11.2 LIQUID WASTE MANAGEMENT SYSTEMS

In general, the liquid radwaste system collects, monitors, and recycles or releases, with or without treatment where appropriate, all potentially radioactive liquid wastes produced by the station during normal operation and maintenance, as well as transient conditions. The only exception is that effluent from the treated water system (Byron only), the condensate polisher sump and the turbine building fire and oil sump, because of minimal activity levels, is normally discharged without being processed through the liquid radwaste system. Effluent from these sumps is monitored by radiation monitors that will automatically terminate sump discharge if unacceptable activity is present in the sump effluent. Corrective action can then be initiated to reroute the sump effluents to the appropriate treatment system prior to release.

11.2.1 Design Bases

11.2.1.1 Safety Design Basis

The liquid radwaste system is designed so that liquid radwaste discharged from the site will have radioactive nuclide concentrations well within the limits specified in 10 CFR 20 and 10 CFR 50, Appendix I.

Each liquid radwaste processing stream terminates in a monitor tank (see Drawing M-48A). Since the liquid radwaste system operates on a batch basis, this arrangement allows each treated batch to be sampled to assure that the treatment was sufficient. If the sample indicates that the waste needs further processing, it is recycled either through the same subsystem or through another subsystem providing a different form of treatment. If the treated waste water is not needed for reuse, the water is sent to either release tank (OWX0lT or OWX26T) for discharge. Each batch is sampled prior to discharge from the release tank to verify that its activity level is within limits for discharge. The actual discharge to the circulating water blowdown line requires manually opening a remotely operated valve with a keylocked switch. The key for the valve lock is controlled by administrative procedures.

11.2.1.2 Power Generation Design Basis

The liquid radwaste system is sized to handle maximum expected liquid waste inputs on the basis of both volume and activity as a result of normal operation, including anticipated abnormal occurrences for Units 1 and 2.

The liquid radwaste system is composed of the following two subsystems:

a. the steam generator blowdown subsystem, and

b. the nonblowdown radwaste subsystem.

These subsystems are extensively crosstied to provide a high degree of availability and reliability.

The purpose of the steam generator blowdown subsystem is to maintain the steam generator water chemistry within specified limits.

The liquid radwaste system is designed to permit recycling of plant water. The stations are designed to minimize noncontaminated inputs from leakage of service water, circulating water, and groundwater into the plant floor drain system.

A cost-benefit analysis is not required for the liquid radwaste
system. This is because Commonwealth Edison has complied with This is because Commonwealth Edison has complied with the Guides on Design Objectives for Light-Water-Cooled Nuclear Power Reactors proposed in the concluding statement of the position of the regulatory staff in Docket RM-50-2 dated February 20, 1974, pp. 25-30.

11.2.1.3 Expected Radioactive Releases

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Byron and Braidwood Nuclear stations have uprated the core power level to 3586.6 MWt. The original licensed power level was 3411 MWt. The original expected liquid radwaste effluent data The original expected liquid radwaste effluent data presented in the UFSAR is based on a power .level of 3565 MWt.

Expected annual average releases of radionuclides from the liquid radwaste system are shown in Table 11.2-1. These releases were determined by using the NUREG 0017/PWR-GALE computer program (References 1 and 2). Both the original as well as the uprated parameters describing the expected normal operation of one unit of the station are listed in Table 11.2-2. These values were used as input to the computer code for the original analyses. The impact of core uprate on the effluent releases was evaluated based on an assessment of the changes in input parameters.

Core uprate results in a maximum potential increase of 0.6% in the liquid effluent release concentrations previously reported.. Taking into consideration the accuracy and error bounds of the operational data utilized in NUREG 00017, this mall percentage change is well within the uncertainty of the calculated results of the original NUREG 0017 based expected liquid effluent concentrations presented in Table 11.2-4 which remain valid for uprate.

11.2.1.4 10 CFR 50 Comparison

Conservatively estimated annual average doses to individuals exposed to radioactive liquid effluents are given in Table 11.2-3. As can be seen from the total dose rates from-the various exposure pathways, the numerical guidelines set forth in Appendix I to 10 CFR 50 are satisfied. As discussed in Section 11.2.1.3, this assessment and Table 11.2-3 remain valid for uprate.

11.2.1.5 10 CFR 20 Comparison

Table 11.2-4 compares expected liquid effluent concentrations with 10 CFR 20 limits. It can be seen that the expected effluent concentrations are significantly below the specified limits. A discussed in Section 11.2.1.3, this assessment and Table 11.2remain valid for uprate.

11.2.1.6 Component Specifications

Table 11.2-5 gives the design parameters of various radwaste system components.

11.2.1.7 Seismic Design and Oualitv Group

The structures housing the liquid radwaste system are Safety Category I for the auxiliary building, and Safety Category II for the turbine building and radwaste building. All components (including tanks, pumps, valves, and piping) of the liquid radwaste system containing radioactive wastes are classified as Quality Group D, with the exception of the containment penetration piping out to and including the second isolation valve from the containment sump pump discharge, which is Quality Group B piping and valves (refer to Section 3.2).

11.2.1.8 Facility and Equipment Design

The liquid radwaste system is designed to minimize radiation exposure to operating personnel. Normal operations, maintenance, and nonroutine operations are discussed in the following.

11.2.1.8.1 Maintenance Operations

Wherever practicable, components of the liquid radwaste systems are segregated to the maximum extent practical. To reduce radiation exposure to maintenance personnel, components are arranged so that access to a low activity component does not necessitate passing near a high activity component. Instruments are located in low dose rate areas wherever practical to minimize the radiation exposure to maintenance personnel.

Valves, where practicable, are located outside of compartments to minimize radiation exposure from tanks or components during valve maintenance. Most radwaste pumps are equipped with mechanical seals to minimize maintenance.

In general, components which may require maintenance are capable of being flushed prior to maintenance.

11.2.1.8.2 Floor, Wall, and Ceiling Coatings

In rooms containing radioactive wastes, the floors, and as necessary, the walls and ceilings, are coated with a two-coat water base epoxy paint for ease of decontamination.

11.2.1.9 Tank Level Control

Provisions are made to preclude uncontrolled spills due to tank overflows. The following criteria apply to tanks outside the containment building which may contain radioactive fluids:

- a. Tank level instrumentation is provided on most radwaste tanks with readout devices in the radwaste control room. A high-level condition on these tanks will be annunciated.
- b. Some radwaste tanks overflow to an adjacent sump, as described in Table 11.2-9. Sumps are provided with

duplex or triplex (redundant) pumps as appropriate. Sumps are level controlled and logic is provided to start and stop pumps automatically.

c. Provisions for tank level indication, level annunciation, and overflows are given in Table 11.2-9 for all tanks outside the containment building containing potentially radioactive liquids.

11.2.1.10 Prevention of Uncontrolled Releases

Based on operating experience during normal operations, it is expected that it will be necessary to make controlled releases of contaminated steam and condensate leakage to the environment. During normal operations, these releases of radioactive liquids to the environment are from the release tank after processing, as needed, by the liquid radwaste system.

As a batch of waste is processed, the effluent is transferred to an appropriate monitor tank (e.g., blowdown monitor tanks, boric acid monitor tanks, and radwaste monitor tanks) for sampling prior to being transferred to the release tanks or being reprocessed. In the. release tanks, the liquid is mixed and sampled for activity prior to discharge. The release tanks discharge must pass through either one of two remotely controlled keylocked valves (OWX353 and OWX896 on Drawing M-48-1) to be released from the station. Limit switches supply status information on the valve position to the operator at the radwaste control panel. A radiation monitor is provided to automatically close the valves on a high radiation signal.

In addition, effluents from the treated water system, the condensate polisher sump and the turbine building fire and oil sump are released to the environment. While normally considered non-radioactive, these effluents can potentially become contaminated, and the sump effluents are monitored by radiation monitors which will halt sump pump operation if unacceptable activity levels are present in sump effluent.

11.2.1.11 ETSB-BTP 11-1 Comparison

The liquid radwaste system is designed to meet the design criteria of the former Effluent Treatment Systems Branch (ETSB), Branch Technical Position BTP 11-1, Revision 1, and meets the criteria of Regulatory Guide 1.143, as described in Appendix A.

11.2.2 System Description

The liquid radwaste system is shared by both units. Unit 1 and Unit 2, however, have separate equipment and floor drain collection sump systems. Process flow diagrams are shown in Drawing M-48A. The systems are depicted in Drawings M-48-1 through M-48-40.

Inputs to the system are separated according to origin and/or concentrations of radioactivity and chemical impurity. Separate collection tanks are provided for each input stream. The waste is routed from the collection tanks to the appropriate processing paths. The system processes the radioactive liquid waste by various combinations of filtration, evaporation (Braidwood only), and/or demineralization.

Provisions are made to bypass any processing device. The release tanks cannot be bypassed.

After being processed through the various equipment items, the purified effluent can be reused as secondary cycle makeup, or released to the environment via the circulating water blowdown line.

The liquid radwaste system is designed to handle wastes generated during design-basis operational occurrences. This is accomplished by providing sufficient process capacity within the subsystems and collection and monitor tanks of adequate size.

The liquid radwaste system consists of two crosstied subsystems:

- a. steam generator blowdown subsystem, and
- b. non-blowdown radwaste subsystem which treats the following waste streams:
	- 1. auxiliary building equipment drains,
	- 2. auxiliary building floor drains,
	- 3. chemical waste drains,
	- 4. regeneration waste drains,
	- 5. laundry (detergent) drains,
	- 6. turbine building equipment and floor drains when contaminated, and
	- 7. condensate polisher sump when unacceptably contaminated.

Expected concentrations of radioactive nuclides in the various input waste streams to the radwaste subsystems are listed in Table 11.1-6. Expected inventories of radioactive nuclides in major components of the liquid waste system are tabulated in Tables 11.1-7 through 11.1-12. Table 11.2-6 lists the annual average and maximum expected daily flows of each waste stream. The expected activities in Table 11.1-1 correspond to the annual average daily flows. The activities for the maximum daily flows vary significantly. Actual release data are available in the effluent release reports, which are prepared in accordance with the ODCM.

Table 11.2-7 lists the design-basis decontamination factors for the processing components used in the analysis of the systems.

The original steam generator blowdown prefilters were replaced with larger prefilter units. However, the expected average and maximum waste stream flows and the design basis decontamination factors for the steam generator blowdown prefilters were not revised to account for the larger prefilter volume.

11.2.2.1 Steam Generator Blowdown Subsystem

The function of the steam generator blowdown subsystem is to maintain steam generator shell side water chemistry within the \mathbf{I}

specified limits. Continuous blowdown constantly removes impurities from the steam generator. The flow rate is varied as required to maintain the steam generator water chemistry within the required limits.

At Byron, steam generator blowdown may be sent to the condensate polisher sump to improve secondary chemistry when excessive impurities are present that would quickly exhaust steam generator blowdown demineralizers.

For a further description of the steam generator blowdown subsystem, see Subsections 10.4.8 and 10.4.9.3.1.

The components of the steam generator blowdown treatment subsystem include four blowdown prefilters; four blowdown mixed bed demineralizers; four blowdown demineralizer after filters; three blowdown monitor tanks; and associated pumps, valves, and instrumentation.

11.2.2.1.1 Normal Operation

Steam generator blowdown is operated in a normal range of 15 to 90 gpm per steam generator, depending on steam generator chemistry requirements. During normal operation, blowdown is pumped from the steam generator blowdown condenser hotwells through the blowdown prefilters, the blowdown mixed-bed demineralizers, and the blowdown after filters to the condensate storage tanks or respective unit hotwell. In the event of high radioactive material in the purified effluent leaving the blowdown mixed-bed demineralizers, the effluent is diverted to the monitoring tanks. Unit I and Unit 2 blowdown is normally segregated, as the Unit 1 and Unit 2 condensate storage tanks are normally segregated.

Blowdown from each steam generator is sampled and analyzed at periodic intervals to determine:

- a. If the blowdown flow rate requires adjustment to maintain the steam generator water chemistry limits.
- b. If leakage condition exists, either at the main condenser or primary to secondary leakage within one or more steam generators so that remedial action can be taken.
- c. If the method of processing the blowdown should be changed.

The time interval between samples of the blowdown from each steam generator depends upon operating experience. \blacksquare

The effluent from each blowdown mixed bed demineralizer is directed through a blowdown afterfilter to a header which is valved so that Unit 1 effluent is normally separated from Unit 2 effluent. Conductivity of each effluent stream from the blowdown mixed bed demineralizers is monitored. The processed liquid can be routed to either Unit 1 or Unit 2 condensate storage tanks as

described above or to a monitor tank. The water in the blowdown monitor tanks is normally drained to the turbine building floor drain system. The water may also be used to sluice blowdow demineralizers or to backwash blowdown demineralizer strainers.

In addition to processing steam generator blowdown, the blowdown mixed bed demineralizers can be used for processing turbine building equipment drains, turbine building floor drains, and for the further processing of the purified effluent from the radwaste subsystems via the radwaste and blowdown monitor tanks. This practice is not recommended for normal operation.

Effluent from the blowdown prefilters of each unit can be diverted to each of the three radwaste evaporators (Braidwood only).

11.2.2.1.2 Circulating Water to Secondary System Leakage

In the event of condenser tube or tube sheet leakage, the blowdown rate may be increased to 360 gpm (180,000 lbs/hr) total per unit to keep the steam generator shell side chemistry within operating limits. The blowdown rate from the four steam generators would be approximately 90 gpm for each steam generator.

11.2.2.1.3 Primary-to-Secondary-Leakage Concurrent with Failed Fuel

The radioisotope concentration in the steam generator blowdown is given in Table 12.2-30 and Table 11.1-6. If primary to secondary leakage occurs in only one steam generator, the blowdown rate from nonleaking steam generators remains high enough to maintain chemistry specifications while the blowdown rate from the leaking steam generator may be increased to the design rate of 90 gpm.

11.2.2.1.4 Primary-to-Secondary Leakage Not Concurrent with Failed Fuel

The steam generator blowdown during primary-to-secondary leakage not concurrent with failed fuel will be processed as discussed in Subsection 11.2.2.1.3 during transient operating conditions.

11.2.2.1.5 Transient Operating Conditions

Increased blowdown may be used to keep the steam generator water chemistry within specifications.

11.2.2.2 Nonblowdown Liquid Radwaste Subsystem

This processing train collects and treats liquid radwastes from sources other than steam generator blowdown. The mode of operation is batchwise. The nonblowdown liquid radwaste subsystem includes the following input sources:

- a. auxiliary building equipment drain,
- b. auxiliary building floor drain,
- c. chemical waste drain,
- d. regeneration waste drain,
- e. laundry (detergent) drain,
- f. turbine building equipment and floor drain (when contaminated),
- g. turbine building fire and oil sump (when contaminated) (Byron only),
- h. condensate polisher sump when unacceptably contaminated, and
- i. waste treatment system (when contaminated) (Byron only).

Each drain system except the chemical waste, regeneration waste, and laundry drains, has two drain collection tanks. The chemical waste and regeneration waste drains utilize one tank each plus a shared dual purpose chemical/regeneration waste drain tank. Waste is usually collected in one of two drain tanks. The contents of the other tank may be sampled or processed. The sample is taken from the recirculation line. Chemical additions to adjust the wastewater pH or filter aids may be added to improve waste processing efficiency.

Oil separators are provided in those sumps that could potentially have oil in the water. A filter is installed downstream of each drain tank pump discharge header, or drain tank effluent is sent to vendor-installed equipment for filtration as needed.

The radwaste evaporator inlet header receives liquid wastes from the previously mentioned drain tanks. The liquid wastes entering the radwaste evaporator inlet header normally bypass the evaporators and are processed by the radwaste demineralizers or by the vendor demineralizers.

At Byron, nonessential service water to the radwaste evaporator skids has been isolated permanently. Blank plates have also been installed in the inlets to the evaporators to prevent liquid wastes from entering.

The radwaste monitor tanks collect radwaste demineralizer effluent. The tanks' contents will be mixed and sampled prior to being transferred to the release tank.

Wastewater may be routed from the radwaste monitor tanks to vendor taps in the radwaste building for additional processing, as needed, and returned to the installed radwaste system for monitored discharge.

Based on this sample and station water balance considerations, the water may be reprocessed or discharged via the release tanks.

See Table 11.2-6 for the design-basis average and maximum .waste stream flows for the various inputs that are discussed in the following. Also refer to Table 11.1-6 for the realistic source terms for these inputs.

At Byron, effluent from the condensate polisher sump, from the turbine building floor and equipment drains (collected in the turbine building fire and oil sump) and from the waste treatment system is processed by the radwaste system if the contamination exceeds effluent limits for the sumps. The sump effluent is monitored by radiation monitors to ensure that ODCM limits are maintained.

At Braidwood, effluent from the condensate polisher sump and from the turbine building floor and equipment drains is processed by the radwaste system if contamination levels exceed effluent limits. The turbine building fire and oil sump effluent is monitored by a radiation monitor to ensure that ODCM limits are maintained.

11.2.2.2.1 Auxiliary Building Equipment Drain

Input sources to the auxiliary building equipment drain tanks include the following:

- a. auxiliary building equipment drain collection sumps,
- b. reactor coolant drain tank, and
- c. spent resin tank drains (Braidwood only).

The waste is normally processed through demineralizers.

11.2.2.2.2 Auxiliary Building Floor Drain

Input sources to the floor drain tanks include leakage from pump seals and stuffing boxes, valve stem packing, equipment overflows, and spills. Oil separators are provided in the subsystem's sumps.

Input sources to the auxiliary building floor drain tanks include the following:

- a. reactor cavity sumps,
- b. containment floor drain sumps,
- c. auxiliary building floor drain sumps,
- d. fuel handling building floor drain sumps, and
- e. radwaste building sump.

The two tanks are sized to accommodate the maximum accumulation of wastes expected in 1 day. The processing flow paths are the same as in the auxiliary building equipment drain.

11.2.2.2.3 Chemical Waste Drain

Input sources to the chemical drain tank and the dual purpose chemical/regeneration drain tank include the following:

- a. laboratory drains,
- b. fuel handling building decontamination sump,
- c. samples containing tritiated water and chemicals required for analysis,
- d. drumming station sumps,
- e. boric acid processing system,
- f. primary water storage tank, and
- g. any other high-conductivity radioactive drains.

One tank is provided solely for the chemical drains. A second tank is used as a dual purpose chemical/regeneration waste drain tank. These wastes are processed through a demineralizer.

11.2.2.2.4 Regeneration Waste Drain

Input sources to the regeneration waste drain tank and the dual purpose chemical/regeneration waste drain tank include the following:

- a. spent resin sluicing drain header,
- b. drumming station decanting tank overflows (Byron only),
- c. release tanks (regeneration waste drain tank only), and
- d. tendon tunnel sumps (when determined to be a source of radiation contamination into the fire and oil sump).

The blowdown and radwaste mixed bed demineralizers are replaced as often as is required to maintain the demineralizers effluent water quality.

11.2.2.2.5 Laundry Drain

The laundry drain tank collects detergent wastes from the radioactive laundry (Braidwood only), personnel decontamination shower and the TSC drains and showers. These waste streams are sent to the release tanks for release or a radwaste demineralizer for further treatment.

11.2.2.2.6 Turbine Building Equipment Drain

Secondary system drains are divided into turbine building equipment drain and turbine building floor drain. The turbine building equipment drain system can recover a large amount of condensate grade water for station reuse.

Two turbine building equipment drain tanks receive water from the turbine building equipment drain sumps. Since this drain water is from the secondary system, the water in the turbine building equipment drains are normally uncontaminated or only very slightly contaminated. The water is normally treated in the wastewater treatment plant for discharge. There are also flowpaths from the turbine building equipment drain system to the radwaste demineralizers and to the liquid release tank.

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At Byron, in the event of excessive leakage of the primary coolant into the secondary system, the water may be processed in the waste treatment plant and returned to the release tank for discharge. At Braidwood, in the event of excessive leakage of the primary coolant into the secondary system, the contaminated water may be processed through the coalescer/carbon filters and through additional filtration as needed and discharged via the release tanks.

11.2.2.2.7 Turbine Building Floor Drain

The two turbine building floor drain tanks collect water from the turbine building floor drain sumps, condensate pit sumps, and essential service water sumps. These wastes are normally nonradioactive, except for tritium, and are released to the environment after filtration via the wastewater treatment (TR) system.

11.2.2.2.8 Turbine Building Fire and Oil Sump

Turbine building waste water collected in the fire and oil sump, including equipment and floor drain water, is monitored by a radiation monitor. Water from this sump is normally discharged to the waste treatment system for removal of oil and solids and then released to the environment via the circulating water system and blowdown line. However, if unacceptable radioactive contamination is detected, the sump pumps are automatically stopped and the water may be sent to the liquid radwaste treatment system, via the waste treatment system (Byron only). If the source of radioactive contamination is determined to be one of the tendon tunnel sumps, either tendon tunnel pump discharge can be sent to the regeneration waste drain tank for processing in the radwaste system. The water may be processed by the waste treatment plant and returned to the release tanks for discharge (Byron only). .

11.2.2.2.9 Condensate Polisher Sump

Water in the condensate polisher sump is monitored by a radiation monitor on the sump discharge. Water from this sump discharge is normally directed to the circulating water system, and then released to the environment via the blowdown line. If a high radiation signal is detected, pump operation is automatically stopped and major condensate polisher inputs into the sump are automatically isolated. If samples confirm that the water is contaminated, the operator may manually change the valve lineup to send the water to the release tank for a monitored discharge.

11.2.2.2.10 Waste Treatment System

The input to the waste treatment system is the Turbine Building Fire and Oil Sump (see 11.2.2.2.8). Water processed by the waste treatment system is normally released to the environment via

the circulating water system and blowdown line. If the radiation monitor on the Turbine Building Fire and Oil Sump should fail, an alarm will be annunciated in the radwaste control room, and the contents of the treated water system would be sampled. If the \sim sample contains radioactive contamination, the system's contents would be pumped to the liquid radwaste system.

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11.2.2.3 Operating Procedures

If the contents of a monitor tank are to be released, the required radioactivity analysis is performed prior to transferring the material to the release tank. The liquid is then pumped to a release tank where a sample is again taken and the required analysis is performed. Based on this analysis, the discharge rate is determined so that, when mixed with cooling water blowdown discharges, the water leaving the plant has a radioactivity level less than the applicable effluent concentration as stated in the Technical Specifications. A remotely operated valve with a keylocked switch may then be manually opened so that water can be discharged. The key for the valve lock is controlled by administrative procedures. As a further backup, a radiation detector monitors the liquid in the discharge line prior to the point where the liquid is mixed with the cooling water blowdown to the river. Upon detecting an abnormal level of radiation, a valve on the release tank line immediately upstream of the mixing point closes and an alarm signal is relayed to the control room. A composite sample of the cooling water blowdown is analyzed to verify that radioactive releases conform with the requirements of the Technical Specifications. Records are maintained of radioactive wastes discharged to the environment.

11.2.2.4 Performance Tests

Liquid wastes may be monitored before and after each processing step on a batch basis. The equipment is therefore subjected to continuous performance testing.

Data on specific isotope decontamination factors are not conclusive. This system was designed using conservative overall decontamination factors. These decontamination factors are based on guidelines from References 2, 4, and 5.

Through system cross-ties, redundancy of equipment, and excess storage capacity, ample provision has been made for equipment maintenance and for reprocessing treated effluents if required.

11.2.2.5 Control and Instrumentation

A large portion of the liquid radwaste system is controlled and monitored from the liquid radwaste control panel (LRCP) located in the radwaste control room. Radwaste and blowdown demineralizers and radwaste evaporator control panels and the liquid/solid radwaste interface are also located in the radwaste control room. The solid radwaste handling system control panel is located in the radwaste building.

Some subsystem operations are controlled by automatic sequencers. Instrumentation on radwaste system tanks includes, as a minimum, a high level detector for LRCP annunciation, a low level detector for pump cutoff, and LRCP level recording. The syste instrumentation is shown in detail on Drawings M-48-1 through M- l 48-40.

11.2.3 Radioactive Releases

11.2.3.1 Release Points

All liquid radwaste system effluent paths for radioactive nuclides to the environment are suitably processed, monitored, and recycled or discharged via the release tanks in accordance with procedures outlined in Subsection 11.2.2.3. The radioactive waste release line joins the circulating water blowdown line. Water from the turbine building fire and oil sump, the condensate polisher sump and the treated water system (Subsections 11.2.2.2.8, 11.2.2.2.9 and 11.2.2.2.10), if not unacceptably contaminated, is discharged after suitable treatment into the circulating water flume, and released via the blowdown line.

11.2.3.2 Dilution Factors

At 100% capacity factor and design-basis ambient air conditions, blowdown from the circulating water system serving the two units is approximately 23,000 gpm. On an average annual basis, the circulating water blowdown is expected to be approximately 13,000 gpm, or 2.6 x 10¹³ cm³ per year. The annual radionuclide release and the concentration in the cooling tower blowdown line are given in Table 11.2-4.

Circulating water blowdown enters the Rock River approximately 50 yards downstream of the intake structure, so releases do not become entrained in makeup water. The circulating water blowdown warming line to the river screen house is isolated during releases to prevent entraining radionuclides in the circulating water and essential service water makeup lines.

11.2.3.3 Estimated Annual Averaae Doses

The estimated total annual release of radionuclides in liquid effluents is given in Table 11.2-1. Using an annual dilution volume of $2.6x10^{13}$ cm³, the concentration of each nuclide in the cooling tower blowdown line can be determined. This is shown in Table 11.2-4.

Estimated annual average doses to individuals exposed to radioactive liquid effluents were calculated using the methodology of Regulatory Guide 1.109 (Reference 3). Fish consumption, drinking water, and recreational exposure pathways were considered. Annual use factors for these pathways are given in Table 11.2-8.

In order to obtain a conservative estimate of the radiation doses, no radioactive decay or dilution by river water was taken into consideration.

Estimates of doses to the whole body and to different organs are summarized in Table 11.2-3. As explained in Subsection 11.2.1.4, these estimated doses are all within Appendix I to 10 CFR 50 guidelines. Actual release data are available in the effluent release reports, which are prepared in accordance with the ODCM.

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11.2.3 Radioactive Releases

11.2.3.1 Release Points

All controlled liquid radwaste system effluent releases of radioactive nuclides to the environment are suitably processed, monitored, and recycled or discharged via the release tanks in accordance with procedures outlined in Subsection 11.2.2.3. radioactive waste release line joins the cooling pond blowdown line as indicated in Drawing M-48-1. Water from the fire and oil sump, and condensate polisher sump (Subsections 11.2.2.2.8 and 11.2.2.2.9), if not unacceptably contaminated, is discharged, after suitable treatment, into the cooling pond at the circulating water discharge canal, where it mixes with circulating water prior to release via the blowdown line.

11.2.3.2 Dilution Factors

At 100% capacity factor, blowdown from the cooling lake is expected to be 25,000 gpm on an annual average basis, or 4.98 x **1303** cm3 per year. The annual radionuclide release and the concentration in the cooling pond blowdown line are given in Table 11.2-4. Blowdown isotope concentrations were calculated using. cooling pond blowdown flow of 12,000 gpm, which is the normally expected blowdown flow rate without the use of blowdown booster pumps.

Cooling pond blowdown enters the Kankakee River approximately 50 yards downstream of the intake structure, so that releases do not become entrained in makeup water.

11.2.3.3 Estimated Annual Average Doses

The estimated total annual release of radionuclides in liquid effluents is. given in Table 11.2-1. Using an annual dilution volume of 2.4×10^{13} cm³, the concentration of each nuclide in the discharge canal can be determined. This is shown in Table 11.2-4.

Estimated annual average doses to individuals exposed to radioactive liquid effluents were calculated using the methodology of Regulatory Guide 1.109 (Reference 3). Fish consumption, drinking water, and recreational exposure pathways were considered. Annual use factors for these pathways are given -in Table 11.2-8.

In order to obtain a conservative estimate of the radiation doses, no radioactive decay or dilution by river water was taken into consideration.

Estimates of doses to the whole body and to different organs are submitted in Table 11.2-3. As explained in Subsection 11.2.1.4, these estimated doses are all within Appendix I to 10 CFR 50 guidelines. Actual release data are available in the effluent release reports, which are prepared in accordance with the ODCM.

11.2.4 References

1. Regulatory Guide 1.112, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light Water-Cooled Power Reactors," U.S. Nuclear Regulatory Commission, April 1976.

2. NUREG-0017, "Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluent from Pressurized Water Reactors (PWR-GALE Code)," Office of Standards Development, U.S. Nuclear Regulatory Commission, April 1976.

3. Regulatory Guide 1.109, "Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I," U.S. Nuclear Regulatory Commission, Revision 1, October 1977.

4. ANSI Standard N199, "Liquid Radioactive Waste Processing System for Pressurized Water Reactor Plants," American National Standards Institute, Inc., 1976.

5. WASH-1258, "Numerical Guides for Design Objectives and Limiting Conditions for Operation to Meet the Criteria 'Low as Practicable' for Radioactive Material in Light-Water-Cooled Nuclear Power Reactor Effluents," U.S. Atomic Energy Commission, 1973.

TABLE 11.2-1

EXPECTED ANNUAL AVERAGE RELEASES OF RADIONUCLIDES IN LIQUID EFFLUENTS

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TABLE 11.2-2

PARAMETERS USED IN THE GALE-PWR COMPUTER PROGRAM (ORIGINAL & UPRATED) - NOTE 1

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TABLE 11.2-2 (Cont'd)

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TABLE 11.2-2 (Cont'd)

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BYRON-UFSAR

TABLE 11.2-3

PATHWAYS DOSES FROM LIQUID EFFLUENTS

(BYRON)

^{*}All activities are assumed to take place in the discharge canal. No credit is taken for dilution of effluents in the Rock River.

BRAIDWOOD-UFSAR

TABLE 11.2-3

PATHWAYS DOSES FROM LIQUID EFFLUENTS

(BRAIDWOOD)

*All activities are assumed to take place in the discharge canal. No credit is taken for dilution of effluents in the Kankakee River.

BYRON-UFSAR

TABLE 11.2-4

COMPARISON OF EXPECTED LIQUID EFFLUENT CONCENTRATIONS TO 10 CFR 20 LIMITS

* Calculated using NUREG-0017. The release reports, which are prepared in accordance with the ODCM. the PWR-GALE computer program described in actual data are available in the effluent

** Annual average cooling tower blowdown = 29.0 cfs.

***Limits used in the comparison are those that were in effect at the time of the analysis.

11.2-24 REVISION 7 - DECEMBER 1998

BRAIDWOOD-UFSAR

TABLE 11.2-4

COMPARISON OF EXPECTED LIQUID EFFLUENT CONCENTRATIONS TO 10 CFR 20 LIMITS

- Calculated using the PWR-GALE computer program described in NUREG-0017. The actual data are available in the effluent release reports, which are prepared in accordance with the ODCM.
- ** Annual average cooling lake blowdown = 13.4 cfs per unit. Original design and without the use of CW blowdown booster pumps installed at Braidwood.
- ***Limits used in the comparison are those that were in effect at the time of the analysis.

11.2-25 REVISION 10 - DECEMBER 2004

TABLE 11.2-5

LIQUID RADWASTE SYSTEM COMPONENTS AND DESIGN PARAMETERS PER STATION

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TABLE 11.2-5,(Cont'd)

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TABLE 11.2-5 (Cont'd)

11.2-27a REVISION 7 - DECEMBER 1998

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TABLE 11.2-5 (Cont'd)

11. 2 -28 REVISION 7 - DECEMBER 1998

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TABLE 11.2-5 (Cont'd)

11. 2-29 REVISION 10 - DECEMBER 2004

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TABLE 11.2-5 (Cont'd)

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TABLE 11.2-5 (Cont'd)

NOTE: Radwaste Evaporator Components have been intentionally deleted from this table. Braidwood and Byron stations do not intend to use this equipment..

11. 2 -31 REVISION 9 - DECEMBER 2002

TABLE 11.2-6

DESIGN-BASIS ANNUAL AVERAGE AND MAXIMUM WASTE STREAM FLOWS

(Two Units)

* Based on average of 28 days primary to secondary leakage (1956 gpm/two units), 28 days condenser to secondary leakage (420 gpm/two units) and 309 days of normal operation (120 gpm/two units).

** Based on condenser to secondary leakage of 420 gpm/two units.

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TABLE 11.2-7

DESIGN-BASIS PROCESS DECONTAMINATION FACTORS

11.2-33 REVISION 10 - DECEMBER 2004

TABLE 11.2-7 (Cont'd)

Basis for Decontamination Factors:

- 1. Filters:
	- (A) Is used for filter source term calculations only.
	- (B) Is used for other calculations. (Table 1-3, NUREG-0017, PWR GALE)

2. (A) Radwaste Demineralizers: (Table 1-3, NUREG-0017, PWR GALE and Table 1 ANSI N199-1976)

(B) Blowdown Demineralizers (Table 1-3, NUREG-0017, PWR GALE)

3. Evaporators:

(Table 1-3, NUREG-0017, PWR GALE and Tablel1 ANSI N199-1976)

4. Evaporator Distillate Demineralizers:

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(Table 1-3, NUREG-0017, PWR GALE)

11. 2 -34 REVISION 3 - DECEMBER 1991

TABLE 11.2-8

CONSUMPTION FACTORS FOR THE MAXIMUM EXPOSED INDIVIDUAL

1400.0 (Infant)

* From Regulatory Guide 1.109, Table E-5 (Reference 3).

** From Offsite Dose Calculation Manual, Revision 1.2, Table D-10.

*** From HERMES as used in Zion Station annual and semiannual reports on station radioactive waste, environmental monitoring, and occupational personnel radiation exposure.

TABLE 11.2-9

SUMMARY OF TANK LEVEL INDICATION, ANNUNCIATORS, AND OVERFLOWS FOR TANKS OUTSIDE OF CONTAINMENT POTENTIALLY CONTAINING RADIOACTIVE LIQUIDS

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TABLE 11.2-9 (Cont'd)

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TABLE $11.2-9$ (Cont'd)

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TABLE 11.2-9 (Cont'd)

NOTES:

AL - Alarm Low ALL - Alarm Low Low ALLL - Alarm Low Low Low

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AH - Alarm **High** AHH - Alarm High High *General Services Panel $\leftarrow -\frac{1}{2}$

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Figures 11.2-1 through 11.2-41 have been deleted intentionally.