

May 16, 2006

Mr. Michael R. Kansler  
President  
Entergy Nuclear Operations, Inc.  
440 Hamilton Avenue  
White Plains, NY 10601

SUBJECT: PILGRIM NUCLEAR POWER STATION - REQUEST FOR ADDITIONAL  
INFORMATION REGARDING RELIEF REQUEST PRR-15, FOURTH 10-YEAR  
INSERVICE INSPECTION PROGRAM PLAN (TAC NO. MC8298)

Dear Mr. Kansler:

By letter dated June 29, 2005, Entergy Nuclear Operations, Inc. (the licensee) requested relief from requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Section XI, for Pilgrim Nuclear Power Station (Pilgrim). You requested specific austenitic reactor pressure vessel nozzle safe-end and dissimilar metal piping weld repair contingency criteria for the fourth 10-year inservice inspection interval at Pilgrim.

The Nuclear Regulatory Commission staff has been reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). A response to this RAI is requested to be provided within 30 days.

Sincerely,

**/RA/**

James J. Shea, Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-293

Enclosure: As stated

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION (RAI)

PILGRIM RELIEF REQUEST NO. PRR-15

ENTERGY NUCLEAR OPERATIONS, INC.

PILGRIM NUCLEAR POWER STATION

DOCKET NO. 50-293

By letter dated June 29, 2005, Entergy Nuclear Operations, Inc. (the licensee) requested relief from requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Section XI, for Pilgrim Nuclear Power Station (Pilgrim). You requested specific austenitic reactor pressure vessel nozzle safe-end and dissimilar metal piping weld repair contingency criteria for the fourth 10-year inservice inspection interval at Pilgrim.

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1. The Table on page 1 of the relief request states that the maximum diameter of the pipe to be overlaid is 13.38 inches yet on pages 3, 6 and 7 of the relief request, reference is made to a 29 inch O.D. nozzle. Since this overlay is for 13 inch diameter nozzles or smaller, delete all references to any size larger than 13 inches, i.e., 29 inches.
2. The table on page 1 should identify the area (in square inches) of the repair that is in contact with the low alloy steel (P-No. 3) material for each overlay.
3. On page 6 of the relief request, the statement is made, "Alloy 52 with its high chromium content provides a high level of resistance to hot cracking." Provide a justification for this statement.
4. The "Basis for the Alternative," as noted on page 6 and continuing on page 7 of the relief request under "Exception from Code Case N-504-2 Paragraph (h)," is inadequate. The relief request should discuss the basis in more detail to justify the performance of an ultrasonic examination in lieu of a radiographic examination of the weld overlay repair.

Enclosure

Pilgrim Nuclear Power Station

cc:

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Chairman  
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Pilgrim Nuclear Power Station

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