April 18, 2006

Mr. Randall K. Edington Vice President-Nuclear and CNO Nebraska Public Power District P. O. Box 98 Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - REQUEST FOR ADDITIONAL INFORMATION

RE: ONE-TIME EXTENSION OF THE INTEGRATED LEAKAGE RATE TEST

(TAC NO. MC9732)

Dear Mr. Edington:

The Nuclear Regulatory Commission (NRC) staff has reviewed the information provided in the January 30, 2005, submittal and has determined that the additional information identified in this enclosure is required in order for the NRC staff to complete its review. The licensee requested NRC staff approval of the subject relief request by August 30, 2006. To meet that target date, the NRC staff requests that the licensee provide its response no later than May 15, 2006.

Sincerely,

/RA/

Brian Benney, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure: Request for Additional Information

cc w/encl: See next page

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Brian Benney, Project Manager Plant Licensing Branch IV Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

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OFFICE	LPL4/PM	LPL4/LA	APLA/BC	LPL4/BC
NAME	BBenney;mxr4	LFeizollahi	MRubin	DTerao
DATE	4/18/06	4/18/06		4/18/06

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REQUEST FOR ADDITIONAL INFORMATION

BY THE OFFICE OF NUCLEAR REACTOR REGULATION

INTEGRATED LEAKAGE RATE TEST INTERVAL EXTENSION

FACILITY OPERATING LICENSE NO. DPR-46

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

- 1. The analysis of external events in Appendix B of the January 30, 2006, submittal presents the estimated delta large early release frequency (Δ LERF) associated with extending the test frequency from one test in 10 years to one test in 15 years. Please provide the estimated Δ LERF (for combined internal and external events) associated with extending the test frequency from the original three tests in 10 years to one test in 15 years (using the Nuclear Energy Institute Interim Guidance.)
- 2. Clarify whether the baseline LERF value of 5.60E-7/year, reported in Appendix B, is based on internal events only, or internal and external events. If the former, provide the total LERF for internal and external events, including the impacts of the requested change.
- 3. Reconcile the low baseline LERF value of 5.60E-7/year reported in Appendix B with the much higher early containment failure frequency of 5.49E-6/year, reported in Table 6-5 of the enclosure to the January 30, 2006, submittal. Describe the definition of LERF used in the Cooper probabilistic risk analysis, and provide an accounting of how the major sequences contributing to internal and external event core damage frequency are classified (as LERF and non-LERF) using this definition.

Cooper Nuclear Station

CC:

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Chairman
Nemaha County Board of Commissioners
Nemaha County Courthouse
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Jerry C. Roberts, Director of Nuclear Safety Assurance Nebraska Public Power District P.O. Box 98 Brownville, NE 68321

Mr. John F. McCann, Director Licensing, Entergy Nuclear Northeast Entergy Nuclear Operations, Inc. 440 Hamilton Avenue White Plains, NY 10601-1813