



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 7, 1994

Mr. John J. Barton
Vice President and Director
GPU Nuclear Corporation
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, NJ 08731

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION INDIVIDUAL PLANT
EXAMINATION (IPE) SUBMITTAL (TAC NO. M74443)

Dear Mr. Barton:

In a letter dated August 2, 1994, the staff transmitted its Staff Evaluation regarding the Oyster Creek IPE for internal events and internal floods. On page 18 of the Staff Evaluation it inadvertently stated that the licensee planned actions regarding:

- Increased training on the importance of the core spray system.
- Changes to maintenance scheduling for the core spray system to improve downtime.
- Programs instituted to reduce blockage and fouling of the isolation condensers.

In a letter dated November 8, 1994, GPU Nuclear Corporation stated that the actions and commitments initially identified in Section 8.1.3 of the Oyster Creek Nuclear Generating Station (OCNGS) IPE Submittal Report pertained to the containment spray system and associated heat exchangers and are not related to the core spray system or the isolation condensers as stated in the NRC Staff Evaluation.

Therefore, the staff has revised page 18 of the Staff Evaluation in accordance with the actions described in section 8.1.3 of the OCNGS IPE Submittal Report.

Sincerely,

A handwritten signature in cursive script, reading "Alexander W. Dromerick, Sr.".

Alexander W. Dromerick, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure: Revised Page 18 of Staff
Evaluation Dated August 2, 1994

cc w/encl: See next page

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Original signed by:

Alexander W. Dromerick, Sr. Project Manager
Project Directorate 1-4
Division of Reactor Projects - 1/11
Office of Nuclear Reactor Regulation

Enclosure: Revised Page 18 of Staff
Evaluation Dated August 2, 1994

cc w/encl: See next page

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Mr. John J. Barton
Vice President and Director

Oyster Creek Nuclear
Generating Station

cc:

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7. Generic Safety Issues

As part of the IPE submittal, the licensee proposed resolution of several generic issues including USI A-17, "System Interaction in Nuclear Power Plants;" USI A-47, "Safety Implications of Control Systems;" Generic Issue (GI)-101, "BWR Water Level Redundancy;" and GI-105, "Interfacing System LOCA at BWRs." However, USI A-17, GI-101, and GI-105 were resolved by staff with no new requirements. Accordingly, the licensee's proposed resolution of these issues was not reviewed in detail. The review of the licensee's response to Generic Letter 89-19, "Request for Action Related to Resolution of Unresolved Safety Issue A-47," addresses USI A-47 resolution.

8. Licensee Actions and Commitments From the IPE

The licensee used the IPE process to identify plant and/or procedural modifications. The IPE took credit for several modifications that the licensee installed during the 14R refueling outage. These include installation of a hard piped containment vent system; operator training for manual initiation of the containment spray system; and installation of interconnection to the combustion turbine generators at the adjacent Forked River Site. The combustion turbine interconnection will make it possible to supply power from the combustion turbines directly to non-essential 4160 V bus 1A and emergency loads of essential 4160 V buses 1C and 1D via cross-tie.

Purchasing a portable power generator and developing procedures for recovering offsite or onsite power were identified as additional improvements for coping with station blackout. While the procedure development is underway, the licensee plans to evaluate the purchasing of an additional AC generator before the 15R refueling outage. The staff recognizes the licensee's intent to address station blackout events by the interconnection to the two combustion turbines and recovery of AC power procedure development.

IPE findings indicate that there are a number of additional "low-cost" improvements which could enhance overall reactor safety. These planned actions include:

- o Development of an emergency procedure for Loss of Offsite Power.
- o Development of an emergency procedure for Loss of DC Power.
- o Increased training on the importance of the containment spray system.
- o Changes to maintenance scheduling for the containment spray system to improve downtime.
- o Modifications to implement the Reactor Overfill Protection System.
- o Consider the development of specific guidance, training, and procedures for reactor overfill transients.
- o Increased emphasis in training on key operator actions as defined by the IPE.
- o Consideration of alternate containment heat removal capability to maintain minimal NPSH as part of Accident Management.
- o Alternate water supply for drywell sprays (Accident Management).