

March 16, 2006

Mr. Michael R. Kansler
President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 - REQUEST FOR
ADDITIONAL INFORMATION REGARDING PROPOSED STEAM GENERATOR
EXAMINATION PROGRAM - 2006 REFUELING OUTAGE (2R17)
(TAC NO. MD0213)

Dear Mr. Kansler:

On February 13, 2006, Entergy Nuclear Operations, Inc. (Entergy), submitted the proposed steam generator examination program for 2006 refueling outage (2R17) in accordance with the requirements of Indian Point Nuclear Generating Unit No. 2 Technical Specifications 5.5.7.f.1.

The Nuclear Regulatory Commission staff is reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). During a telephone call on March 14, 2006, the Entergy staff indicated that a response to the RAI would be provided within 30 days from the date of this letter.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,

/RA/

John P. Boska, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosure:
RAI

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING PROPOSED STEAM GENERATOR INSPECTION PROGRAM
2006 REFUELING OUTAGE (2R17)
ENTERGY NUCLEAR OPERATIONS, INC.
INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247

In a letter dated February 13, 2006, Agencywide Documents Access and Management System Accession No. ML060530378, Entergy Nuclear Operations, Inc. (Entergy) submitted its proposed steam generator examination program for the spring 2006 refueling outage for Indian Point Nuclear Generating Unit No. 2 in accordance with Technical Specification 5.5.7.f.1. The Nuclear Regulatory Commission (NRC) staff is reviewing the submittal and has the following questions:

1. A couple of plants with thermally treated Alloy 600 steam generator tubes have identified crack-like indications in the portion of the tube within the tubesheet region (refer to Information Notice 2005-09, "Indications in Thermally Treated Alloy 600 Steam Generator Tubes and Tube-to-Tubesheet Welds)." These crack-like indications occurred prior to observing cracking at any other location (e.g., expansion transition, U-bend). Please discuss your basis for not performing any rotating probe inspections near the tube-to-tubesheet weld, tack expansion, or overexpansions (bulges or tubesheet anomalies).
2. The NRC staff notes that as part of your proposed Steam Generator Examination Program for your 2006 refueling outage (2R17), no plans were identified to inspect dings or dents with a rotating probe. Depending on their size, dings and dents may not be readily inspected with a bobbin probe. In addition, circumferential cracking that could occur at dents and dings will generally not be detectable with a bobbin probe. Dings and dents can increase the stress levels in the tube making them an area more susceptible to cracking than other portions of the tube. As a result, please provide your basis for not inspecting dings or dents with a rotating probe.

Enclosure