

February 22, 2006

Mr. Christopher M. Crane, President
and Chief Nuclear Officer
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: DRESDEN NUCLEAR POWER STATION, UNIT 3 - REQUEST FOR
ADDITIONAL INFORMATION RELATED TO RELIEF REQUEST CR-28
(TAC NO. MC7068)

Dear Mr. Crane:

By letter to the Nuclear Regulatory Commission (NRC) dated May 6, 2005, Exelon Generation Company, LLC requested relief from American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," and the augmented examinations specified in 10 CFR 50.55a(g)(6)(ii)(A)(2), for the Dresden Nuclear Power Station, Units 2 and 3.

The NRC staff is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested was e-mailed to your staff on January 25, 2006, and is addressed in the enclosure to this letter. During a discussion with your staff on February 7, 2006, it was agreed that you would provide a response to the RAI by March 10, 2006.

The NRC staff considers that timely responses to requests for additional information help ensure sufficient time is available for staff review and contribute toward the NRC's goal of efficient and effective use of staff resources. If circumstances result in the need to revise the requested response date, please contact me at (301) 415-2277.

Sincerely,

/RA/

Maitri Banerjee, Senior Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-249

Enclosure:
Request for Additional Information

cc w/encl: See next page

Dresden Nuclear Power Units 2 and 3

cc:

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REQUEST FOR ADDITIONAL INFORMATION

DRESDEN NUCLEAR POWER STATION, UNIT 3

DOCKET NO. 50-249

In reviewing the Exelon Generation Company's (Exelon's) submittal dated May 8, 2005, related to the request for relief from American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," and the augmented examinations specified in 10 CFR 50.55a(g)(6)(ii)(A)(2), for the Dresden Nuclear Power Station, Unit 3 (Dresden 3), the NRC staff has determined that the following information is needed in order to complete its review:

By letter dated March 23, 2005, the NRC approved an alternative Reactor Pressure Vessel (RPV) weld examination pursuant to Title 10 of the Code of Federal Regulations (10 CFR), Section 50.55a(a)(3)(I) and 10 CFR 50.55a(g)(6)(ii)(A)(5) for Dresden Nuclear Power Station, Units 2 and 3 (Dresden 2 and 3) that allows permanent deferral of the requirements to perform a volumetric examination of the RPV circumferential welds for the extended operating license terms for Dresden 2 and 3. This alternative requires examinations of essentially 100 percent of all longitudinal welds and examinations of approximately 2 to 3 percent of the circumferential welds at their points of intersection with the longitudinal welds.

In proposed Relief Request CR-28, Exelon did not indicate the extent to which the reduced examination volumes for the RPV longitudinal welds, as specified in the attachment to their May 6, 2005, letter, included the points of intersection with the circumferential welds. Please provide supplemental information indicating whether the reduced examination volumes for the specified Dresden 3 RPV longitudinal welds included all of the points of intersection with the circumferential welds. In addition, please indicate whether 2 to 3 percent volumetric coverage of the circumferential welds in the vicinity of the points of intersection with the longitudinal welds was achieved.

Enclosure