

April 27, 2006

Mr. William Levis  
Senior Vice President & Chief Nuclear Officer  
PSEG Nuclear LLC - N09  
Post Office Box 236  
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT 1 AND UNIT 2 -  
WITHDRAWAL OF RELIEF REQUEST (TAC NOS. MC8949 AND MC8950)

Dear Mr. Levis:

By letter dated November 16, 2005, PSEG Nuclear LLC (PSEG), the licensee for the Salem Nuclear Generating Station, Unit Nos. 1 and 2 (Salem), applied for relief from certain provisions of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) at Salem. The proposed relief would permit application of the ASME Code, Section VIII, Division 2, 2004 Edition, plastic analysis methodology for determining the pressure rating of the component cooling (CC) heat exchangers. The purpose of the request was to align the design pressure of the CC heat exchangers with the design pressure of the remainder of the CC system. The relief was requested pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 55a(a)(3)(i).

In a conference call on January 31, 2006, the Nuclear Regulatory Commission (NRC) staff discussed the subject application with members of your staff. The NRC staff explained that the proposed change could not be granted as a relief request pursuant to 10 CFR 50.55a(a)(3)(i) because this regulation applies to the inspection and repair/replacement activities in Sections III and XI of the ASME Code, not design under Section VIII of the ASME Code. The proposed change to the CC heat exchanger design code should be evaluated in accordance with 10 CFR 50.59 and, if necessary, a license amendment should be requested in accordance with 10 CFR 50.90. The NRC staff also discussed several technical issues related to the proposed change, in part because the CC heat exchanger does not meet all of the provisions of the ASME Code sections you propose to adopt. The NRC staff questioned the material properties used in the design calculation (which do not appear consistent with Section II of the ASME Code), the mixing of requirements from different parts of the ASME Code, and the need to consider fatigue. The NRC staff was also concerned about the lack of full radiography of the welds, which is not required by the code of record for the CC heat exchanger, but is required for the sections of the ASME Code you propose to adopt. In the conference call, members of your staff indicated that the relief request would be withdrawn.

By letter dated April 12, 2006, PSEG withdrew the relief request. Therefore, the NRC staff is discontinuing its efforts under TAC Nos. MC8949 and MC8950.

W. Levis

- 2 -

If you have any questions about this matter, please contact me at (301) 415-1321 or [snb@nrc.gov](mailto:snb@nrc.gov).

Sincerely,

**/RA/**

Stewart N. Bailey, Senior Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

cc: See next page

W. Levis

- 2 -

If you have any questions about this matter, please contact me at (301) 415-1321 or snb@nrc.gov.

Sincerely,

**/RA/**

Stewart N. Bailey, Senior Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

cc: See next page

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DATE	4/27/06	4/26/06	4/19/06	4/27/06

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Salem Nuclear Generating Station, Unit Nos. 1 and 2

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