Mr. Mano K. Nazar Senior Vice President and Chief Nuclear Officer Indiana Michigan Power Company Nuclear Generation Group One Cook Place Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 - CORRECTION OF

AMENDMENTS RE: CONVERSION TO IMPROVED TECHNICAL

SPECIFICATIONS (TAC NOS. MC2629 AND MC2630)

Dear Mr. Nazar:

On June 1, 2005, the Nuclear Regulatory Commission (NRC) issued Amendment No. 287 to Facility Operating License No. DPR-58 and Amendment No. 269 to Facility Operating License No. DPR-74 for Donald C. Cook Nuclear Plant, Units 1 and 2. The amendments converted the units' Technical Specifications to the Improved Technical Specifications format.

Subsequent to issuance, the NRC staff noted that the amendment package did not include replacement pages with the new license conditions for the operating licenses. This omission is a purely administrative error. Enclosed please find replacement pages for the units' Renewed Operating Licenses: Pages 4 and 4A for Unit 1, Pages 5 and 5A for Unit 2. This correction does not change any of the conclusion made by the NRC staff in the safety evaluation. We regret any inconvenience this administrative error may have caused you.

Sincerely,

\RA\

Peter S. Tam, Senior Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: As stated

cc w/encls: See next page

Mr. Mano K. Nazar Senior Vice President and Chief Nuclear Officer Indiana Michigan Power Company Nuclear Generation Group One Cook Place Bridgman, MI 49106

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 - CORRECTION OF

AMENDMENTS RE: CONVERSION TO IMPROVED TECHNICAL

SPECIFICATIONS (TAC NOS. MC2629 AND MC2630)

Dear Mr. Nazar:

On June 1, 2005, the Nuclear Regulatory Commission (NRC) issued Amendment No. 287 to Facility Operating License No. DPR-58 and Amendment No. 269 to Facility Operating License No. DPR-74 for Donald C. Cook Nuclear Plant, Units 1 and 2. The amendments converted the units' Technical Specifications to the Improved Technical Specifications format.

Subsequent to issuance, the NRC staff noted that the amendment package did not include replacement pages with the new license conditions for the operating licenses. This omission is a purely administrative error. Enclosed please find replacement pages for the units' Renewed Operating Licenses: Pages 4 and 4A for Unit 1, Pages 5 and 5A for Unit 2. This correction does not change any of the conclusion made by the NRC staff in the safety evaluation. We regret any inconvenience this administrative error may have caused you.

Sincerely,

## \RA\

Peter S. Tam, Senior Project Manager Plant Licensing Branch III-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

Enclosures: As stated

cc w/encls: See next page

DISTRIBUTION

PUBLIC LPLIII-1 R/F RidsNrrPMPTam RidsNrrLATHarris RidsOgcRp GHill(2) RidsAcrsAcnwMailCenter RidsNrrDorlLple

RidsRgn3MailCenter RidsNrrDirsItsb RidsNrrDorl

ADAMS Accession Number: ML061080584

OFFICE	NRR/LPL3-1/PM	NRR/LPL3-1/LA	NRR/LPL3-1/BC
NAME	PTam	THarris	LRaghavan
DATE	04/21/06	04/20/06	04/21/06

June 30, 1986, January 28, 1987, May 26, 1987, June 16, 1988, June 17, 1988, June 7, 1989, February 1, 1990, February 9, 1990, March 26, 1990, April 26, 1990, March 31, 1993, April 8, 1993, December 14, 1994, January 24, 1995, April 19, 1995, June 8, 1995, and March 11, 1996, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if these changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- (5) Deleted by Amendment No. 279
- (6) Deleted by Amendment No. 80
- (7) Deleted by Amendment No. 287
- (8) Deleted by Amendment No. 279
- (9) Deleted by Amendment No. 279
- (10) Deleted by Amendment No. 279
- (11) Deleted by Amendment No. 279
- (12) The 72 hour allowed outage time of Technical Specifications 3.1.2.4 and 3.5.2 Action "a," which was entered at 0130 on January 13, 2005, may be extended by an additional 24 hours to complete repair and testing of the 1 West Centrifugal Charging Pump.
- (13) The 72 hour allowed outage time of Technical Specifications 3.8.1.1 Action "a" may be extended to 14 days one time for the 69 kilovolt (alternate) independent offsite power circuit when it is made inoperable to complete connection of the Supplemental Diesel Generators to the existing plant electrical system and to perform upgrades to the alternate offsite power supply circuit.
- (14) Implementation of Amendment No. 287

This amendment authorizes the relocation of certain current Technical Specification requirements and operating license conditions to other licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the other documents, as described in (1) Section 5.0 of the NRC staff's Safety Evaluation and (2) Table LA of Removed Details and Table R of Relocated Specifications attached to the NRC staff's Safety Evaluation, which is enclosed with this amendment.

The schedule for the performance of new and revised Surveillance Requirements (SRs) shall be as follows:

- 4A -

For SRs that are new in this amendment, the first performance is due at the end of the first surveillance interval, which begins on the date of implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.

For SRs that existed prior to this amendment that have modified acceptance criteria, the first performance is due at the end of the first surveillance interval that began on the date the surveillance was last performed prior to the implementation of this amendment, except as noted below for SRs that have modified acceptance criteria as a result of revised allowable values.

For SRs that have modified acceptance criteria as a result of revised allowable values, the current allowable Values and current channel calibration frequencies are required to be met until the trip setpoints are changed to reflect the new allowable values and channel calibration frequencies. The trip setpoints are required to be changed no later than the unit startup after the first planned outage of sufficient duration to change all of the trip setpoints for the unit following implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being extended, the first extended surveillance interval begins upon completion of the last surveillance performed prior to implementation of this amendment, except as noted above for SRs that have modified acceptance criteria as a result of revised allowable values.

## D. Physical Protection

The Indiana Michigan Power Company shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revision to 10 CFR 73.55 (51 FR 27817 and 27822), and the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans<sup>1</sup>, which contain Safeguards Information protected under 10 CFR 73.21, is entitled:"Donald C. Cook Nuclear Plant Security Plan, Training and Qualification Plan, and Safeguards Contingency Plan, Revision 0," submitted by letter dated October 12, 2004.

- E. Deleted by Amendment No. 80
- F. Deleted by Amendment No. 80
- G. In all places of this renewed operating license, the reference to the Indiana and Michigan Electric Company is amended to read Indiana Michigan Power Company.

<sup>&</sup>lt;sup>1</sup>The Training and Qualification Plan and Safeguards Contingency Plan are Appendices to the Security Plan.

April 8, 1993, December 14, 1994, January 24, 1995, April 19, 1995, June 8, 1995, and March 11, 1996, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if these changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- (p) Deleted by Amendment No. 121
- (q) Deleted by Amendment No. 2
- (r) Deleted by Amendment No. 68
- (s) Deleted by Amendment No. 261
- (t) Deleted by Amendment No. 63
- (u) Deleted by Amendment No. 261
- (v) Deleted by Amendment No. 269
- (w) Deleted by Amendment No 261
- (x) Deleted by Amendment No. 261
- (y) Deleted by Amendment No. 261
- (z) The 72 hour allowed outage time of Technical Specifications 3.8.1.1 Action "b," which was entered at 0923 on December 7, 2003, may be extended one time by an additional 72 hours to complete repair and testing of the 2 AB diesel generator.
- (aa) The 72 hour allowed outage time of Technical Specifications 3.8.1.1 Action "a" may be extended to 14 days one time for the 69 kilovolt (alternate) independent offsite power circuit when it is made inoperable to complete connection of the Supplemental Diesel Generators to the existing plant electrical system and to perform upgrades to the alternate offsite power supply circuit.
- (bb) Implementation of Amendment No. 269

This amendment authorizes the relocation of certain current Technical Specification requirements and operating license conditions to other licensee-controlled documents. Implementation of this amendment shall include the relocation of these requirements to the other documents, as described in (1) Section 5.0 of the NRC staff's Safety Evaluation and (2) Table LA of Removed Details and Table R of Relocated Specifications attached to the NRC staff's Safety Evaluation, which is enclosed with this amendment.

The schedule for the performance of new and revised Surveillance Requirements (SRs) shall be as follows:

For SRs that are new in this amendment, the first performance is due at the end of the first surveillance interval, which begins on the date of implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being reduced, the first reduced surveillance interval begins upon completion of the first surveillance performed after implementation of this amendment.

For SRs that existed prior to this amendment that have modified acceptance criteria, the first performance is due at the end of the first surveillance interval that began on the date the surveillance was last performed prior to the implementation of this amendment, except as noted below for SRs that have modified acceptance criteria as a result of revised allowable values.

For SRs that have modified acceptance criteria as a result of revised allowable values, the current allowable Values and current channel calibration frequencies are required to be met until the trip setpoints are changed to reflect the new allowable values and channel calibration frequencies. The trip setpoints are required to be changed no later than the unit startup after the first planned outage of sufficient duration to change all of the trip setpoints for the unit following implementation of this amendment.

For SRs that existed prior to this amendment whose intervals of performance are being extended, the first extended surveillance interval begins upon completion of the last surveillance performed prior to implementation of this amendment, except as noted above for SRs that have modified acceptance criteria as a result of revised allowable values.

## D. Physical Protection

The Indiana Michigan Power Company shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revision to 10 CFR 73.55 (51 FR 27817 and 27822), and the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans<sup>1</sup>, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Donald C. Cook Nuclear Plant Security Plan, Training and Qualification Plan, and Safeguards Contingency Plan, Revision 0," submitted by letter dated October 12, 2004.

<sup>&</sup>lt;sup>1</sup>The Training and Qualification Plan and Safeguards Contingency Plan are Appendices to the Security Plan.

## CC:

Regional Administrator, Region III U.S. Nuclear Regulatory Commission Suite 210 2443 Warrenville Road Lisle, IL 60532-4351

Attorney General Department of Attorney General 525 West Ottawa Street Lansing, MI 48913

Township Supervisor Lake Township Hall P.O. Box 818 Bridgman, MI 49106

U.S. Nuclear Regulatory Commission Resident Inspector's Office 7700 Red Arrow Highway Stevensville, MI 49127

James M. Petro, Jr., Esquire Indiana Michigan Power Company One Cook Place Bridgman, MI 49106

Mayor, City of Bridgman P.O. Box 366 Bridgman, MI 49106

Special Assistant to the Governor Room 1 - State Capitol Lansing, MI 48909

Mr. John A. Zwolinski Safety Assurance Director Indiana Michigan Power Company Nuclear Generation Group One Cook Place Bridgman, MI 49106 Michigan Department of Environmental Quality
Waste and Hazardous Materials Div.
Hazardous Waste & Radiological
Protection Section
Nuclear Facilities Unit
Constitution Hall, Lower-Level North
525 West Allegan Street
P. O. Box 30241
Lansing, MI 48909-7741

Lawrence J. Weber, Plant Manager Indiana Michigan Power Company Nuclear Generation Group One Cook Place Bridgman, MI 49106

Mr. Joseph N. Jensen, Site Vice President Indiana Michigan Power Company Nuclear Generation Group One Cook Place Bridgman, MI 49106