

April 14, 2006

Mr. L. M. Stinson
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
P.O. Box 1295
Birmingham, AL 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 — ISSUANCE OF
AMENDMENTS RE: CONTAINMENT TENDON SURVEILLANCE PROGRAM
(TAC NOS. MC7205 AND MC7206)

Dear Mr. Stinson:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 172 to Renewed Facility Operating License No. NPF-2 and Amendment No. 165 to Renewed Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant, Units 1 and 2. The amendments consists of changes to the Technical Specifications (TS) in response to your application dated June 1, 2005, as supplemented on February 13, 2006.

The amendments revise TS Section 5.5.6, "Pre-Stressed Concrete Containment Tendon Surveillance Program," for consistency with the requirements of 10 CFR 50.55a(g)(4) for components classified as Code Class CC. The amendment also deletes the provisions of Surveillance Requirement 3.0.2 from this TS. In addition, the amendment deletes the reporting requirements in TS 5.6.9, "Tendon Surveillance Report."

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/
Robert E. Martin, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosures:

1. Amendment No. 172 to NPF-2
2. Amendment No. 165 to NPF-8
3. Safety Evaluation

cc w/encls: See next page

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The amendments revise TS Section 5.5.6, "Pre-Stressed Concrete Containment Tendon Surveillance Program," for consistency with the requirements of 10 CFR 50.55a(g)(4) for components classified as Code Class CC. The amendment also deletes the provisions of Surveillance Requirement 3.0.2 from this TS. In addition, the amendment deletes the reporting requirements in TS 5.6.9, "Tendon Surveillance Report."

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cc w/encls: See next page

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Package Number: ML060830380

Tech Spec Number: ML061150446

Amendment Number: ML060830368

*SE memo NRR-058

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SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 RE: ISSUANCE OF AMENDMENTS RE: CONTAINMENT TENDON SURVEILLANCE PROGRAM (TAC NOS. MC7205 AND MC7206)

Date: April 14, 2006

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SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 172
Renewed License No. NPF-2

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated June 1, 2005, as supplemented on February 13, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 172, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Evangelos C. Marinos, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 2006

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 165
Renewed License No. NPF-8

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated June 1, 2005, as supplemented on February 13, 2006, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 165, are hereby incorporated in the license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented shall be implemented within 90 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Evangelos C. Marinos, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 14, 2006

ATTACHMENT TO LICENSE AMENDMENT NOS. 172 and 165
TO RENEWED FACILITY OPERATING LICENSE NO. NPF-8
TO RENEWED FACILITY OPERATING LICENSE NO. NPF-2
DOCKET NOS. 50-348 AND 50-364

Replace the following pages of the Appendix A Technical Specifications and associated Bases with the attached revised pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

5.5-4
5.5-5
5.5-6
5.6-5
5.6-6
B 3.6.1-5

Insert

5.5-4
5.5-5
5.5-6
5.6-5
5.6-6
B 3.6.1-5

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 172 TO

RENEWED FACILITY OPERATING LICENSE NO. NPF-2

AND AMENDMENT NO. 165 TO

RENEWED FACILITY OPERATING LICENSE NO. NPF-8

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

1.0 INTRODUCTION

By letter dated June 1, 2005, as supplemented on February 13, 2006, the Southern Nuclear Operating Company, Inc. (the licensee), submitted a request for changes to the Joseph M. Farley Nuclear Plant, Units 1 and 2 (FNP), Technical Specifications (TS). The requested changes would revise TS 5.5, "Programs and Manuals," and section 5.6, "Reporting Requirements." By this amendment, the requirements of Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, Section 50.55a(b)(2)(vi), will be incorporated in the FNP Containment Tendon Surveillance Program. The proposed changes are a result of the Nuclear Regulatory Commission (NRC) amending its regulations to incorporate, by reference, the 1992 Edition with the 1992 Addenda of Subsection IWE and Subsection IWL, of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) (Ref. 5.2, 5.3), to assure that critical areas of containments are routinely inspected to detect, and take corrective action for, defects that could compromise a containment's structural integrity. This evaluation addresses the appropriateness of the licensee's proposed amendments to FNP's TS. The licensee has also proposed to delete the provisions of Surveillance Requirement (SR) 3.0.2 from this TS. The licensee also provided, in its application, the corresponding changes to the TS Bases for SR 3.6.1, "Containment." The February 13, 2006, letter provided clarifying information that did not change the June 1, 2005, application and the initial proposed no significant hazards consideration determination.

2.0 REGULATORY REQUIREMENTS

Section 50.55a(b)(2)(vi) of 10 CFR requires that licensees use either the 1992 Edition with the 1992 Addenda, or the 1995 Edition with the 1996 Addenda of Subsection IWE and Subsection IWL of the ASME Code, as modified and supplemented by the requirements in paragraphs (b)(2)(viii) and (b)(2)(ix) of 10 CFR 50.55a when implementing the initial 120-month inspection interval for the containment inservice inspection (ISI) requirements of 10 CFR 50.55a.

The licensee proposes to update the FNP TS to reflect the above regulatory requirement for the inspection of the concrete containments. TS section 5.5.6, " Pre-stressed Concrete Containment Tendon Surveillance Program," will be revised to state: "The Tendon Surveillance Program, inspection frequencies, and acceptance criteria shall be in accordance with Section XI, Subsection IWL of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50.55a, except where an alternative, exemption or relief has been authorized by the NRC."

The licensee, in addition, proposes to include a prior relief request commitment (Ref. 5.4) as part of the proposed amendments. "The performance of the IWL requirements for containment sample tendon force measurements and tendon wire and strand sample examinations will be performed by the end of 2006."

3.0 TECHNICAL EVALUATION

3.1 Proposed Changes to Technical Specifications

In its application, the licensee proposed the following changes to TS 5.5.6 and 5.5.16:

1. TS 5.5.6 – at the end of the first paragraph after the phrase "in accordance with," the words "Regulatory Guide 1.35, Revision 2, 1976," would be replaced by the phrase "Section XI, Subsection IWL of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50.55a, except where an alternative, exemption or relief has been authorized by the NRC. The first performance of the IWL requirements for containment sample tendon force measurements and tendon wire and strand sample examinations will be performed by the end of 2006."
2. TS 5.5.6 – SR 3.0.2 would be deleted from the second paragraph. If SR 3.0.2 is applicable, then the interval, as specified in the TS, could be extended up to 1.25 times that interval.
3. TS 5.6.9 – The Tendon Surveillance Report required by TS 5.6.9 is deleted in lieu of the reporting requirements that are included in Subsection IWL.

The proposed amendment modifies the requirements on containment tendon surveillance. It does not change the design requirements on the containment.

3.2 Evaluation

The requirements of 10 CFR 50.55a were amended (61 FR 41303) to incorporate, by reference, Subsections IWE and IWL of Section XI of the ASME Code, for the inspection of containments of light water-cooled reactors. Subsection IWE provides the requirements for inservice inspection, repair, and replacement of Class MC pressure retaining components, and metallic shell and penetration liners of Class CC pressure retaining components, and their integral attachments. Subsection IWL provides the requirements for preservice examination, ISI, and repair of the reinforced containments.

Currently, TS 5.5.6 states in part:

This program provides controls for monitoring any tendon degradation in pre-stressed concrete containments, including effectiveness of its corrosion protection medium, to ensure containment structural integrity. The program shall include baseline measurements prior to initial operations. The Tendon Surveillance Program, inspection frequencies, and acceptance criteria shall be in accordance with Regulatory Guide 1.35, Revision 2, 1976.

The regulation covering the ISI of the containment tendons is 10 CFR 50.55a(g)(4). This requires that throughout the service of a boiling or pressurized water-cooled nuclear power facility, components which are classified as ASME Code Class CC, must meet the requirements, except design and access provisions and preservice examination requirements, set forth in Section XI of editions of the ASME Code and Addenda that are incorporated by reference in paragraph (b) of 10 CFR 50.55a, subject to the limitations and modifications listed therein.

The licensee has proposed to replace the reference to Regulatory Guide 1.35 by a reference to the regulations and to the specific subsection of Section XI of the ASME Code for containment tendons (Subsection IWL). Because the licensee has proposed the specific regulatory requirements for the containment tendon surveillance program, the NRC staff concludes that the proposed change is acceptable.

The licensee also proposes to delete the reference in TS 5.5.6 to surveillance requirement SR 3.0.2, as the inspection frequency is now prescribed by the referenced Code and regulated by 10 CFR 50.55a. Additionally, since the tendon inspection frequencies will be in accordance with ASME Code, Section XI, Subsection IWL, the provisions of SR 3.0.2 are no longer necessary and are deleted from TS 5.5.6. As such, 10 CFR 50.55a requires the implementation of ASME Code, Section XI, Subsection IWL, and specifies the requirements for extending inspection frequencies. Based on this, the NRC staff concludes that deletion of SR 3.0.2 from TS 5.5.6 is acceptable. In a prior relief request (Ref. 5.4) granted by the NRC, the NRC staff has discussed the inspection frequency, and approved a modified inspection schedule based on a "common administrative date" to facilitate inspection.

The licensee also proposes to delete TS section 5.6.9, which addresses the reporting of abnormal degradation of containment conditions to NRC. This is no longer required, as all reportable events are addressed under Subsections IWA and IWL of the ASME Code, as regulated by 10 CFR 50.55a, and it is not necessary to repeat these Federal regulations in the TS to ensure safe plant operation. Also, since the reporting requirements will be in accordance with 10 CFR 50.55a(b)(2)(viii), the provisions of TS 5.6.9 are no longer necessary and are deleted from the TS. On this basis, the NRC staff concludes that deletion of TS 5.6.9 is acceptable.

The licensee has also amended the last sentence of the bases section to SR 3.6.1.2 to read as follows: "Testing and Frequency are consistent with the requirements of Section XI, Subsection IWL of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50.55a, except where an alternative, exemption or relief has been authorized by the NRC." This revision to the bases is consistent with the proposed revisions to

the TS that are being implemented to bring FPN's Containment Tendon Surveillance Program into compliance with 10 CFR 50.55a requirements.

3.3 Conclusions

Based on the above findings and the consistency of the proposed amendment to Section XI of the ASME Code, the NRC staff concludes that the proposed amendment is acceptable. The NRC staff also reviewed the licensee's identified changes to the TS Bases that were included in the application and finds that they are consistent with the proposed changes to the TS.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of Alabama official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change the surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (70 FR 35739, June 21, 2005). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

5.0 REFERENCES

- 5.1 Southern Nuclear Operating Company, "Joseph M. Farley Nuclear Plant - Request for Technical Specification Amendment, Containment Surveillance Program," Dockets 50 - 348/364, June 1, 2005.
- 5.2 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, 1992 Edition, Division 1, Section XI, Subsection IWE, "Requirement for Class MC and Metallic Liners of Class CC Components of Light-Water Cooled Plants," 1992 addenda, New York.

- 5.3 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, 1992 Edition, Division 1, Section XI, Subsection IWL, "Requirement for Class CC Concrete Components of Light-Water Cooled Plants," 1992 addenda, New York.
- 5.4 Letter, NRC to Southern Nuclear Operating Company, "Joseph M. Farley Nuclear Plant, Units 1 and 2 - Request for Relief No. RR-57 and RR-58 Regarding Containment Tendon Inspections (TAC Nos MC7209 and MC7210), dated March 28, 2006.

Principal Contributor: S. Samaddar

Date: April 14, 2006

Joseph M. Farley Nuclear Plant, Units 1 & 2

cc:

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