

February 9, 2006

Mr. Michael R. Kansler
President
Entergy Nuclear Operations, Inc.
440 Hamilton Avenue
White Plains, NY 10601

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT - REQUEST FOR
ADDITIONAL INFORMATION REGARDING RELIEF REQUEST RR-39,
IMPLEMENTATION OF BWRVIP GUIDELINES (TAC NO. MC8587)

Dear Mr. Kansler:

On October 7, 2005, Entergy Nuclear Operations, Inc. (Entergy), submitted a relief request for the James A. FitzPatrick Nuclear Power Plant which would implement various Boiling Water Reactor Vessel Internals Program (BWRVIP) guidelines in lieu of the American Society for Mechanical Engineers Boiler and Pressure Vessel Code inspection requirements on reactor vessel internals and components.

The Nuclear Regulatory Commission staff is reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). On February 6, 2006, the Entergy staff indicated that a response to the RAI would be provided within 60 days.

Please contact me at (301) 415-2901 if you have any questions on this issue.

Sincerely,

/RA/

John P. Boska, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure:
RAI

cc w/encl: See next page

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FitzPatrick Nuclear Power Plant

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FitzPatrick Nuclear Power Plant

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REQUEST FOR ADDITIONAL INFORMATION
REGARDING RELIEF REQUEST RR-39
ENTERGY NUCLEAR OPERATIONS, INC.
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333

By letter dated October 7, 2005, Agencywide Documents Access and Management System accession number ML052900075, Entergy Nuclear Operations Inc. (Entergy or the licensee) submitted relief request RR-39 for the James A. FitzPatrick Nuclear Power Plant (JAFNPP). The licensee proposes to use Boiling Water Reactor Vessel Internal Program (BWRVIP) guidelines as an alternative to certain requirements of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) for inservice inspection of Reactor Pressure Vessel (RPV) internal components.

The Nuclear Regulatory Commission (NRC) staff has reviewed the information the licensee provided that supports the proposed relief request and would like additional information to clarify the submittal:

1. According to Sections 3.3 and 3.4 of the BWRVIP-76, "BWR Vessel and Internals Project, BWR Core Shroud and Flaw Evaluation Guidelines" report, core shroud welds shall be inspected every 6 years when the enhanced visual test (EVT-1) method is used for one-sided weld inspections, and shall be inspected every 10 years when the subject welds are examined with the ultrasonic test (UT) method. The inspection frequency for the core shroud welds as indicated in Section 6.4 of the submittal is not consistent with the aforementioned requirement. Therefore, the NRC staff requests that the licensee revise the inspection frequency requirement for the subject welds in Section 6.4 of the submittal.
2. In Attachment 2 of the submittal, the licensee provides a comparison of ASME Category B—1 and B—2 examination requirements with the BWRVIP guidelines. The NRC staff requests that the licensee include the following welds in this attachment.
 - (A) ASME Item B13.30, Table IWB-2500-1, Shroud support welds H9 and H12.
 - (B) ASME Item B13.40, Table IWB-2500-1, Shroud horizontal welds H1 and H2.
 - (C) ASME Item B13.40, Table IWB-2500-1, Shroud H8 Attachment welds.
3. According to Figure 3-3 of the BWRVIP-76 report, vertical and top guide ring segment welds shall be inspected every 6 years when the EVT-1 is used for one-sided weld inspections and shall be inspected every 10 years when the subject welds are examined with the UT method. In Attachment 2 of the submittal, the inspection frequency

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requirement for the vertical and top guide ring segment welds (Item B13.40 in Table IWB-2500-1 of the ASME Code, Section XI) is not consistent with the aforementioned requirement specified in the BWRVIP-76 report. Therefore, the NRC staff requests that the licensee revise the inspection frequency requirement for the subject welds in Attachment 2 of the submittal.

4. In Attachment 2 of the submittal, the licensee indicates that the BWRVIP-38, "BWR Vessel Internal Project, BWR Shroud Support Inspection and Flaw Evaluation Guidelines," report will be used for inspecting core shroud attachment welds. The staff's final safety evaluation (SE) of the BWRVIP-38 report indicates that when inspection tooling and methodologies are developed that allow the welds in the lower plenum to be accessible, the guidelines will state that the licensee will inspect these welds with the appropriate non-destructive examination methods in order to establish a baseline for these welds. One of the core shroud attachment welds that has limited accessibility in the lower plenum is the H12 weld. Therefore, the NRC staff requests that the licensee revise Attachment 2 to include a commitment that the shroud support weld-H12 be inspected when the inspection tooling and methodologies permit such an inspection.
5. In Attachment 4 of the submittal, the licensee does not address the inspection requirements for the following welds, and the NRC staff requests that the licensee include them in this attachment.

Jet pumps riser welds-RS-4 and RS-5.

6. In a letter dated October 1, 2003, the licensee submitted a similar relief request for the Vermont Yankee Nuclear Power Station (VYNPS). The NRC staff approved the VYNPS relief request by an SE dated September 19, 2005. In Table 1 of the VYNPS relief request, the licensee provided details of its inspection requirements for the reactor vessel internals (RVI) components and the corresponding inspection bases that were delineated in the respective BWRVIP reports. The NRC staff requests that the licensee provide similar inspection requirements and the inspection frequency for the JAFNPP's RVI components. This information will enable the NRC staff to perform an effective review of the BWRVIP inspection criteria that will be used by the licensee for the JAFNPP's RVI components.