

March 10, 2006

Bill Eaton, BWRVIP Chairman
Entergy Operations, Inc.
Echelon One
1340 Echelon Parkway
Jackson, MS 39213-8202

SUBJECT: NRC APPROVAL LETTER FOR BWRVIP-58-A, "BWR VESSEL AND
INTERNALS PROJECT, CRD INTERNAL ACCESS WELD REPAIR"

Dear Mr. Eaton:

By letter dated October 31, 2005, the Boiling Water Reactor Vessel and Internals Project (BWRVIP) submitted Proprietary Report BWRVIP-58-A, "BWR Vessel and Internals Project, CRD Internal Access Weld Repair," for Nuclear Regulatory Commission (NRC) staff review. The BWRVIP-58-A report provides the technical basis and application of an internal access weld repair for leaking control rod drive (CRD) housing penetrations in BWR/2 through BWR/5 reactors. The repair weld was designed to replace the load carrying capability of the CRD housing-to-stub tube J-weld, in addition to providing a seal to prevent leakage from the reactor pressure vessel (RPV). The BWRVIP provided the BWRVIP-58-A report to support generic regulatory efforts related to the weld repair of BWR CRD housing penetrations.

The BWRVIP-58-A report presents a compilation of information from the BWRVIP-58 report and the NRC staff final safety evaluation (SE) dated July 12, 2004, which includes the BWRVIP's associated responses to NRC staff requests for additional information (RAIs) and open items.

The NRC staff has reviewed the information in the BWRVIP-58-A report and has found that the report accurately incorporates all of the relevant information which was submitted by the BWRVIP in the documents noted above to support NRC staff approval of the report. The staff found that minimal revisions were made to the BWRVIP-58 report in the production of the BWRVIP-58-A report. These revisions are discussed in detail below.

The first revision was that the BWRVIP added text to Section 4.2.1 of the BWRVIP-58 report to address an open item that was identified in the staff's initial SE, dated October 17, 2001, on the BWRVIP-58 report. With respect to this open item, the staff requested that the BWRVIP update the references to the American Society of Mechanical Engineers (ASME) Code Case N-606 to ASME Code Case N-606-1. The staff determined that the BWRVIP adequately revised Section 4.2.1 of the BWRVIP-58 report to include a discussion of ASME Code Case N-606-1 and revised all references to ASME Code Case N-606 to the updated version, ASME Code Case N-606-1.

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The second revision was that the BWRVIP revised the text in Section 2.2 of the BWRVIP-58 report to address the staff's recommendation to clarify that crack growth from fatigue was evaluated for life extension. The staff determined that the BWRVIP adequately revised Section 2.2 of the BWRVIP-58 report to indicate that crack growth from fatigue was evaluated for life extension.

Based on the discussion above, the staff has determined that the BWRVIP-58-A report is acceptable. Please contact Meena Khanna of my staff at (301) 415-2150 if you have any further questions regarding this subject.

Sincerely,

/RA/

William H. Bateman, Deputy Director
Division of Component Integrity
Office of Nuclear Reactor Regulation

cc: BWRVIP Service List

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Based on the discussion above, the staff has determined that the BWRVIP-58-A report is acceptable. Please contact Meena Khanna of my staff at (301) 415-2150 if you have any further questions regarding this subject.

Sincerely,

/RA/

William H. Bateman, Deputy Director
Division of Component Integrity
Office of Nuclear Reactor Regulation

cc: BWRVIP Service List

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