

March 15, 2006

Mr. L. M. Stinson
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 — REQUEST FOR
ADDITIONAL INFORMATION RE: USE OF BEST ESTIMATE LOCA ANALYSIS
(TAC NOS. MC8588 AND MC8589)

Dear Mr. Stinson:

By letter dated October 6, 2005, Southern Nuclear Operating Company requested a revision to the Joseph M. Farley Nuclear Plant Unit 1 and 2 Technical Specifications to allow the use of the best Westinghouse estimate loss-of-coolant accident analysis methodology in WCAP-16009-P-A, "Realistic Large Break LOCA Evaluation Methodology Using Automated Statistical Treatment of Uncertainty Method (ASTRUM)." The U.S. Nuclear Regulatory Commission staff has reviewed the application and has determined that additional information is required, as identified in the enclosure.

We discussed these issues with your staff on March 14, 2006. Your staff indicated that you would attempt to provide your response within 30 days of the date you receive the questions.

Please contact me at (301) 415-1493, if you have any other questions on these issues.

Sincerely,

/RA/

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosure: Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

The U. S. Nuclear Regulatory Commission (NRC) staff has reviewed Southern Nuclear Operating Company's (SNC's) submittal dated October 6, 2005, regarding the use of the Westinghouse best estimate large break loss-of-coolant accident (LBLOCA) analysis methodology. The NRC staff has identified the following information that is needed to enable the continuation of its review.

1. Please provide a statement to the effect that: SNC and its LBLOCA analyses vendor have ongoing processes that assure the input values and ranges of parameters for Farley Units 1 and 2 LBLOCA analyses conservatively bound the values and ranges of those parameters for the as-operated Farley plants. (This statement addresses certain programmatic requirements of Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.46(c).)
2. In Section 4.0, of its October 6, 2005 letter, SNC stated:
 - a) Therefore the Unit 2 model was utilized for the subsequent steps of the original application of the best estimate large break LOCA evaluation model.
 - b) Moreover, investigations revealed that the remaining differences in the vessels were small enough to justify the use of a single WCOBRA/TRAC geometric model for both Units 1 and 2.

While each plant may individually adopt the same calculation as its initial Analysis of Record (AOR), each plant must identify that analysis as its own specific AOR and track it separately from the other unit's AOR and AOR tracking, even if the changes and their calculated effects on peak cladding temperature (PCT) are identical for both plants. This is necessary because it is possible for differences to arise between the units that affect the LOCA analyses for each plant differently.

Please identify all differences between the two Farley units individually and the AOR analytical model, and their effects on calculated PCT, including the "remaining differences in the vessels." These differences and PCT effects must be reported in the initial AOR submittal, and subsequent emergency core cooling system reports, as required by 10 CFR 50.46 (a)(3) for each plant separately.

Enclosure

Joseph M. Farley Nuclear Plant

cc:

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