59179		0.88	125.620	2.52166	1.37762
	0.9	128.475	2.30907	1.59788		0.9	128.475	2.53147	1.38298
	1.0	142.750	2.15988	1.49464		1.0	142.750	2.37605	1.29807

Docket Number 50-346
License Number NPF-3
Serial Number 3252
Attachment 1

COMMITMENT LIST

The following list identifies those actions committed to by the Davis-Besse Nuclear Power Station, Unit Number 1, (DBNPS) in this document. Any other actions discussed in the submittal represent intended or planned actions by the DBNPS. They are described only for information and are not regulatory commitments. Please notify Gregory A. Dunn, Manager – FENOC Fleet Licensing (330-315-7243) of any questions regarding this document or associated regulatory commitments.

COMMITMENTS	DUE DATE
None	N/A