

April 14, 2006

Mr. Randall K. Edington  
Vice President-Nuclear and CNO  
Nebraska Public Power District  
P. O. Box 98  
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - REQUEST FOR ADDITIONAL INFORMATION  
RE: RELIEF REQUEST RI-05 FOR THE FOURTH 10-YEAR INSERVICE  
INSPECTION INTERVAL (TAC NO. MD0280)

Dear Mr. Edington:

The Nuclear Regulatory Commission (NRC) staff has reviewed the information provided in the February 23, 2006, submittal and has determined that the additional information identified in this enclosure is required in order for the NRC staff to complete its review. The licensee requested NRC staff approval of the subject relief request by September 1, 2006. To meet internal scheduling dates, the NRC staff requests that the licensee provide its response no later than May 5, 2006.

Sincerely,

**/RA/**

Brian Benney, Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure: Request for Additional Information

cc w/encl: See next page

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**ADAMS ACCESSION NO: ML061030015 NRR-088**

OFFICE	LPL4/PM	LPL4/LA	LPL4/BC
NAME	BBenney	LFeizollahi	DTerao
DATE	4/14/06	4/13/06	4/14/06

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REQUEST FOR ADDITIONAL INFORMATION  
BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELIEF REQUEST RI-05 FOR THE  
FOURTH 10-YEAR INSERVICE INSPECTION INTERVAL  
FACILITY OPERATING LICENSE NO. DPR-46  
NEBRASKA PUBLIC POWER DISTRICT  
COOPER NUCLEAR STATION  
DOCKET NO. 50-298

By letter dated February 23, 2006, Nebraska Public Power District (NPPD), the licensee for Cooper Nuclear Station (CNS), submitted a request for relief for U.S. Nuclear Regulatory Commission (NRC) review and approval to address the requirements of American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, Table IWC-2500-1, Examination Category C-A, Item Number C1.10. Relief Request RI-05 asserts that compliance with the examination requirements is impractical because the geometry of the subject residual heat removal (RHR) heat exchanger shell welds prevents a meaningful ultrasonic examination from being performed on the welds. In the relief request submittal, NPPD proposed that, as an alternative to the above requirement, a VT-1 visual examination would be performed of the applicable welds each inspection interval. Additionally, a VT-2 visual examination will be performed on the shell side of the RHR heat exchanger in accordance with Table IWC-2500-1, Category C-H.

The NRC staff has reviewed the information the licensee provided that supports the proposed relief request and requires the following information from the licensee to clarify the submittal:

1. The submittal dated February 23, 2006, stated that “[a]s an alternative to the [ASME] [C]ode-required examination, CNS will perform a visual examination, VT-1, of the applicable welds each inspection interval.” Were the VT-1 visual examinations performed in the second and third inservice inspection (ISI) intervals for which this relief has been previously granted, and if so, what were the results of the examinations?
2. Please describe why the proposed VT-1 examination will continue to provide reasonable assurance of structural integrity through the end of the fourth ISI interval.

Cooper Nuclear Station

cc:

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February 2006