

**James H. Lash**  
Site Vice President

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February 22, 2006  
L-06-025

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555-0001

**Subject: Beaver Valley Power Station, Unit No. 1  
BV-1 Docket No. 50-334, License No. DPR-66  
Supplemental Information Pertaining to License Amendment Request  
No. 302 (TAC No. MC4645)**

On February 9, 2006 the NRC issued License Amendment 273 for Beaver Valley Power Station (BVPS) Unit No. 1. The License Amendment approved the Technical Specification changes proposed in FirstEnergy Nuclear Operating Company (FENOC) License Amendment Request (LAR) 320, Replacement Steam Generators, submitted by FENOC letter L-05-069 and supplemented by FENOC letters L-05-168 and L-05-163.

The Technical Specification changes approved in License Amendment 273 were originally submitted as part of FENOC LAR 302, Extended Power Uprate, submitted by FENOC letter L-04-125. This letter withdraws these specific Technical Specification changes from LAR 302. Attachment A provides a complete listing of the Unit No. 1 Technical Specification changes proposed in LAR 302 and identifies those withdrawn.

This letter also documents that the remaining Unit No. 1 Technical Specification changes proposed in LAR 302 will be implemented during power operations. Letter L-04-125 states that an outage is needed to implement the Unit No. 1 changes proposed in LAR 302. However, FENOC has determined that all the remaining Unit No. 1 changes can be implemented during power operations. In all cases the existing Technical Specification requirements are either more conservative or restrictive than the changes proposed in LAR 302. As stated in FENOC letter L-05-168, the Unit No. 1 EPU amendment shall be implemented within 120 days following issuance of the amendment.

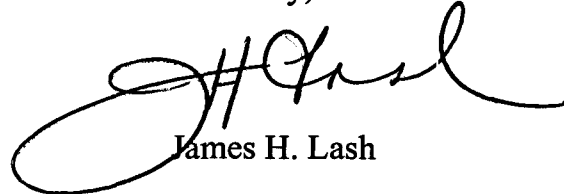
No new regulatory commitments are contained in this submittal. If there are any questions or if additional information is required, please contact Mr. Gregory A. Dunn, Manager, FENOC Fleet Licensing, at (330) 315-7243.

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I declare under penalty of perjury that the foregoing is true and correct. Executed on  
February 22, 2006.

Sincerely,

A handwritten signature in black ink, appearing to read "J. H. Lash", with a large, sweeping flourish at the end.

James H. Lash

Attachment:

- A. Status of LAR 302 Unit No.1 changes
  
- c: Mr. T. G. Colburn, NRR Senior Project Manager  
Mr. P. C. Cataldo, NRC Senior Resident Inspector  
Mr. S. J. Collins, NRC Region I Administrator  
Mr. D. A. Allard, Director BRP/DEP  
Mr. L. E. Ryan (BRP/DEP)

## ATTACHMENT A of L-06-025

### Status of LAR 302 Unit No.1 changes

The following table lists the changes originally proposed in License Amendment Request (LAR) 302, Extended Power Uprate (EPU). Those with a status of "WITHDRAWN" have been approved by Unit No. 1 License Amendment 273 and are thus being withdrawn from LAR 302. Those with a status of "REMAINING" are requested to be approved as part of the EPU License Amendment for Unit No. 1.

No.	Unit No. 1	Status	Title
1	License page 3	REMAINING	Item 2.C(1) Maximum Power Level
2	License page 3	REMAINING	Item 2.C(2) Technical Specifications
3	1.0	REMAINING	DEFINITIONS – 1.3 RATED THERMAL POWER
4	2.1.1.1	WITHDRAWN	SAFETY LIMITS – REACTOR CORE
5	3.1.2.8	REMAINING	REFUELING WATER STORAGE TANK (RWST)
6	3.3.1.1	REMAINING	REACTOR TRIP SYSTEM INSTRUMENTATION (Tables 3.3-1 and 4.3-1, FUNCTIONAL UNIT 4, Power Range, Neutron Flux High Negative Rate Trip)
7	3.3.1.1	WITHDRAWN	REACTOR TRIP SYSTEM INSTRUMENTATION (Table 3.3-1, FUNCTIONAL UNIT 14, Steam Generator Water Level Low-Low)
8	3.3.1.1	WITHDRAWN	REACTOR TRIP SYSTEM INSTRUMENTATION (Table Notation, Overtemperature/Overpower $\Delta T$ )
9	3.3.1.1	REMAINING	REACTOR TRIP SYSTEM INSTRUMENTATION (Table Notation, Action 8)
10	3.3.2.1	REMAINING	ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION (Table 3.3-3, Footnote to Steamline Pressure – Low)
11	3.3.2.1	WITHDRAWN	ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION (Table 3.3-3, FUNCTIONAL UNIT 5.a, Steam Generator Water Level High-High, and 7.a, Steam Generator Water Level Low-Low)
12	3.4.1.3	WITHDRAWN	REACTOR COOLANT SYSTEM – SHUTDOWN (SR 4.4.1.3.3)
13	3.4.3	REMAINING	REACTOR COOLANT SYSTEM – SAFETY VALVES
14	3.4.5	WITHDRAWN	STEAM GENERATORS
15	3.4.8	REMAINING	REACTOR COOLANT SYSTEM – SPECIFIC ACTIVITY
16	3.5.1	WITHDRAWN	ACCUMULATORS
17	3.5.4.1.1	REMAINING	BORON INJECTION TANK $\geq 350^{\circ}\text{F}$

No.	Unit No. 1	Status	Title
18	3.5.4.1.2	REMAINING	BORON INJECTION TANK < 350°F
	3.5.2	REMAINING	ECCS SUBSYSTEMS - $T_{avg} \geq 350^\circ\text{F}$
	3.5.3	REMAINING	ECCS SUBSYSTEMS - $T_{avg} < 350^\circ\text{F}$
19	3.5.5	WITHDRAWN	SEAL INJECTION FLOW
20	3.7.1.1	REMAINING	TURBINE CYCLE - MAIN STEAM SAFETY VALVES (MSSVs)
21	3.7.1.3	REMAINING	PRIMARY PLANT DEMINERALIZED WATER (PPDW)
22	3.7.1.4	REMAINING	PLANT SYSTEMS - ACTIVITY
23	6.9.5	WITHDRAWN	CORE OPERATING LIMITS REPORT (COLR)