

Withhold Attachment 2 from Public Disclosure pursuant to 10 CFR 2.390(d)

April 3, 2006

10 CFR Part 20, Subpart E
10 CFR 50.75(g)
10 CFR 50.82
10 CFR 50.83
10 CFR Part 100

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

**DOCKETS 50-155 AND 72-043 - LICENSE DPR-6 - BIG ROCK POINT PLANT –
LETTER OF INTENT CONCERNING THE RELEASE OF PART OF THE BIG ROCK
POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE**

This letter provides written notification to the Nuclear Regulatory Commission (NRC) of Big Rock Point's intent to release a portion of the site from its 10 CFR Part 50 License (DPR-6), pursuant to the approved Big Rock Point License Termination Plan (BRP LTP).

The land area proposed for release (referred to hereafter as the "Release Area") consists of approximately 475 acres of land within the originally licensed site boundary. The originally licensed site boundary area and approximate Independent Spent Fuel Storage Installation (ISFSI) site are illustrated in the Figure in Attachment 1.

A drawing and legal description of the BRP ISFSI site and a legal description of the BRP site as originally licensed are contained in Attachment 2. The legal description of the site as originally licensed contained in Attachment 2 was developed from Big Rock Point Plant Site Property Survey drawing of the licensed area, as submitted in the original license application. Attachment 2 serves as the record of the licensed area pursuant to 10 CFR 50.75(g)(4). The information in Attachment 2 should be withheld from public disclosure pursuant to 10 CFR 2.390(d). When separated from Attachment 2, this letter may be released for public disclosure.

The BRP ISFSI site extends at least 300 meters from the ISFSI pad and consists of approximately 105 acres. The BRP ISFSI site includes the protected area (contained within a nuisance fence) and an owner controlled area around the ISFSI (Parcels A and B in Attachment 2). The area containing the ISFSI administrative building, and the utility and roadway access corridor is also included in the BRP ISFSI site.

Withhold Attachment 2 from Public Disclosure pursuant to 10 CFR 2.390(d)

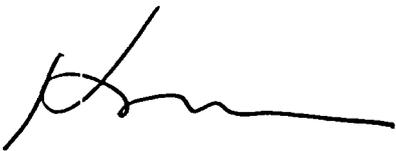
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In accordance with Section 1.4.2 of the BRP LTP, we have reviewed and assessed the impact of our proposed action to separate the release area from our 10 CFR Part 50 license. Attachment 3 contains this assessment.

Attachment 4 of this correspondence contains a preliminary Turbine Building East Excavation Survey Package. It is being included with this correspondence to continue our open communication with the NRC and to efficiently manage the application of limited Consumers Energy decommissioning resources; especially during the final period of the first phase of license termination activities.

Prior to partial site release, the unrestricted release criteria and methods to demonstrate compliance, as outlined in the BRP LTP, will be documented and submitted to the U.S. NRC. No deviation or special requests from the approved LTP are anticipated. It is our intent to forward necessary survey packages for review. A Final Status Survey (FSS) Summary Report will be submitted in the fall of 2006.

We request that the NRC be prepared to review and concur with the acceptability of the release of the subject land from the BRP Part 50 license by the end of the fourth quarter of 2006. Timely review and issuance of the requested approval as well as timely review and acceptance of FSS information is particularly important for the Big Rock Point Restoration Project. For the first time in the decommissioning of the former plant area, NRC review activities have the potential to impact end-use of the non-ISFSI land. If you should have any questions regarding this submittal, please contact me at (231) 547-8388.



Kurt M. Haas
Site General Manager

cc: Administrator, Region III U.S. NRC
NRC Region III Inspector
NMSS Project Manager
U.S. NRC State of Michigan Representative
Michigan State Police – Emergency Management Division
State of Michigan Historical Preservation Office

Page 2 of 2

Withhold Attachment 2 from Public Disclosure pursuant to 10 CFR 2.390(d)

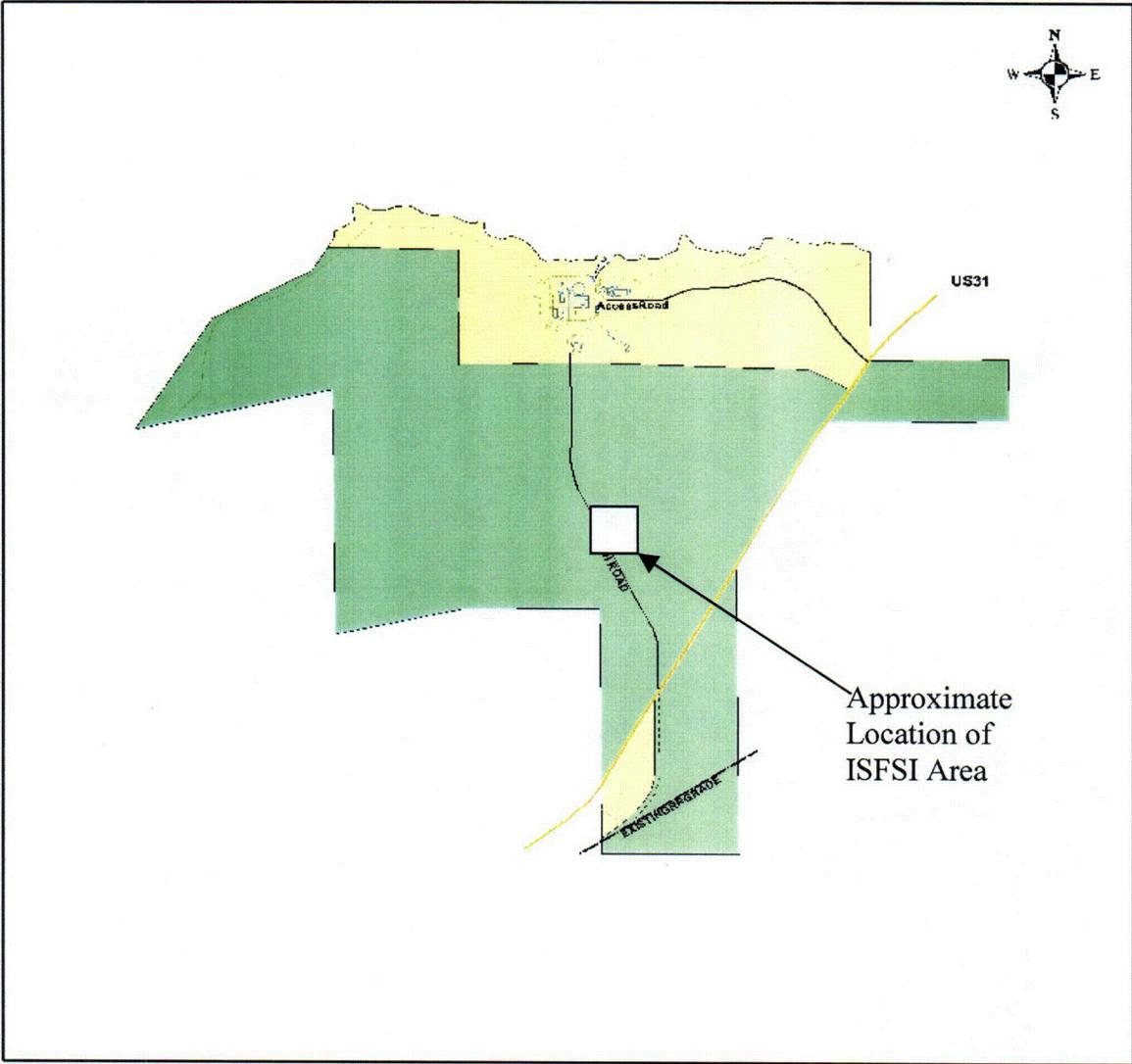
ATTACHMENT 1

**Big Rock Point Plant
April 3, 2006**

**Originally Licensed Site
And
Approximate ISFSI Area**

1 Page

Attachment 1
Originally Licensed Site and Approximate ISFSI Area



ATTACHMENT 3

**Big Rock Point Plant
April 3, 2006**

**ASSESSMENT
RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY
FROM THE 10 CFR PART 50 LICENSE**

15 pages

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

1.0 BACKGROUND:

On April 1, 2003, Consumers Energy submitted, for U.S. NRC approval, the Big Rock Point License Termination Plan (BRP LTP), pursuant to 10CFR50.82(a)(9). The initial BRP LTP incorporated survey methodology consistent with MARSSIM ("Multi- Agency Radiation Survey and Site Investigation Manual" or NUREG-1575), as well as appropriate regulatory guidance.

The BRP LTP was noticed in the Federal Register on March 21, 2005 (70 FR 13545) and a Safety Evaluation Report and Environmental Assessment of no Significant Environmental findings were issued on March 24, 2005.

Section 1.4.2 of the LTP defines the information that will be provided to support release of land from Big Rock's 10 CFR Part 50 license. This assessment serves as a review of the impacts on the following documents before releasing land areas from the license:

- Updated Final Hazards Summary Report and Technical Specifications,
- Environmental Monitoring Program,
- Offsite Dose Calculation Manual,
- Defueled Emergency Plan,
- Security Plan,
- Post Shutdown Decommissioning Activities Report,
- 10 CFR 100 Siting Criteria, and
- Environmental Report.

This assessment also includes a summary of the site history related to the release area, an evaluation of the potential for re-contamination from subsequent activities and controls applied and a Safety Analysis including a discussion of no significant hazards determination evaluation.

The Final Status Survey (FSS) Summary Report for the land subject to this release request will be submitted to the NRC as activities are completed.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

2.0 RELEASE AREA

Reference Consumers Power Company to the U.S. Atomic Energy Commission (AEC), dated October 14, 1960 (reference 8.1). The original license area is described as approximately 600 acres. The legal description in Attachment 2 of this letter was developed from Big Rock Point Plant Site Property Survey Drawing (Consumers Energy Drawing S-G16408), drawn from field surveys performed in September of 1959.

2.1. Radiological History

2.1.1. Non-Impacted Portions of the Release Area

Of the approximately 475 acres of land to be released, approximately 340 acres are classified as non-impacted according to MARSSIM criteria.

Historical data indicate that plant operations had no impact on this area based on the following:

1. Samples collected as part of the Radiological Environmental Monitoring Program (REMP) throughout the plant's operational and post-operational history showed no evidence of any radiological impact due to plant operations;
2. Radiological environmental monitoring and sampling was performed in the non-impacted "Release Area" in accordance with the REMP and Off-site Dose Calculation Manual (ODCM).
3. Radiological analyses were performed with gamma exposure measuring devices, on samples for radionuclide airborne particulates and for reactor-produced iodine, and on broad leaf vegetation.
4. The BRP Annual Radiological Environmental Operating Reports showed no long-lived radionuclides other than Sr-90 and Cs-137 above the Minimum Detection Level (MDL). Sr-90 and Cs-137 are measured at levels consistent with those found throughout the area and are attributed to past atmospheric nuclear weapons testing. The results are included in the BRP Annual Radiological Environmental Operating Reports.
5. A conservative evaluation of the potential impact of particulate effluents to soils outside of the industrial area using a Gaussian dispersion/deposition model substantiated the conclusion that this source of plant-derived radioactive material would be expected to contribute (at a maximum) a very

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

small fraction of the DCGL. Beyond the impacted area boundary (in the non-impacted areas), concentrations of this plant-derived radioactive material are shown to be undetectable and indistinguishable from background; and

6. A statistical comparison of soil sample analytical data from the non-impacted area and an environmentally equivalent reference area (unaffected by plant releases) was performed.

Possible reclassification or impact on these areas due to decommissioning activities is discussed in the BRP LTP. Administrative and engineering controls were utilized to prevent cross contamination. At the time of construction, the BRP ISFSI area was located within a site area classified as non-impacted by the operation of the former reactor/plant.

2.1.2 Impacted Areas as Identified in the License Termination Plan

Those areas identified in the License Termination Plan as impacted will meet the unrestricted release criteria using the methodology outlined in the LTP Chapter 5 prior to release.

2.2 Non-Radiological and Cultural History

The methodology outlined in LTP Chapter 8 for control of non-radiological impacts continues to apply.

On February 16, 2006, the U.S. NRC, the State of Michigan Historic Preservation Office, and BRP signed a Memorandum of Agreement (MOA). BRP, before transfer of ownership of any portion of land, will implement resource management plans and recommendations contained therein. MOA stipulations of recordation within one year of the date of the MOA (2/13/2006) and establishment of an exhibit shall be performed in accordance with the MOA.

3.0 ASSESSMENT - IMPACT OF THE RELEASE OF THE AREA

3.1 Impact of Releasing the Specific Area on Part 50 Licensing Basis

The license basis for the BRP includes the maintenance of certain programs to fulfill regulatory requirements and functional responsibilities. Throughout decommissioning, these programs are modified as necessary and terminated when the applicable concern is no longer relevant. These program changes are implemented using the change processes specified for each type of program.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

The phased release approach is described in Section 1.4.2 of the BRP LTP. The methodology for releasing land requires a review and assessment of the impact on license programs for the site lands remaining within the domain of the Part 50 License. The impact of releasing the "Release Area" from the Part 50 License on each of the licensing programs included in Section 1.4.2 of the LTP is described below.

3.1.1 Updated Final Hazards Summary Report (UFHSR) and Environmental Report

Minor change will be required to the Updated Final Safety Analysis Report (March 2005), Section 2, Figure 2.2, "Site Boundary and Exclusion Area" to show the reduced site area resulting from the removal of the "Release Area" from the Part 50 License and to identify the new site boundary on Figure 2.2. UFHSR Chapter 1 contains the 10 CFR 72.104 analysis results pursuant to 10 CFR 72.212. The ISFSI restricted area is encompassed by the ISFSI site (Parcel B in Attachment 2).

The revisions are consistent with LTP Chapter 2 groundwater discussions. The BRP Environmental Report and LTP Chapter 8, Update to the Environmental Report requires no significant revisions due to the release of the "Release Area."

Upon release of property, the 10 CFR 50.59 process will be used to appropriately revise the UFHSR, to meet the requirements of a stand-alone ISFSI site.

3.1.2 Technical Specifications

The BRP Technical Specifications are not impacted by the release of the "Release Area." On January 13, 2000, the U.S. NRC issued amendment 121 to the Big Rock Point Defueled Technical Specifications. This amendment removed the figure with the site boundary from the Defueled Technical Specifications. The remaining description of the site location in the Technical Specifications can be applied to the ISFSI after partial site release.

3.1.3 Off-site Dose Calculation Manual (ODCM)

The ODCM will be revised consistent with the alteration of the ISFSI site boundary on which the offsite dose calculations are based. The ODCM contains the Environmental Monitoring Program, which is discussed in section 3.1.8, below. Exclusion area boundary (EAB) is discussed in section 3.1.7 of this assessment.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

Upon release of property, the 10 CFR 50.59 process will be used to appropriately revise the environmental monitoring program, as contained in the ODCM, to meet the requirements of a stand-alone ISFSI site.

3.1.4 Independent Spent Fuel Storage Installation (ISFSI) Emergency Plan

No reduction in effectiveness of this plan will result due to the release of the "Release Area." The existing "Site Map" will be replaced with a Site Map of the ISFSI area in the ISFSI Emergency Plan. Upon partial site release, this will be an administrative revision.

The ISFSI Emergency Plan defines a 300-meter "ISFSI Owner-Controlled Area," consistent with the ISFSI Security Plan. Although the ISFSI restricted area is less than 300 meters, the BRP ISFSI site boundary is at or beyond 300 meters. ISFSI site boundary, on which the offsite dose calculations are based, depend on the EAB. EAB is discussed in section 3.1.7 of this assessment.

Upon release of property, the 10 CFR 50.59 process will be used to appropriately revise the ISFSI Emergency Plan, to meet the requirements of a stand-alone ISFSI site.

3.1.5 ISFSI Security Plan

This plan will not be affected by the release of the "Release Area." The 300-meter area is included in the BRP ISFSI site. This area will remain as part of the licensed BRP ISFSI site, after the first phase of partial site release.

The ISFSI restricted area for BRP is equivalent to the area within approximately 150-meter radius around the ISFSI, as shown on Attachment 2. The licensee will continue to own and control access to the property within 300 meters of the ISFSI until such time as the spent fuel and reactor-related Greater Than Class C (GTCC) radioactive waste is removed from the site and the 10 CFR Part 50 license is terminated.

3.1.6 Post Shutdown Decommissioning Activities Report (PSDAR)

Removing the "Release Area" from the terms of the 10 CFR Part 50 license will not impact the BRP PSDAR. LTP chapters 1, 3, and 7 have replaced the information in the PSDAR and are maintained consistent with the process for BRP UFHSR. Review requirements to ensure 10 CFR 50.82 commitments are met have been incorporated into the BRP screening process for revisions to procedures and plans.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

3.1.7 10 CFR Part 100 Siting Criteria

“EAB” will no longer be a meaningful term and its use will be discontinued and replaced with ISFSI site boundary.

The definition of the 10 CFR Part 50 exclusion area is based upon the existence of a reactor. Upon the submittal of the 10 CFR 50.82 (a) certifications to permanently shutdown the reactor and permanently remove fuel from the reactor, BRP no longer had a reactor. The UFHSR contained accident analyses where offsite dose consequences were calculated at the exclusion area boundary. The EAB has been maintained as a point of reference with the appropriate radiological criteria, e.g. EPA PAG's.

At Big Rock Point, the owner controlled area and the EAB were the original licensed site boundary.

By letter dated September 30, 1998, BRP was granted an exemption from certain requirements of 10 CFR 50.54(q), 10 CFR 50.47(b) and (c), and Appendix E to 10 CFR Part 50. This exemption from certain offsite emergency planning requirements was based, in part, upon the assertion that the calculated maximum offsite dose from postulated releases to an individual at the exclusion area boundary is less than the U.S. Environmental Protection Agency (EPA) Protective Action Guides (PAG's). The dose criterion in the EPA PAG's is 1 rem total effective dose equivalent and 5 rem to the thyroid, (Reference 8.3). Since this criterion is more restrictive than the Part 100 criteria, BRP has used the EPA PAG's as the standard for acceptable accident doses at the EAB.

Throughout decommissioning, there have been only a few EAB related UFHSR accident analyses that were applicable to BRP: Fuel Handling Accident, Low Level Waste Release Incidents and Spent Fuel Pool Accidents. As decommissioning has proceeded the requisite initial conditions for these accidents have progressively ceased to exist. All of the remaining fuel associated with the historical operation of the reactor has been transferred to NRC certified dry casks stored at the ISFSI. The accident analyses that will continue to be applicable to dry cask storage at the ISFSI are described and evaluated in the associated dry cask Safety Analysis Report (references 8.5 and 8.11).

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

The primary boundary of concern for the ISFSI design basis accident dose evaluation is the "restricted area" established pursuant to 10 CFR 72.104. The ISFSI restricted area is independent from the EAB. Remediation associated with decommissioning activities to meet unrestricted release criteria will remove any other radiological source term of significance from the non-ISFSI site land. The ISFSI area defined in Attachment 2 (Parcel B) encompasses the ISFSI restricted area.

Liability insurance will continue to be maintained as required by 10 CFR 50.54(w).

3.1.8 Environmental Monitoring

The current Environmental Monitoring Program will be revised as a result of the release of the "Release Area."

The BRP LTP demonstrates the criteria for unrestricted release of the "Release Area," which may have been impacted by plant operation. The ISFSI area is up gradient and a considerable distance from areas of former plant operation. The BRP ground water monitoring program in the ODCM is applicable to the former operating plant area. Once the former operating plant area FSS surveys are complete, and the criteria for unrestricted release is demonstrated, ground water monitoring will be complete. No additional groundwater monitoring is expected for ISFSI operations.

Upon release of property, the 10 CFR 50.59 process will be used to appropriately revise the environmental monitoring program, as contained in the ODCM, to meet the requirements of the BRP ISFSI site.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

3.2 Statement of Dismantlement Activities

No dismantlement activities, other than those documented in the LTP, are required in the "Release Area" for BRP to release this area. A large portion of the subject area is already deemed non-impacted. Final Status Surveys have been or will be performed in impacted areas in accordance with the BRP LTP.

3.3 Potential Combined Dose Effects

Since the release area will meet the unrestricted release criteria, as demonstrated by the methodology outlined in the BRP LTP, dose effects on remaining ISFSI operations and decommissioning are within the bounds of the approved LTP.

3.4 Potential for Cross-Contamination from Subsequent Activities

Final Status Surveys have been or will be performed in accordance with the LTP on impacted portions of the release areas to demonstrate they meet the unrestricted release criteria of the 10 CFR Part 20, in accordance with the approved BRP LTP.

The BRP LTP Section 5.2.4, Access Control Measures, describes isolation and control measures that are used to protect areas where FSS is being performed or has been completed.

As discussed in the prior section on environmental monitoring, there are no groundwater-monitoring wells that will be left within or near the "Release Area." The results of environmental monitoring to date indicate that there is no reasonable potential for contamination conveyed by groundwater to impact the "Release Area." ISFSI operations are passive and no effluents are created. Therefore, there is no future impact on groundwater from the operation of the ISFSI.

After partial site release, if new information is discovered for the release area, it will be evaluated pursuant to 10 CFR 20.1401(c),.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

4.0 SITE RELEASE CRITERIA AND FINAL STATUS SURVEY REPORT

The site release criteria for the BRP site corresponds to the 10 CFR 20.1402 criteria for unrestricted use. The residual radioactivity, including that from ground water sources, that is distinguishable from background, must not cause the total effective dose equivalent (TEDE) to an average member of the critical group to exceed 25 mrem/yr. The residual radioactivity must also be reduced to levels that are ALARA. A Final Status Survey Summary Report in accordance with the BRP LTP will be provided to the U.S. NRC to summarize the results of data collected for the release area and demonstrate compliance with this release criterion.

No deviations or special requests from the approved BRP LTP are anticipated before the submittal of the Final Status Survey Summary Report needed to facilitate this partial site release request.

BRP LTP section 5.7 identifies the contents of the written reports of final status survey results that are to be submitted to the NRC. These contents include the items described in NUREG 1757, Vol. 2, Section 4.5.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

5.0 SAFETY ANALYSIS

5.1 No Significant Hazards Consideration

Big Rock has evaluated whether or not a significant hazards consideration is involved with the proposed release by examining the three standards set forth in 10CFR 50.92, "Issuance of Amendments". As discussed below, Big Rock has concluded that the proposed release does not involve a significant hazards consideration.

1. Does the proposed release involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No

The evaluation involves release of land presently considered part of the BRP plant site under license DPR-6. The release of this land will occur after all demolition activities are completed and final status surveys have been performed to document the final radiological conditions of the land. When the release occurs, the only remaining radiological hazard at the site will be contained in the Independent Spent Fuel Storage Installation (ISFSI). Therefore, the focus of the analysis is on the potential impact on the probability and consequences of accidents associated with the ISFSI.

The accident conditions evaluated for the spent fuel storage casks include the following: accident pressurization, misloading of fuel canisters, drop of the vertical concrete casks, explosion, fires, maximum anticipated heat load, earthquakes, floods, lightning strikes, tornado and tornado driven missiles, tip over of vertical concrete cask, and full blockage of vertical concrete cask air inlets and outlets. The release of the non-ISFSI land from the license will not affect the probability of any of these accidents. The licensee will retain control over activities performed in the Owner Controlled Area to ensure that there is no impact on consequences from postulated accidents. Therefore, the proposed release of the land will not affect the consequences of any of these postulated accidents.

The proposed action, therefore, does not increase either the probability or the consequences of any accidents that have been considered.

ATTACHMENT 3

ASSESSMENT
RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY
FROM THE 10 CFR PART 50 LICENSE

2. Does the proposed release create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No

The activity involves release of land presently considered part of the BRP plant site under license DPR-6. When the release becomes effective, demolition activities will be complete and all systems, structures and components will have been removed from the land. The requested release of the land does not create the possibility of a new or different kind of accident that could affect the ISFSI that has not been considered in the design, installation or operation of the ISFSI. As noted above, the licensee will retain control over activities performed in the Owner Controlled Area for the ISFSI to assure that no new hazards are introduced that could create the potential for a new or different kind of accident. Therefore, the release does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the proposed release involve a significant reduction in the margin of safety?

Response: No

The margin of safety defined in the statements of consideration for the final rule on the radiological criteria for license termination is described as the margin between the 100 mrem/yr public dose limit established in 10 CFR 20.1301 for licensed operation and the 25 mrem/yr dose limit to the average member of the critical group at a site considered acceptable for unrestricted use. This margin of safety accounts for the potential effect of multiple sources of radiation exposure to the critical group.

The LTP prepared by BRP and updated in accordance with 10 CFR 50.59 on an as needed frequency or at a minimum frequency concurrent with the Updated Final Hazards Summary Report establishes conservative criteria for residual radiation levels following completion of demolition activities at the site. The LTP demonstrates that when these conservative criteria are met, the dose to the average member of the critical group will be below the regulatory dose limits established by the NRC. The proposed release of the site lands, once the criteria established in the LTP have been met will, therefore, not result in any reduction in the margin of safety.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

5.2 Applicable Regulatory Requirements / Criteria

The release of the remaining non-ISFSI site lands is part of the licensee's overall efforts to terminate license DPR-6 and achieve unrestricted release of the entire site in accordance with the criteria in Subpart E of 10CFR 20. 10 CFR 20.1402, "Radiological Criteria for Unrestricted Use," allows termination of a license and release of a site for unrestricted use if the residual radioactivity that is distinguishable from background radiation results in a total effective dose equivalent to an average member of a critical group that does not exceed 25 mrem/yr and the residual radioactivity has been reduced to levels that are ALARA. The LTP assures that these regulatory requirements will be met so that the land to be released from the license under this licensing action will meet these requirements.

10 CFR 50.83(b) establishes the criteria to be used by the NRC for approval of partial site release. These criteria include (1) dismantlement has been performed in accordance with the approved LTP and, (2) the final radiation survey and associated documentation demonstrate that the facility and site have met the criteria for decommissioning in 10CFR 20, Subpart E. Although no further dismantling activities or a FSS will be required for the "Release Area," the release of the "Release Area" supports the process of license termination by demonstrating that this portion of the site land can be released from BRP license.

The proposed release supports the process of license termination by demonstrating that an additional portion of the remaining site lands can be released from the Site license. This letter, along with future letters, provides documentation that demolition activities have been performed in accordance with the LTP and that the final status survey confirms the residual radioactivity in each survey unit meets the criteria established in the LTP. Thus, the requested amendment supports the overall license termination process in accordance with NRC regulations.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

6.0 ENVIRONMENTAL CONSIDERATIONS

This request meets the criteria specified in 10 CFR 51.22(c) (9) for a categorical exclusion from the requirements to perform an environmental assessment or to prepare an environmental impact statement. The specific criteria of 10 CFR 51.22(c)(9) are discussed below:

1. The partial site release involves no significant hazards consideration.

As demonstrated in section 5.1, partial site release of property from the license DPR-6 involves no significant hazards considerations.

2. There is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite.

The proposed partial site release involves release of land that has been demonstrated to meet the radiological criteria established in the LTP. There will be no effluents from this land. The only remaining effluents for the site will be the storm water runoff from the ISFSI. Thus, the proposed licensing action will not result in a change in type or increase in the amount of any effluents released offsite.

3. There is no increase in individual or cumulative occupational radiation worker exposure.

When partial site release of property from license DPR-6 is effective, all decommissioning and demolition activities on the site land other than the portion used for the ISFSI facility operation will have been completed. The residual radiation levels will have been confirmed to meet the criteria established in the LTP so that the dose to the critical group will meet the enhanced state cleanup standards requiring that the residual radioactivity distinguishable from background radiation result in a total effective dose equivalent to an average member of a critical group not more than 25 mrem/yr for all pathways. The only residual sources of exposure to radiation workers will be the ISFSI. Therefore, the proposed action does not result in any increase in exposure to an individual, or increased cumulative doses to radiation workers.

Based on the foregoing information, BRP concludes that the requested release of non-ISFSI site lands is acceptable and meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22 (c)(9). Thus no environmental impact statement or environmental assessment need be prepared in connection with the activity.

ATTACHMENT 3

ASSESSMENT RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY FROM THE 10 CFR PART 50 LICENSE

7.0 CONCLUSION

The partial site release of property from license DPR-6 is part of BRP's overall effort to terminate license DPR-6 and to achieve unrestricted release of the entire site in accordance with the criteria in Subpart E of 10 CFR 20. This action is also consistent with the approach described in Section 1.4.2 of the BRP LTP.

8.0 REFERENCES

- 8.1 Consumers Power Company to the U.S. AEC, *Application to the U.S Atomic Energy Commission for Reactor Construction Permit and Operating License*, October 14, 1960
- 8.2 Consumers Power Company to the U.S. NRC, *Docket 50-155 – License DPR-6 – Big Rock Point Plant – Certification of Permanent Fuel Removal*, September 23, 1997
- 8.3 U.S. NRC to Consumers Power Company, *Exemption from Certain Requirements of 10 CFR 50.54(q) Regarding Offsite Emergency Planning Activities and Approved Defueled Emergency Plan*, September 30, 1998
- 8.4 U.S. NRC to Consumers Power Company, *NRC Issuance of Amendment 121 to the Defueled Technical Specifications – Removal of Site Boundary*, January 13, 2000
- 8.5 To BNFL from the U.S. NRC, *Certificate of Compliance for Spent Fuel Storage Casks – Docket 72-1026, Issued to BNFL FuelSolutions*, February 15, 2001
- 8.6 Consumers Power Company to the U.S. NRC, *Docket 50-155 – License DPR-6 – Big Rock Point Plant – Historical Site Assessment (HSA) in Support of Big Rock Point License Termination Plan Submitted July 1, 2004*, July 14, 2004
- 8.7 U.S. NRC to Consumers Power Company- *Big Rock Point – Issuance of Amendment 126 to Approve the License Termination Plan (TAC No. L52096)*, March 24, 2005
- 8.8 Consumers Power Company to the U.S. NRC, *Docket 50-155 – License DPR-6 – Big Rock Point Plant – Revision 2 to the Big Rock Point License Termination Plan*, September 27, 2005

ATTACHMENT 3

ASSESSMENT
RELEASE OF PORTION OF THE BIG ROCK POINT SITE PROPERTY
FROM THE 10 CFR PART 50 LICENSE

- 8.9 Consumers Power Company to the U.S. NRC, *Docket 50-155 – License DPR-6 – Big Rock Point Plant – Memorandum of Agreement between the U.S. Nuclear Regulatory Commission and the Michigan State Historic Preservation Office Regarding the Decommissioning of the Big Rock Point Nuclear Facility*, February 16, 2006
- 8.10 Big Rock Point Volume 33, *BRP212, 10CFR72.212 and Certificate of Compliance Evaluation Report*
- 8.11 BNG / FuelSolutions™ Storage System Final Safety Analysis Reports (FSARs), *WSNF-220, FuelSolutions™ FSAR, Revision 2*, and *WSNF-220 FuelSolutions™ W74 Canister Storage FSAR, Revision 4*.
- 8.12 U.S. Environmental Protection Agency, *EPA 400-R-92-001, Manual of Protective Action Guides and Protective Actions for Nuclear Incidents*, May 1992
- 8.13 U.S. Nuclear Regulatory Commission NUREG-1575, *Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM)*, August 2000