



UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 LISLE, ILLINOIS 60532

March 13, 2006

MEMORANDUM TO: Bruce L. Burgess, Chief
 Reactor Projects Branch 2
 Division of Reactor Projects

FROM: David E. Hills, Chief /RA by Ann Marie Stone Acting For/
 Engineering Branch 1
 Division of Reactor Safety

SUBJECT: LASALLE NUCLEAR POWER STATION
 DRS INPUT TO INTEGRATED REPORT
 50-373/2006-003; 50-374/2006-003

Attached is the report input for the LaSalle County Station, Units 1 and 2, Inspection Report No. 05000373/2006003; 5000374/2006003. This report input documents completion of Inspection Procedure (IP) 71111.08 Inservice Inspection (ISI) Activities. I have reviewed this input and have determined it is ready for distribution to the licensee and dissemination to the public.

POST INSPECTION DATA INPUT TO INSPECTION REPORT 50-373/2006-003; 50-374/2006-003:

Procedure Status and Sample Size:

IP 71111.08
 Status - Incomplete
 Sample size = 1

CONTACT: Don Jones, DRP
 (630) 829-9622

DOCUMENT NAME: C:\FileNet\ML060730573.wpd

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OFFICE	RIII		RIII				
NAME	DJones: ls		AMStone for DHills				
DATE	03/09/06		03/13/06				

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Cover Letter

Title Page

Inspector: D. Jones, Reactor Inspector

SUMMARY OF FINDINGS

ADAMS boilerplate - Inspectable areas: Routine Baseline Inspection Activities,

Modify second paragraph as follows:

The inspection was conducted by an inspector based in the NRC Region III office.

A. Inspector-Identified and Self-Revealing Findings

None.

REPORT DETAILS

1. REACTOR SAFETY

Cornerstone: Mitigating Systems

1R08 Inservice Inspection (ISI) Activities (71111.08)

.1 Piping Systems ISI

a. Inspection Scope

From February 27, 2006 to March 2, 2006, the inspectors conducted a review of the implementation of the licensee's ISI program for monitoring degradation of the reactor coolant system (RCS) boundary, and the risk significant piping system boundaries during the Unit 1 outage (L1R11). The inspectors selected the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Section XI required examinations and Code components in order of risk priority as identified in Section 71111.08-02 of IP 71111.08, "Inservice Inspection Activities," based upon the ISI activities available for review during the on-site inspection period.

The inspectors conducted an on-site review of the following types of nondestructive examination activities to evaluate compliance with the ASME Code Section XI, and Section V requirements, and to verify that indications and defects, (if present) were dispositioned in accordance with the ASME Code Section XI requirements. Specifically, the inspectors observed/reviewed the following examinations:

- Ultrasonic examination (UT) of a pipe-to-valve weld (weld 1-RH-1003-2), residual heat removal;

- Magnetic Particle examination (MT) of the reactor pressure vessel top head-to-flange weld (GEL-1009-AG) from 180 to 360 degrees azimuth; and
- an automated phased array UT of reactor core shroud weld H6.

The inspectors reviewed an examination with recordable indications that was accepted for continued service to verify that the licensee's acceptance was in accordance with the ASME Code or an NRC approved alternative. Specifically, the inspectors reviewed the following record:

- The inspectors reviewed L1R10 UT record (Data Report Number 1R10-70) of a feedwater (FW) valve-to-pipe weld (1FW-1001-21) performed on February 13, 2004. A recordable indication was dispositioned as being outside the required examination volume, however, it was evaluated and found to be acceptable per ASME Section XI, IWB-3514.

There were no pressure boundary welds, for Class 1 or 2 systems, completed by the licensee, and hence the inspectors did not perform the step of the inspection procedure that verifies that the welding process, and welding examinations were performed in accordance with ASME Code requirements, or an NRC approved alternative.

The inspectors performed a review of ISI related problems that were identified by the licensee, and entered into the corrective action program. Additionally, the inspectors' review included confirmation that the licensee had an appropriate threshold for identifying issues, and had implemented effective corrective actions. The inspectors evaluated the threshold for identifying issues through interviews with licensee staff, and review of licensee actions to incorporate lessons learned from industry issues related to the ISI program. The inspectors performed these reviews to ensure compliance with 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," requirements. The corrective action documents reviewed by the inspectors are listed in the attachment to this report. In addition, the inspectors verified that the licensee correctly assessed operating experience for applicability to the Inservice Inspection group.

The reviews as discussed above counted as one inspection sample.

b. Findings

No findings of significance were identified.

4OA6 Meetings

.2 Interim Exit Meetings

Interim exits were conducted for:

- Inservice inspection (IP 71111.08) with Ms. S. Landahl on February 2, 2006.

KEY POINTS OF CONTACT

Licensee

S. Landahl, Site Vice President
D. Enright, Plant Manager
P. Holland, Regulatory Assurance - NRC Coordinator
A. Kochis, ISI Coordinator
H. Madronero, Engineering Programs Manager
T. Simpkin, Regulatory Assurance Manager

Nuclear Regulatory Commission

B. Burgess, Chief, Reactor Projects Branch 2

LIST OF DOCUMENTS REVIEWED

1R08 Inservice Inspection Activities (IP 71111.08)

AR00458950; Jet Pump 12 Main Wedge Wear; No Change; dated February 26, 2006

AR00458993; Jet Pump 13 Main Wedge Wear; No Change; dated February 26, 2006

IR 457474; Stud nut on the north rod from can to the clamp was loose; dated February 24, 2006

GE ROP-002; Reactor and Field Services Operating Experience Program; dated January 2005

GE ROP-004; Briefings and Shift Turnover; dated January 2005

GE-PDI-UT-1; PDI Generic Procedure for the Ultrasonic Examination of Ferritic Pipe Welds; dated July 2005

GE-MT-100; Procedure for Magnetic Particle Examination (Dry Particle, Color Contrast or Wet Particle, Fluorescent); dated September 2004

LIST OF ACRONYMS USED

ASME	American Society of Mechanical Engineers
CFR	Code of Federal Regulations
DRS	Division of Reactor Safety
FW	Feedwater
IP	Inspection Procedure
ISI	Inservice Inspection
MT	Magnetic Particle Examination
NDE	Nondestructive Examination
RCS	Reactor Coolant System
RHR	Residual Heat Removal
UT	Ultrasonic Examination