

SMUD

SACRAMENTO MUNICIPAL UTILITY DISTRICT
The Power To Do More.®

P.O. Box 15830, Sacramento, CA 95852-1830; 1-888-742-SMUD (7683)

MPC&D 06-017

January 25, 2006

U.S. Nuclear Regulatory Commission
Attn.: Document Control Desk
Washington, DC 20555

Docket No. 50-312
Rancho Seco Nuclear Generating Station
License No. DPR-54

EMBEDDED PIPING SCENARIO AND DCGL DETERMINATION BASIS

Attention: John Hickman

Attached is a copy of Decommissioning Technical Basis Document DTBD-05-009 "Embedded Piping Scenario and DCGL Determination Basis." This DTBD establishes the embedded piping scenario and the derived concentration guideline levels (DCGLs) that are appropriate for evaluating the internal surfaces of the embedded piping at Rancho Seco.

As they become available, we will submit additional DTBDs in anticipation that the review of these documents will facilitate NRC review and approval of the LTP to be submitted later this year. Members of your staff with questions requiring additional information or clarification may contact Bob Jones at (916) 732-4843.

Sincerely,

Steve Redeker
Manager, Plant Closure and Decommissioning

Attachment

Cc w/ attachment: B.S. Mallett, NRC, Region IV

Nmss01

**Rancho Seco Nuclear Generating Station
Decommissioning Technical Basis Document**

DTBD-05-009

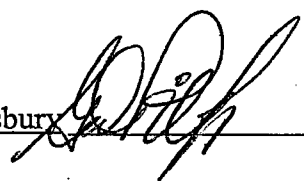
Revision No. 0

DPT 05-062

RIC 2A.900

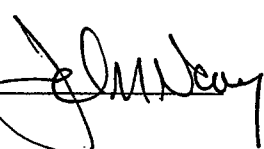
Embedded Piping Scenario and DCGL Determination Basis

PREPARED BY: George Pillsbury
Author



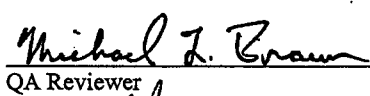
9/26/05
Date

REVIEWED BY: John Newey
Technical Reviewer



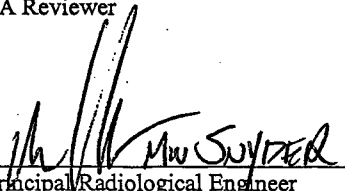
11/15/05
Date

REVIEWED BY: Michael J. Brown
QA Reviewer



11/30/05
Date

APPROVED BY: M. W. Snyder
Principal Radiological Engineer



12/10/05
Date

1.0 PURPOSE

The purpose of this Decommissioning Technical Basis Document (DTBD) is to establish the embedded piping scenario and the derived concentration guideline levels (DCGLs) that are appropriate for evaluating the internal surfaces of embedded piping associated with the Rancho Seco Nuclear Generating station (RSNGS).

2.0 DISCUSSION

The methodology established in this DTBD is to be used as the basis for determining the disposition of embedded piping systems associated with RSNGS. Because embedded piping is located within concrete structures, the dose contribution is primarily whole body gamma dose to industrial workers within a room or area containing embedded piping. This dose contribution is additive to the whole body dose from contamination on the interior surfaces of the room in which the pipe is embedded.

Although not part of the building occupancy scenario, consideration was also given to the possibility of a future worker removing the pipe from within a wall or floor and being exposed to its residual contamination to ensure the grouting action level was appropriate.

3.0 DEFINITIONS

None

4.0 TECHNICAL POSITION

Only the gamma dose reaching the worker through the wall or floor must be considered in determining the DCGL. Pipe removal is considered to be a low-probability event which is further mitigated by grouting at a very conservative level of internal contamination.

5.0 LIMITATIONS

This DTBD applies only to piping embedded within concrete walls or floors.

6.0 TECHNICAL BASES

6.1 Scope of Embedded Pipe

Approximately 5360 linear feet of embedded pipe have been identified at RSNGS. The embedded pipe ranges from 0.75 inch I.D. to 18 inch I.D. and is associated with the Turbine Building, Auxiliary Building, Reactor Building, and Fuel Building drains (Attachment 8.1). Embedded pipe is located at depths between 9 to 30 inches beneath the concrete surface, depending on the building.

The basic dose scenario for embedded pipe is one in which the industrial worker receives gamma dose from the residual activity located within the concrete-encased piping in addition to the dose received from the contaminated surfaces of the room.

6.2 Embedded Pipe Radionuclides and DCGLs

Embedded pipe is partially shielded and constrained by the encasing concrete which limits the dose to the industrial worker to that arising from the gamma emitters in the nuclide mixture. The impact of nuclides which are not gamma emitters is minimal because the pipe is not easily extracted nor is the interior surface readily accessible through the overlying concrete. A conservative DCGL of 100,000 dpm/100 cm² has been evaluated and found acceptable for embedded pipe. Microshield runs were used to model the gamma exposure from 100,000 dpm/100 cm² (4.5E-4 uCi/cm²) in the maximum size pipe in a given building one meter from the surface of the concrete covering the embedded pipe (Attachment 8.2). The amount of the concrete shielding included in the model was based on the thinnest concrete covering above the largest diameter embedded pipe for the given building as determined from site drawings. Results are shown in Table 1 below. The annual dose rates are all less than 1 percent of the 25 mrem/y annual limit.

Building	Turbine Bld	Fuel Bld	Auxiliary Bld	Reactor Bld
Max Pipe Size	8"	8"	6"	18"
Concrete Depth	18"	30"	9"	12"
Annual Dose Rate	0.01 mrem/y	0.0002 mrem/y	0.19 mrem/y	0.12 mrem/y

A total of 53 samples were collected and analyzed by gamma spectroscopy from various drains, sumps, and trenches in the buildings previously mentioned. The 20 samples found in Attachments 8.3 and 8.6 reflect the different piping systems covered by the 53 samples. In many instances, several samples were collected from one system. As can be seen from the nuclide analyses, the primary contributors to whole body dose are Cs-137 and Co-60 (Attachment 8.3). The Fuel Building pipe sample results indicate the presence of a small portion of non-gamma emitters in the nuclide fraction. (The mean nuclide fractions for Cs-137 and Co-60, as determined by the 20 samples, were 0.802 and 0.161 respectively. The individual building mean fractions were within 2 standard deviations of the overall mean values indicating a consistent nuclide ratio. This compares well with the concrete nuclide fractions of 0.86 and 0.14 for Cs-137 and Co-60 respectively.)

Furthermore, a decision was made to grout embedded pipe when the residual activity exceeded the NRC Screening Levels. Five of the pipe samples were submitted for hard to detect analysis. The sample analyses and subsequent evaluation resulted in DCGL values for grouting based on an adjustment to the NRC Screening Levels for hard to detect nuclides. The mean resulting grout DCGL was 21,040 dpm/100 cm² based on the gamma-emitting nuclide fractions (Attachment 8.4). This value will be used as an action level for grouting embedded piping.

6.3 Embedded Piping Scenario

The embedded piping scenario assumes that the piping remains in place following decommissioning and that the dose to the industrial worker is from direct gamma exposure from the residual activity in the pipe with allowance made for photon attenuation by the wall or floor thickness of concrete remaining over the pipe. Whole body dose from the embedded pipe will be considered additive along with the dose to the industrial worker resulting from residual activity on the walls or floors of the room or area in which the embedded pipe is present. The structure surface DCGL will be reduced by the dose contribution from the embedded piping in order to ensure compliance with the annual dose limit.

The whole body dose rate for pipe contamination at the 100,000 dpm/100 cm² DCGL has been calculated as described above using Microshield. Dose rates at a distance of 1 m from the wall or floor surface have been determined assuming the minimum thickness of concrete remaining over the largest diameter pipe. Based on 2000 hours per year of exposure, the dose to the industrial worker would be 0.06 mrem/y to 2.02 mrem/y depending on the building if all of the activity is assumed to be Co-60. If the actual nuclide fractions are employed, the annual dose rate ranges from 0.01 mrem/y to 0.19 mrem/y as shown in Table 1. If the assumed indoor exposure time for the industrial worker is considered, the annual dose rates would be further reduced by a factor of two.

The potential for the removal of the embedded pipe and consequent dose to an industrial worker at some time in the future was examined even though this was not part of the building occupancy scenario. The published source of dose factors that came the closest to a pipe cutting and removal scenario was NUREG-1640. Assuming a DCGL of 100,000 dpm/100 cm² for embedded pipe, results in an annual dose rate of 3.96 mrem/y for Cs-137 and 2.7 mrem/y for Co-60 if the mean dose factors (NUREG-1640, Table 3.24) and scenario for converting pipe into scrap material as outlined in NUREG-1640 are employed. The dose contribution from Cs-137 was principally from the release of contamination and subsequent inhalation by the worker while the dose from Co-60 was mostly the whole body dose from handling the contaminated pipe. In order to preclude the additional dose contribution from embedded pipe, RSNCS plans to grout piping which has residual contamination above the adjusted NRC screening levels (Table 5.19 of NUREG-5512 V3). A conservatively applied, adjusted screening value for embedded pipe is 21,040 dpm/100 cm² (Attachment 8.4). Using 20,000 dpm/100 cm² and the NUREG-1640, Table 3.24 values would result in a mean annual dose rate of 1.34 mrem/y (20,000 dpm/220 dpm/pCi/cm² x 3.8E-2 mrem/y/pCi/cm² Co + 20,000 dpm/220 dpm/pCi/cm² x 1.1E-2 mrem/y/pCi/cm² Cs adjusted for a 80:16 Cs to Co

ratio), if the ungrouted piping was removed immediately following decommissioning without any further controls established. Alternatively, if the 20,000 dpm/100 cm² residual contamination is converted into a specific activity value assuming a contamination layer thickness of 0.1 cm and a density for steel pipe of 7.86 g/cm³, annual dose rates for inhalation and ingestion can be determined using the Building Renovation scenario factors found in NUREG-5512.

$$\begin{aligned}\text{Pipe activity} &= 20,000 \text{ dpm}/100 \text{ cm}^2 \times \text{cm}^3/7.86 \text{ g} \times \text{pCi}/2.22 \text{ dpm} \times \text{cm}^2/0.1 \text{ cm}^3 \\ &= 114.6 \text{ pCi/g}\end{aligned}$$

$$\text{Inhalation dose rate} = 114.6 \text{ pCi/g} \times 9.5\text{E-}6 \text{ mrem/y per pCi/g Co-60}$$

$$= 1.09\text{E-}3 \text{ mrem/y Co-60 (Cs-137 at } 6.7\text{E-}5 \text{ mrem/y} = 7.7\text{E-}5 \text{ mrem/y)}$$

for a 80:16 Cs to Co ratio, the total dose rate would be 2.36E-4 mrem/y.

$$\text{Ingestion dose rate} = 114.6 \text{ pCi/g} \times 5.0\text{E-}5 \text{ mrem/y per pCi/g Co-60}$$

$$= 0.006 \text{ mrem/y Co-60 (Cs-137 at } 1.2\text{E-}4 \text{ mrem/y} = 0.014 \text{ mrem/y)}$$

for a 80:16 Cs to Co ratio, the total dose rate would be 0.012 mrem/y.

The combined internal annual dose rate for pipe removal would be 0.012 mrem/y.

These two methods of dose analysis place bounds on pipe removal activities of 0.012 mrem/y to 1.34 mrem/y or 0.05% and 5.4% of the annual dose limit.

Given the plans for the site following decommissioning, such an exposure scenario (removal of un-grouted embedded pipe) has a very low probability of occurrence along with a very low potential annual dose rate.

6.4 Embedded Pipe Area Factors

Area Factors will vary with the internal diameter (I.D.) of the pipe being surveyed. In order to calculate specific area factors, determine the limiting length of pipe in the room or area being surveyed. Calculate the interior surface area for the given pipe I.D. Enter the specific physical dimensions into Microshield using the Cylinder Surface, External Dose Point Geometry. Perform subsequent calculations while reducing the pipe length for each run until the dose rates bound the hot spot area of interest. Divide the dose rate for the initial pipe size by the dose rate for the reduced lengths of pipe. The resulting ratios are the area factors for the given pipe size.

7.0 REFERENCES

- 7.1 NUREG-1640, Radiological Assessments for Clearance of Equipment and Materials from Nuclear Facilities.
- 7.2 NUREG/CR-5512 V1, Residual Radioactive Contamination From Decommissioning.
- 7.3 NUREG/CR-5512 V3, Residual Radioactive Contamination From Decommissioning.

8.0 ATTACHMENTS

- 8.1 Embedded Pipe Inventory
- 8.2 Microshield Results For Embedded Piping
- 8.3 Nuclide Fraction For Embedded Pipe
- 8.4 Adjusted Screening Value DCGLs For Embedded Pipe

9.0 RESPONSIBLE INDIVIDUAL

George D. Pillsbury

Attachment 8.1**Page 1****Embedded Piping Inventory
(linear feet)**

Size (inches)	Turbine Bld	Auxiliary Bld	Reactor Bld	Fuel Bld	Total
0.75	-	-	-	20	20
1	-	45	-	266	311
2	405	1264	352	267	2288
2.5	-	-	-	182	182
3	59	50	-	39	148
4	1164	513	53	-	1730
6	202	178	44	-	424
8	72	-	-	53	125
16	-	-	-	5	5
18	-	-	121	-	121
Total	1902	2050	570	832	5354

Turbine Building

Size (inches)	Below Grade	Grade	+40	Total
2	59	346	-	405
3	-	59	-	59
4	536	560	68	1164
6	32	170	-	202
8	27	45	-	72

Auxiliary Building

Size (inches)	-47	-20	Grade	Total
1	-	45	-	45
2	86	1044	134	1264
3	-	50	-	50
4	44	464	5	513
6	71	107	-	178

**Attachment 8.1
Page 2 (Continued)**

Reactor Building

Size (inches)	-26	+40	Total
2	180	172	352*
4	53	-	53
6	44	-	44
18	121	-	121

* Removal of Reactor Building interior concrete will eliminate 2" pipe.

Fuel Building

Size (inches)	Grade	Vertical	+40	Total
0.75	-	-	20	20
1	224	42	-	266
2	-	-	267	267
2.5	-	31	151	182
3	-	29	10	39
8	-	53	-	53
16	-	5**	-	5

**This is a 5 foot throughwall pipe penetration.

Attachment 8.2

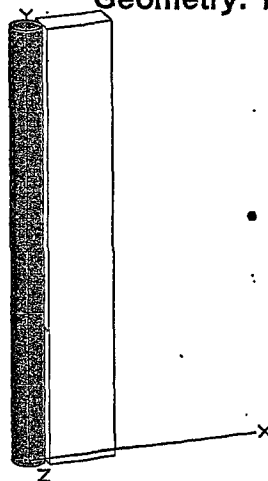
Microshield Results For Embedded Pipe

MicroShield v5.05 (5.05-00488)
Sacramento Municipal Utility District

Page : 1
DOS File : EMBD_D2.MS5
Run Date : November 7, 2005
Run Time : 2:50:07 PM
Duration : 00:00:00

File Ref: _____
Date: _____
By: _____
Checked: _____

Case Title: 8" Embedded Pipe
Description: Turbine Bld 18" Concrete
Geometry: 10 - Cylinder Surface - External Dose Point



Source Dimensions

Height	305.0 cm	10 ft 0.1 in
Radius	10.0 cm	3.9 in

Dose Points

#	X	Y	Z
# 1	156.52 cm 5 ft 1.6 in	152.5 cm 5 ft 0.0 in	0 cm 0.0 in

Shields

Shield Name	Dimension	Material	Density
Cyl. Core	10.0 cm ²	Air	0.00122
Transition		Air	0.00122
Shield 2	45.72 cm	Concrete	2.4
Air Gap		Air	0.00122
Wall Clad	.8 cm	Iron	7.86

Source Input

Grouping Method : Actual Photon Energies

Nuclide	curies	becquerels	μCi/cm ²	Bq/cm ²
Ba-137m	6.9181e-006	2.5597e+005	3.6100e-004	1.3357e+001
Co-60	1.3875e-006	5.1336e+004	7.2400e-005	2.6788e+000
Cs-137	6.9181e-006	2.5597e+005	3.6100e-004	1.3357e+001

Buildup

The material reference is : Shield 2

Integration Parameters

Y Direction (axial)	20
Circumferential	20

Results

Energy MeV	Activity photons/sec	Fluence Rate		Exposure Rate	
		No Buildup	With Buildup	No Buildup	With Buildup
0.0318	5.299e+03	5.575e-71	8.312e-30	4.644e-73	6.924e-32
0.0322	9.777e+03	1.050e-68	1.594e-29	8.451e-71	1.283e-31
0.0364	3.558e+03	6.983e-52	8.881e-30	3.968e-54	5.046e-32

Page : 2
 DOS File : EMBD_D2.MS5
 Run Date : November 7, 2005
 Run Time : 2:50:07 PM
 Duration : 00:00:00

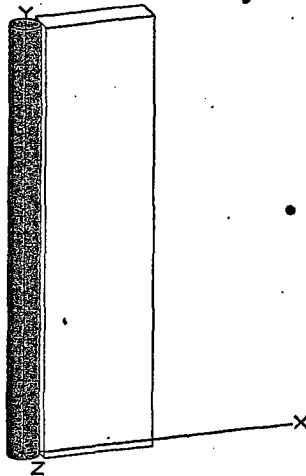
<u>Energy</u> <u>MeV</u>	<u>Activity</u> <u>photons/sec</u>	<u>Fluence Rate</u> <u>MeV/cm²/sec</u> <u>No Buildup</u>	<u>Fluence Rate</u> <u>MeV/cm²/sec</u> <u>With Buildup</u>	<u>Exposure Rate</u> <u>mR/hr</u> <u>No Buildup</u>	<u>Exposure Rate</u> <u>mR/hr</u> <u>With Buildup</u>
0.6616	2.303e+05	1.895e-05	5.237e-04	3.675e-08	1.015e-06
0.6938	8.374e+00	8.858e-10	2.283e-08	1.710e-12	4.408e-11
1.1732	5.134e+04	7.541e-05	9.165e-04	1.348e-07	1.638e-06
1.3325	5.134e+04	1.362e-04	1.402e-03	2.363e-07	2.433e-06
TOTALS:	3.516e+05	2.305e-04	2.842e-03	4.078e-07	5.086e-06

MicroShield v5.05 (5.05-00488)
Sacramento Municipal Utility District

Page : 1
DOS File : EMBD_D2.MS5
Run Date : November 7, 2005
Run Time : 2:52:25 PM
Duration : 00:00:01

File Ref: _____
Date: _____
By: _____
Checked: _____

Case Title: 8" Embedded Pipe
Description: Fuel Bld 30" Concrete
Geometry: 10 - Cylinder Surface - External Dose Point



Source Dimensions			
Height	305.0 cm	10 ft 0.1 in	
Radius	10.0 cm	3.9 in	

Dose Points			
	X	Y	Z
# 1	187 cm 6 ft 1.6 in	152.5 cm 5 ft 0.0 in	0 cm 0.0 in

Shields			
Shield Name	Dimension	Material	Density
Cyl. Core	10.0 cm ²	Air	0.00122
Transition		Air	0.00122
Shield 2	76.2 cm	Concrete	2.4
Air Gap		Air	0.00122
Wall Clad	.8 cm	Iron	7.86

Source Input

Nuclide	Grouping Method : Actual Photon Energies			
	curies	becquerels	µCi/cm ²	Bq/cm ²
Ba-137m	6.9181e-006	2.5597e+005	3.6100e-004	1.3357e+001
Co-60	1.3875e-006	5.1336e+004	7.2400e-005	2.6788e+000
Cs-137	6.9181e-006	2.5597e+005	3.6100e-004	1.3357e+001

Buildup

The material reference is : Shield 2

Integration Parameters

Y Direction (axial)	20
Circumferential	20

Results

Energy MeV	Activity photons/sec	Fluence Rate MeV/cm ² /sec No Buildup	Fluence Rate MeV/cm ² /sec		Exposure Rate mR/hr	
			With Buildup	With Buildup	No Buildup	With Buildup
0.0318	5.299e+03	2.567e-102	6.159e-30	2.138e-104	5.130e-32	
0.0322	9.777e+03	4.011e-99	1.181e-29	3.228e-101	9.503e-32	
0.0364	3.558e+03	1.937e-74	6.581e-30	1.101e-76	3.739e-32	

Page : 2
 DOS File : EMBD_D2.MS5
 Run Date: November 7, 2005
 Run Time: 2:52:25 PM
 Duration : 00:00:01

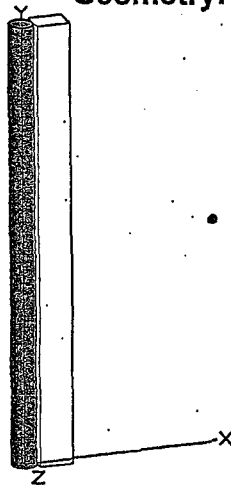
<u>Energy</u> MeV	<u>Activity</u> photons/sec	<u>Fluence Rate</u> MeV/cm ² /sec		<u>Exposure Rate</u> mR/hr	
		No Buildup	With Buildup	No Buildup	With Buildup
0.6616	2.303e+05	4.468e-08	2.663e-06	8.662e-11	5.162e-09
0.6938	8.374e+00	2.355e-12	1.290e-10	4.546e-15	2.491e-13
1.1732	5.134e+04	6.918e-07	1.543e-05	1.236e-09	2.758e-08
1.3325	5.134e+04	1.640e-06	2.991e-05	2.845e-09	5.188e-08
TOTALS:	3.516e+05	2.376e-06	4.800e-05	4.168e-09	8.462e-08

MicroShield v5.05 (5.05-00488)
Sacramento Municipal Utility District

Page : 1
DOS File : EMBD_D2.MS5
Run Date : November 7, 2005
Run Time : 2:53:47 PM
Duration : 00:00:01

File Ref: _____
Date: _____
By: _____
Checked: _____

Case Title: 6" Embedded Pipe
Description: Auxilliary Bld 9" Concrete
Geometry: 10 - Cylinder Surface - External Dose Point



Source Dimensions

Height	305.0 cm	10 ft 0.1 in
Radius	7.62 cm	3.0 in

Dose Points

#	X	Y	Z
# 1	131.18 cm 4 ft 3.6 in	152.5 cm 5 ft 0.0 in	0 cm 0.0 in

Shields

Shield Name	Dimension	Material	Density
Cyl. Core	7.62 cm ²	Air	0.00122
Transition		Air	0.00122
Shield 2	22.86 cm	Concrete	2.4
Air Gap		Air	0.00122
Wall Clad	.7 cm	Iron	7.86

Source Input

Grouping Method : Actual Photon Energies

Nuclide	curies	becquerels	µCi/cm ²	Bq/cm ²
Ba-137m	5.2716e-006	1.9505e+005	3.6100e-004	1.3357e+001
Co-60	1.0572e-006	3.9118e+004	7.2400e-005	2.6788e+000
Cs-137	5.2716e-006	1.9505e+005	3.6100e-004	1.3357e+001

Buildup

The material reference is : Shield 2

Integration Parameters

Y Direction (axial)	20
Circumferential	20

Results

Energy MeV	Activity photons/sec	Fluence Rate		Exposure Rate	
		No Buildup	With Buildup	No Buildup	With Buildup
0.0318	4.038e+03	3.340e-45	8.413e-30	2.782e-47	7.008e-32
0.0322	7.450e+03	1.079e-43	1.613e-29	8.681e-46	1.298e-31
0.0364	2.711e+03	1.985e-33	8.989e-30	1.128e-35	5.107e-32

Page : 2
 DOS File : EMBD_D2.MS5
 Run Date : November 7, 2005
 Run Time : 2:53:47 PM
 Duration : 00:00:01

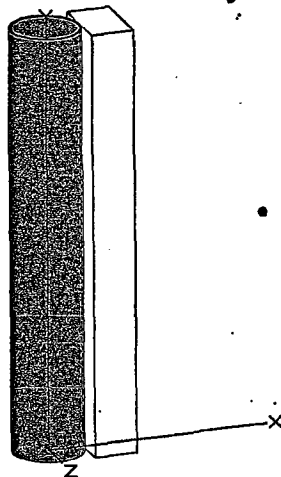
<u>Energy</u> MeV	<u>Activity</u> photons/sec	<u>Fluence Rate</u> MeV/cm ² /sec		<u>Exposure Rate</u> mR/hr	
		<u>No Buildup</u>	<u>With Buildup</u>	<u>No Buildup</u>	<u>With Buildup</u>
0.6616	1.755e+05	1.718e-03	1.924e-02	3.330e-06	3.730e-05
0.6938	6.381e+00	7.318e-08	7.769e-07	1.413e-10	1.500e-09
1.1732	3.912e+04	2.394e-03	1.435e-02	4.278e-06	2.564e-05
1.3325	3.912e+04	3.502e-03	1.850e-02	6.076e-06	3.209e-05
TOTALS:	2.679e+05	7.613e-03	5.209e-02	1.368e-05	9.503e-05

MicroShield v5.05 (5.05-00488)
 Sacramento Municipal Utility District

Page : 1
 DOS File : EMBD_D2.MS5
 Run Date: November 7, 2005
 Run Time: 2:55:23 PM
 Duration : 00:00:01

File Ref: _____
 Date: _____
 By: _____
 Checked: _____

Case Title: 18" Embedded Pipe
 Description: Reactor Bld 12" Concrete
 Geometry: 10 - Cylinder Surface - External Dose Point



Source Dimensions

Height	305.0 cm	10 ft 0.1 in
Radius	22.86 cm	9.0 in

Dose Points

	X	Y	Z
# 1	154.26 cm 5 ft 0.7 in	152.5 cm 5 ft 0.0 in	0 cm 0.0 in

Shields

Shield Name	Dimension	Material	Density
Cyl. Core	22.86 cm ²	Air	0.00122
Transition		Air	0.00122
Shield 2	30.0 cm	Concrete	2.4
Air Gap		Air	0.00122
Wall Clad	1.4 cm	Iron	7.86

Source Input

Grouping Method : Actual Photon Energies

Nuclide	curies	becquerels	μCi/cm ²	Bq/cm ²
Ba-137m	1.5815e-005	5.8515e+005	3.6100e-004	1.3357e+001
Co-60	3.1717e-006	1.1735e+005	7.2400e-005	2.6788e+000
Cs-137	1.5815e-005	5.8515e+005	3.6100e-004	1.3357e+001

Buildup

The material reference is : Shield 2

Integration Parameters

Y Direction (axial)	20
Circumferential	20

Results

Energy MeV	Activity photons/sec	Fluence Rate		Exposure Rate	
		No Buildup	With Buildup	No Buildup	With Buildup
0.0318	1.211e+04	1.956e-68	1.966e-29	1.629e-70	1.638e-31
0.0322	2.235e+04	3.555e-66	3.770e-29	2.861e-68	3.034e-31
0.0364	8.134e+03	1.914e-49	2.101e-29	1.087e-51	1.194e-31

Page : 2
 DOS File : EMBD_D2.MS5
 Run Date : November 7, 2005
 Run Time : 2:55:23 PM
 Duration : 00:00:01

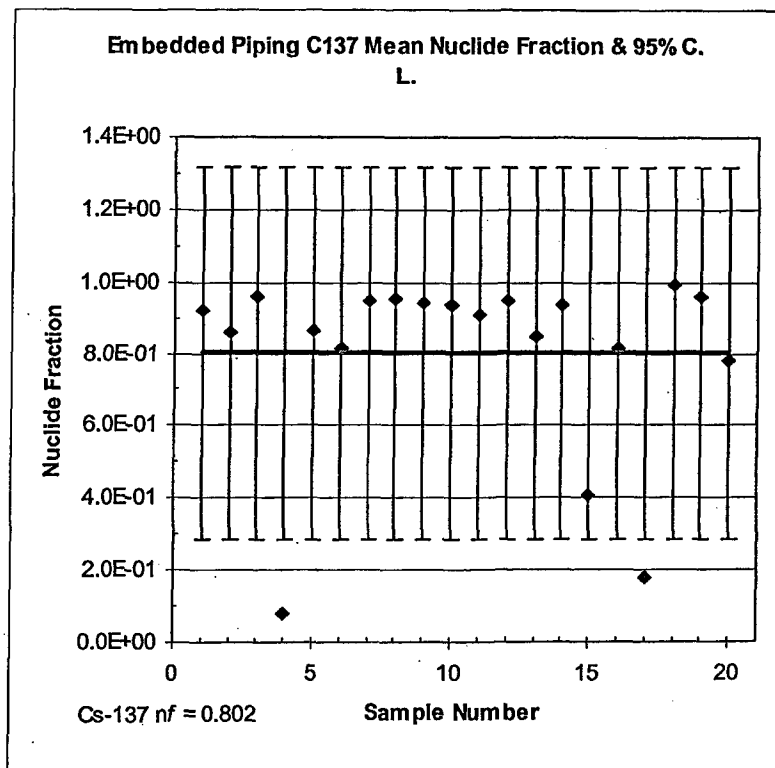
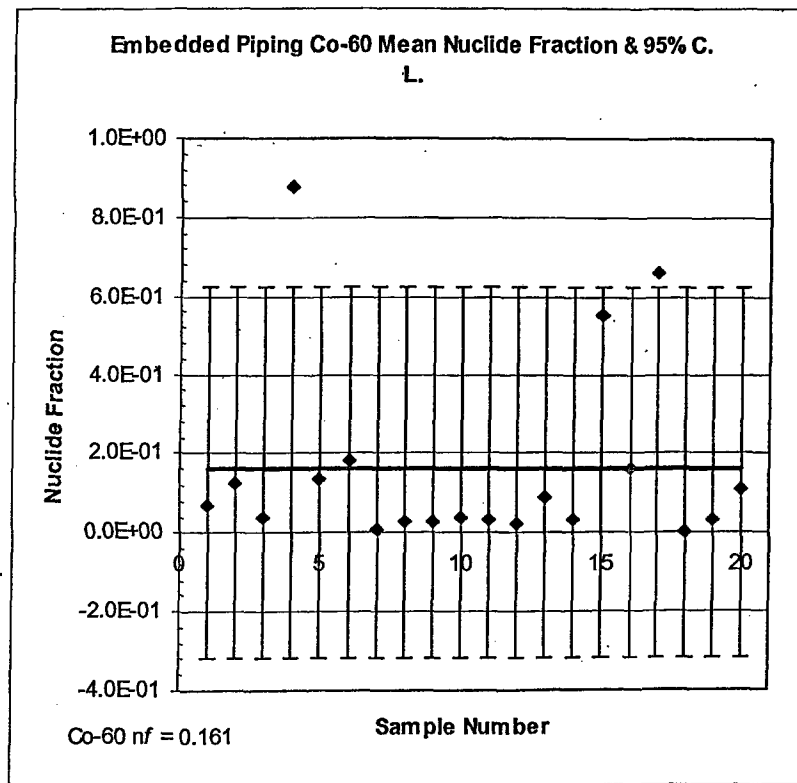
<u>Energy</u> <u>MeV</u>	<u>Activity</u> <u>photons/sec</u>	<u>Fluence Rate</u> <u>MeV/cm²/sec</u>		<u>Exposure Rate</u> <u>mR/hr</u>	
		<u>No Buildup</u>	<u>With Buildup</u>	<u>No Buildup</u>	<u>With Buildup</u>
0.6616	5.265e+05	5.581e-04	9.591e-03	1.082e-06	1.859e-05
0.6938	1.914e+01	2.473e-08	3.998e-07	4.775e-11	7.719e-10
1.1732	1.174e+05	1.214e-03	1.022e-02	2.170e-06	1.827e-05
1.3325	1.174e+05	1.942e-03	1.415e-02	3.369e-06	2.454e-05
TOTALS:	8.038e+05	3.714e-03	3.396e-02	6.621e-06	6.141e-05

Attachment 8.3 Gamma Nuclide Fraction For Embedded Piping

Nuclide	Turbine Air	Turbine	Aux Acid	SFP 16"	Aux Drain	Aux Drain	SFP #3	SFP	SFP	SFP	SFP
	Air Drier #6	Drain	Drains #1	DHS	Common	Common	Leak	Leak-Chase	Leak Chase	Leak Chase	Leak Chase
	Normalized	Normalized	Normalized	Normalized	Normalized	Normalized	Chase	#3	#1	#2	#4
Co-60	6.916E-02	1.255E-01	3.691E-02	8.757E-01	1.346E-01	1.839E-01	6.304E-03	2.628E-02	2.822E-02	3.498E-02	3.191E-02
Nb-94	1.202E-03	1.926E-03	3.512E-04	1.828E-02	1.333E-03	5.336E-04	< 1.060E-03	< 9.061E-04	< 1.431E-03	< 1.241E-03	< 2.078E-03
Ag-108m	2.777E-03	3.420E-03	4.086E-04	< 7.336E-03	7.234E-04	2.299E-04	3.205E-02	1.124E-02	1.537E-02	1.705E-02	4.427E-02
Sb-125	3.453E-03	4.246E-03	< 2.937E-04	< 6.313E-03	2.646E-04	2.990E-04	< 6.225E-03	< 3.955E-03	< 5.487E-03	4.513E-03	6.308E-03
Cs-134	5.712E-04	< 8.833E-04	4.318E-04	< 2.082E-03	4.579E-04	2.499E-04	< 9.345E-04	< 5.941E-04	< 8.893E-04	< 7.088E-04	< 1.016E-03
Cs-137	9.182E-01	8.574E-01	9.613E-01	7.584E-02	8.618E-01	8.116E-01	9.480E-01	9.537E-01	9.433E-01	9.367E-01	9.075E-01
Eu-154	2.388E-03	< 3.545E-03	< 1.538E-04	< 4.490E-03	6.972E-04	2.821E-03	< 3.113E-03	< 1.892E-03	< 2.793E-03	< 2.445E-03	< 3.155E-03
Eu-155	< 2.293E-03	< 3.128E-03	< 1.458E-04	< 9.934E-03	1.303E-04	3.657E-04	< 2.357E-03	< 1.422E-03	< 2.478E-03	< 2.406E-03	< 3.808E-03
	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00
Co-60 mean	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.412E-01
Cs-137 mean	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.341E-01

Nuclide	SFP	SFP	SFP	SFP 8"	SFP 3"	SFP 2.5"	7' NE B	9' E B Sump	S +40 EL	Mean	Standard Deviation
	Leak Chase	Leak Chase	Leak Chase	Cooler Pipe	Skimmer	Nozzle	SD8110351	SD8110351	SF8990420		
	#7	#5	#8	Pipe	Pipe	Return	SC01A	SC01A	PM01		
Co-60	2.335E-02	8.803E-02	3.213E-02	5.547E-01	1.632E-01	6.596E-01	1.188E-03	3.191E-02	1.115E-01	1.610E-01	2.426E-01
Nb-94	< 1.483E-03	< 6.893E-03	< 1.532E-03	1.592E-02	1.096E-02	< 1.965E-02	< 2.002E-04	< 4.189E-04	< 1.172E-02	4.955E-03	6.526E-03
Ag-108m	1.945E-02	2.274E-02	1.623E-02	7.891E-03	3.329E-03	7.389E-02	< 2.456E-04	< 5.735E-03	1.384E-02	1.491E-02	1.804E-02
Sb-125	< 7.352E-04	< 1.370E-02	< 6.110E-03	< 4.424E-03	3.764E-03	< 1.880E-02	< 1.383E-03	< 1.029E-03	< 3.051E-02	6.091E-03	7.325E-03
Cs-134	< 1.240E-03	< 2.180E-03	< 9.643E-04	< 1.237E-03	9.786E-04	< 5.970E-03	2.536E-04	2.084E-04	< 6.450E-04	1.125E-03	1.257E-03
Cs-137	9.462E-01	8.481E-01	9.369E-01	4.074E-01	8.151E-01	1.775E-01	9.942E-01	9.590E-01	7.828E-01	8.021E-01	2.631E-01
Eu-154	< 3.665E-03	< 6.736E-03	< 2.956E-03	< 2.749E-03	9.402E-04	< 1.261E-02	< 2.300E-03	< 1.208E-03	< 1.816E-02	3.941E-03	4.259E-03
Eu-155	< 3.895E-03	< 1.163E-02	< 3.125E-03	< 5.663E-03	< 1.764E-03	< 3.195E-02	< 2.542E-04	< 4.928E-04	< 3.077E-02	5.901E-03	9.219E-03
	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00	1.000E+00
Co-60 mean	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01	1.610E-01
Cs-137 mean	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01	8.021E-01

Attachment 8.3 Gamma Nuclide Fraction For Embedded Piping



Attachment 8.4 Adjusted Screening Value DCGLs

GEL results for Turbine Air Driers #6

		SB8990070		NRC		
		PM02	Screening			
7/1/2008	Normalized	Level DCGL				
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	6.916E-02	7.050E+03	9.810E-06	2.262E+01	6.916E-02	
Nb-94	< 1.202E-03	8.290E+03	1.449E-07	3.342E-01	1.202E-03	
Ag-108m	< 2.777E-03	1.020E+04	2.723E-07	6.279E-01	2.777E-03	
Sb-125	< 3.453E-03	4.430E+04	7.794E-08	1.797E-01	3.453E-03	
Cs-134	< 5.712E-04	1.270E+04	4.498E-08	1.037E-01	5.712E-04	
Cs-137	9.182E-01	2.800E+04	3.279E-05	7.562E+01	9.182E-01	
Eu-154	< 2.388E-03	1.150E+04	2.076E-07	4.789E-01	2.388E-03	
Eu-155	< 2.293E-03	1.570E+05	1.460E-08	3.368E-02	2.293E-03	
	1.000E+00		4.336E-05	1.000E+02	1.00e+00	
		DCGL Total	2.306E+04			

1 GEL results for Turbine Drain Sump #5

		SB8990070		NRC		
		PM01	Screening			
7/1/2008	Normalized	Level DCGL				
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	1.255E-01	7.050E+03	1.780E-05	3.597E+01	1.255E-01	
Nb-94	1.926E-03	8.290E+03	2.323E-07	4.695E-01	1.926E-03	
Ag-108m	3.420E-03	1.020E+04	3.353E-07	6.776E-01	3.420E-03	
Sb-125	4.246E-03	4.430E+04	9.585E-08	1.937E-01	4.246E-03	
Cs-134	< 8.833E-04	1.270E+04	6.955E-08	1.406E-01	8.833E-04	
Cs-137	< 8.574E-01	2.800E+04	3.062E-05	6.188E+01	8.574E-01	
Eu-154	< 3.545E-03	1.150E+04	3.083E-07	6.230E-01	3.545E-03	
Eu-155	< 3.128E-03	1.570E+05	1.993E-08	4.027E-02	3.128E-03	
	1.000E+00		4.948E-05	1.000E+02	1.000E+00	
		DCGL Total	2.021E+04			

GEL results for Aux Acid Drains #1

		SE8990420		NRC		
		PM01A	Screening			
7/1/2008	Normalized	Level DCGL				
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	3.691E-02	7.050E+03	5.235E-06	1.319E+01	3.691E-02	
Nb-94	3.512E-04	8.290E+03	4.237E-08	1.067E-01	3.512E-04	
Ag-108m	4.086E-04	1.020E+04	4.006E-08	1.009E-01	4.086E-04	
Sb-125	< 2.937E-04	4.430E+04	6.630E-09	1.670E-02	2.937E-04	
Cs-134	4.318E-04	1.270E+04	3.400E-08	8.563E-02	4.318E-04	
Cs-137	9.613E-01	2.800E+04	3.433E-05	8.647E+01	9.613E-01	
Eu-154	< 1.538E-04	1.150E+04	1.337E-08	3.368E-02	1.538E-04	
Eu-155	< 1.458E-04	1.570E+05	9.285E-10	2.338E-03	1.458E-04	
	1.000E+00		3.971E-05	1.000E+02	1.000E+00	
		DCGL Total	2.519E+04			

3 Results for SFP 16" DHS Piping

		SA899010		NRC		
		PM01	Screening			
7/1/5008	Normalized	Level DCGL				
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	8.757E-01	7.050E+03	1.242E-04	9.479E+01	8.757E-01	
Nb-94	1.828E-02	8.290E+03	2.205E-06	1.682E+00	1.828E-02	
Ag-108m	< 7.336E-03	1.020E+04	7.192E-07	5.488E-01	7.336E-03	
Sb-125	< 6.313E-03	4.430E+04	1.425E-07	1.087E-01	6.313E-03	
Cs-134	< 2.082E-03	1.270E+04	1.640E-07	1.251E-01	2.082E-03	
Cs-137	7.584E-02	2.800E+04	2.708E-06	2.067E+00	7.584E-02	
Eu-154	< 4.490E-03	4.430E+04	1.013E-07	7.734E-02	4.490E-03	
Eu-155	< 9.934E-03	1.270E+04	7.822E-07	5.969E-01	9.934E-03	
			1.310E-04	1.000E+02	1.000E+00	
		DCGL Total	7.631E+03			

GEL results for Aux Drain #3; Common to -26 Sump

SE8990420		NRC				
PM03		Screening				
7/1/2008	Normalized	Level DCGL				
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	1.346E-01	7.050E+03	1.909E-05	3.802E+01	1.346E-01	
Nb-94	1.333E-03	8.290E+03	1.608E-07	3.202E-01	1.333E-03	
Ag-108m	7.234E-04	1.020E+04	7.092E-08	1.413E-01	7.234E-04	
Sb-125	2.646E-04	4.430E+04	5.972E-09	1.190E-02	2.646E-04	
Cs-134	4.579E-04	1.270E+04	3.605E-08	7.182E-02	4.579E-04	
Cs-137	8.618E-01	2.800E+04	3.078E-05	6.131E+01	8.618E-01	
Eu-154	6.972E-04	1.150E+04	6.063E-08	1.208E-01	6.972E-04	
Eu-155	< 1.303E-04	1.570E+05	8.296E-10	1.653E-03	1.303E-04	
	1.000E+00		5.020E-05	1.000E+02	1.000E+00	
		DCGL Total	1.992E+04			

5 GEL results for Aux Drain #2; Common to -47 Sumps

SE8990420		NRC				
PM02A		Screening				
7/1/2008	Normalized	Level DCGL				
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	1.839E-01	7.050E+03	2.608E-05	4.705E+01	1.839E-01	
Nb-94	5.336E-04	8.290E+03	6.437E-08	1.161E-01	5.336E-04	
Ag-108m	< 2.299E-04	1.020E+04	2.254E-08	4.066E-02	2.299E-04	
Sb-125	< 2.990E-04	4.430E+04	6.750E-09	1.218E-02	2.990E-04	
Cs-134	2.499E-04	1.270E+04	1.967E-08	3.550E-02	2.499E-04	
Cs-137	8.116E-01	2.800E+04	2.899E-05	5.229E+01	8.116E-01	
Eu-154	2.821E-03	1.150E+04	2.453E-07	4.425E-01	2.821E-03	
Eu-155	3.657E-04	1.570E+05	2.329E-09	4.202E-03	3.657E-04	
	1.000E+00		5.543E-05	1.000E+02	1.000E+00	
		DCGL Total	1.804E+04			

SFP #3 Leak Chase #6

SB8990440		NRC				
PM01		Screening				
7/1/2008	Normalized	Level DCGL	Observable			
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	6.304E-03	7.050E+03	8.942E-07	2.323E+00	6.304E-03	
Nb-94	< 1.060E-03	8.290E+03	1.279E-07	3.323E-01	1.060E-03	
Ag-108m	3.205E-02	1.020E+04	3.142E-06	8.163E+00	3.205E-02	
Sb-125	< 6.225E-03	4.430E+04	1.405E-07	3.651E-01	6.225E-03	
Cs-134	< 9.345E-04	1.270E+04	7.359E-08	1.912E-01	9.345E-04	
Cs-137	9.480E-01	2.800E+04	3.386E-05	8.796E+01	9.480E-01	
Eu-154	< 3.113E-03	4.430E+04	7.027E-08	1.826E-01	3.113E-03	
Eu-155	< 2.357E-03	1.270E+04	1.856E-07	4.821E-01	2.357E-03	
			3.849E-05	1.000E+02	1.000E+00	
		DCGL Total	2.598E+04			

7 SFP Leak Chase #3

SB8990440		NRC				
PM02		Screening				
7/1/2008	Normalized	Level DCGL	Observable			
Nuclide	nf	dpm/100cm ²	nf/DCGL	% Dose	Beta f	
Co-60	2.628E-02	7.050E+03	3.727E-06	9.486E+00	2.628E-02	
Nb-94	< 9.061E-04	8.290E+03	1.093E-07	2.782E-01	9.061E-04	
Ag-108m	1.124E-02	1.020E+04	1.102E-06	2.805E+00	1.124E-02	
Sb-125	< 3.955E-03	4.430E+04	8.927E-08	2.272E-01	3.955E-03	
Cs-134	< 5.941E-04	1.270E+04	4.678E-08	1.191E-01	5.941E-04	
Cs-137	9.537E-01	2.800E+04	3.406E-05	8.669E+01	9.537E-01	
Eu-154	< 1.892E-03	4.430E+04	4.270E-08	1.087E-01	1.892E-03	
Eu-155	< 1.422E-03	1.270E+04	1.120E-07	2.850E-01	1.422E-03	
			3.929E-05	1.000E+02	1.000E+00	
		DCGL Total	2.545E+04			

SFP Leak Chase #1 9

SB8990440		NRC			
PM03		Screening		Observable	
7/1/2008	Normalized	Level DCGL			
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	2.822E-02	7.050E+03	4.003E-06	1.005E+01	2.822E-02
Nb-94	< 1.431E-03	8.290E+03	1.726E-07	4.334E-01	1.431E-03
Ag-108m	1.537E-02	1.020E+04	1.507E-06	3.783E+00	1.537E-02
Sb-125	< 5.487E-03	4.430E+04	1.239E-07	3.110E-01	5.487E-03
Cs-134	< 8.893E-04	1.270E+04	7.003E-08	1.758E-01	8.893E-04
Cs-137	9.433E-01	2.800E+04	3.369E-05	8.460E+01	9.433E-01
Eu-154	< 2.793E-03	4.430E+04	6.304E-08	1.583E-01	2.793E-03
Eu-155	< 2.478E-03	1.270E+04	1.951E-07	4.900E-01	2.478E-03
			3.982E-05	1.000E+02	1.000E+00
DCGL Total		2.511E+04			

SFP #2 Leak Chase 10

SB8990440		NRC			
PM04		Screening		Observable	
7/1/2008	Normalized	Level DCGL			
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	3.498E-02	7.050E+03	4.961E-06	1.221E+01	3.498E-02
Nb-94	< 1.241E-03	8.290E+03	1.497E-07	3.683E-01	1.241E-03
Ag-108m	1.705E-02	1.020E+04	1.672E-06	4.114E+00	1.705E-02
Sb-125	4.513E-03	4.430E+04	1.019E-07	2.507E-01	4.513E-03
Cs-134	< 7.088E-04	1.270E+04	5.581E-08	1.373E-01	7.088E-04
Cs-137	9.367E-01	2.800E+04	3.345E-05	8.232E+01	9.367E-01
Eu-154	< 2.445E-03	4.430E+04	5.520E-08	1.358E-01	2.445E-03
Eu-155	< 2.406E-03	1.270E+04	1.895E-07	4.662E-01	2.406E-03
			4.064E-05	1.000E+02	1.000E+00
DCGL Total		2.461E+04			

SFP Leak Chase #4 11

SB8990440		NRC			
PM05		Screening		Observable	
7/1/2008	Normalized	Level DCGL			
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	3.191E-02	7.050E+03	4.526E-06	1.075E+01	3.191E-02
Nb-94	< 2.078E-03	8.290E+03	2.507E-07	5.951E-01	2.078E-03
Ag-108m	4.427E-02	1.020E+04	4.340E-06	1.030E+01	4.427E-02
Sb-125	< 6.308E-03	4.430E+04	1.424E-07	3.380E-01	6.308E-03
Cs-134	< 1.016E-03	1.270E+04	7.999E-08	1.899E-01	1.016E-03
Cs-137	9.075E-01	2.800E+04	3.241E-05	7.695E+01	9.075E-01
Eu-154	< 3.155E-03	4.430E+04	7.123E-08	1.691E-01	3.155E-03
Eu-155	< 3.808E-03	1.270E+04	2.998E-07	7.119E-01	3.808E-03
			4.212E-05	1.000E+02	1.000E+00
DCGL Total		2.374E+04			

SFP Leak Chase #7 12

SB8990440		NRC			
PM06		Screening		Observable	
7/1/2008	Normalized	Level DCGL			
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	2.335E-02	7.050E+03	3.312E-06	8.343E+00	2.335E-02
Nb-94	< 1.483E-03	8.290E+03	1.789E-07	4.507E-01	1.483E-03
Ag-108m	1.945E-02	1.020E+04	1.907E-06	4.805E+00	1.945E-02
Sb-125	< 7.352E-04	4.430E+04	1.660E-08	4.181E-02	7.352E-04
Cs-134	< 1.240E-03	1.270E+04	9.762E-08	2.459E-01	1.240E-03
Cs-137	9.462E-01	2.800E+04	3.379E-05	8.513E+01	9.462E-01
Eu-154	< 3.665E-03	4.430E+04	8.273E-08	2.084E-01	3.665E-03
Eu-155	< 3.895E-03	1.270E+04	3.067E-07	7.727E-01	3.895E-03
			3.969E-05	1.000E+02	1.000E+00
DCGL Total		2.519E+04			

SFP Leak Chase #5 13

SB8990440		NRC			
7/1/2008	PM08	Screening	Observable		
Nuclide	Normalized nf	Level DCGL dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	8.803E-02	7.050E+03	1.249E-05	2.635E+01	8.803E-02
Nb-94	< 6.893E-03	8.290E+03	8.314E-07	1.755E+00	6.893E-03
Ag-108m	2.274E-02	1.020E+04	2.229E-06	4.705E+00	2.274E-02
Sb-125	< 1.370E-02	4.430E+04	3.093E-07	6.527E-01	1.370E-02
Cs-134	< 2.180E-03	1.270E+04	1.717E-07	3.623E-01	2.180E-03
Cs-137	8.481E-01	2.800E+04	3.029E-05	6.392E+01	8.481E-01
Eu-154	< 6.736E-03	4.430E+04	1.520E-07	3.209E-01	6.736E-03
Eu-155	< 1.163E-02	1.270E+04	9.161E-07	1.933E+00	1.163E-02
			4.739E-05	1.000E+02	1.000E+00
DCGL Total		2.110E+04			

SFP Leak Chase #8 14

SB8990440		NRC			
7/1/2008	PM07	Screening	Observable		
Nuclide	Normalized nf	Level DCGL dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	3.213E-02	7.050E+03	4.558E-06	1.130E+01	3.213E-02
Nb-94	< 1.532E-03	8.290E+03	1.848E-07	4.582E-01	1.532E-03
Ag-108m	1.623E-02	1.020E+04	1.591E-06	3.946E+00	1.623E-02
Sb-125	< 6.110E-03	4.430E+04	1.379E-07	3.420E-01	6.110E-03
Cs-134	< 9.643E-04	1.270E+04	7.593E-08	1.883E-01	9.643E-04
Cs-137	9.369E-01	2.800E+04	3.346E-05	8.299E+01	9.369E-01
Eu-154	< 2.956E-03	4.430E+04	6.673E-08	1.655E-01	2.956E-03
Eu-155	< 3.125E-03	1.270E+04	2.461E-07	6.103E-01	3.125E-03
			4.032E-05	1.000E+02	1.000E+00
DCGL Total		2.480E+04			

SFP Cooler 8" Pipe 15

SA8990440		NRC			
7/1/2008	PM01	Screening	Observable		
Nuclide	Normalized nf	Level DCGL dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	5.547E-01	7.050E+03	7.869E-05	8.143E+01	5.547E-01
Nb-94	1.592E-02	8.290E+03	1.920E-06	1.987E+00	1.592E-02
Ag-108m	7.891E-03	1.020E+04	7.736E-07	8.006E-01	7.891E-03
Sb-125	< 4.424E-03	4.430E+04	9.987E-08	1.034E-01	4.424E-03
Cs-134	< 1.237E-03	1.270E+04	9.737E-08	1.008E-01	1.237E-03
Cs-137	4.074E-01	2.800E+04	1.455E-05	1.506E+01	4.074E-01
Eu-154	< 2.749E-03	4.430E+04	6.206E-08	6.422E-02	2.749E-03
Eu-155	< 5.663E-03	1.270E+04	4.459E-07	4.614E-01	5.663E-03
			9.663E-05	1.000E+02	1.000E+00
DCGL Total		1.035E+04			

SFP Skimmer 3" Pipe 16

SA8990440		NRC			
7/1/2008	PM02	Screening	Observable		
Nuclide	Normalized nf	Level DCGL dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	1.632E-01	7.050E+03	2.315E-05	4.268E+01	1.632E-01
Nb-94	1.096E-02	8.290E+03	1.322E-06	2.437E+00	1.096E-02
Ag-108m	3.329E-03	1.020E+04	3.263E-07	6.018E-01	3.329E-03
Sb-125	3.764E-03	4.430E+04	8.497E-08	1.567E-01	3.764E-03
Cs-134	9.786E-04	1.270E+04	7.706E-08	1.421E-01	9.786E-04
Cs-137	8.151E-01	2.800E+04	2.911E-05	5.368E+01	8.151E-01
Eu-154	9.402E-04	4.430E+04	2.122E-08	3.914E-02	9.402E-04
Eu-155	< 1.764E-03	1.270E+04	1.389E-07	2.562E-01	1.764E-03
			5.423E-05	1.000E+02	1.000E+00
DCGL Total		1.844E+04			

SFP Nozzle Return 2.5" Pipe 17

SA8990440 NRC
PM03 Screening

7/1/2008	Normalized	Level DCGL	Observable		
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	6.596E-01	7.050E+03	9.357E-05	8.265E+01	6.596E-01
Nb-94 <	1.965E-02	8.290E+03	2.371E-06	2.094E+00	1.965E-02
Ag-108m	7.389E-02	1.020E+04	7.244E-06	6.398E+00	7.389E-02
Sb-125 <	1.880E-02	4.430E+04	4.244E-07	3.749E-01	1.880E-02
Cs-134 <	5.970E-03	1.270E+04	4.701E-07	4.152E-01	5.970E-03
Cs-137	1.775E-01	2.800E+04	6.339E-06	5.599E+00	1.775E-01
Eu-154 <	1.261E-02	4.430E+04	2.846E-07	2.514E-01	1.261E-02
Eu-155 <	3.195E-02	1.270E+04	2.516E-06	2.222E+00	3.195E-02
			1.132E-04	1.000E+02	1.000E+00
DCGL Total			8.833E+03		

Rx Trench 7' NE B Sump 18

SA8110351 NRC
SC01A Screening

7/1/2008	Normalized	Level DCGL	Observable		
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	1.188E-03	7.050E+03	1.685E-07	4.701E-01	1.188E-03
Nb-94 <	2.002E-04	8.290E+03	2.415E-08	6.736E-02	2.002E-04
Ag-108m <	2.456E-04	1.020E+04	2.408E-08	6.718E-02	2.456E-04
Sb-125 <	1.383E-03	4.430E+04	3.123E-08	8.711E-02	1.383E-03
Cs-134	2.536E-04	1.270E+04	1.997E-08	5.571E-02	2.536E-04
Cs-137	9.942E-01	2.800E+04	3.551E-05	9.905E+01	9.942E-01
Eu-154 <	2.300E-03	4.430E+04	5.192E-08	1.448E-01	2.300E-03
Eu-155 <	2.542E-04	1.270E+04	2.001E-08	5.583E-02	2.542E-04
			3.585E-05	1.000E+02	1.000E+00
DCGL Total		2.790E+04			

Rx Trench 9' E B Sump 19

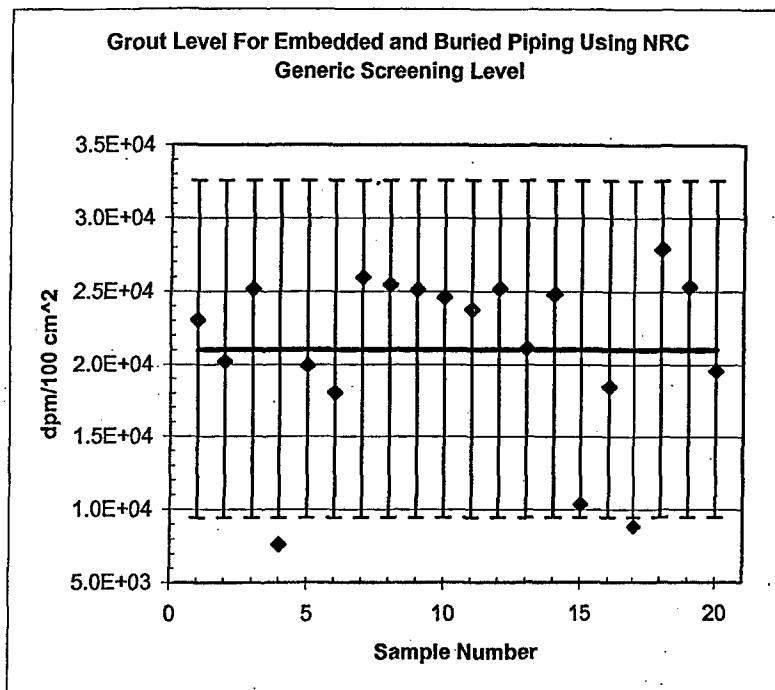
SD8110351 NRC
SC01A Screening

7/1/2008	Normalized	Level DCGL	Observable		
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	3.191E-02	7.050E+03	4.527E-06	1.146E+01	3.191E-02
Nb-94	4.189E-04	8.290E+03	5.053E-08	1.279E-01	4.189E-04
Ag-108m	5.735E-03	1.020E+04	5.622E-07	1.424E+00	5.735E-03
Sb-125	1.029E-03	4.430E+04	2.322E-08	5.880E-02	1.029E-03
Cs-134	2.084E-04	1.270E+04	1.641E-08	4.156E-02	2.084E-04
Cs-137	9.590E-01	2.800E+04	3.425E-05	8.672E+01	9.590E-01
Eu-154	1.208E-03	4.430E+04	2.728E-08	6.907E-02	1.208E-03
Eu-155	4.928E-04	1.270E+04	3.880E-08	9.824E-02	4.928E-04
			3.950E-05	1.000E+02	1.000E+00
DCGL Total		2.532E+04			

SFP 2" Drain +40 EL. 20

NRC
Screening

7/1/2008	Normalized	Level DCGL	Observable		
Nuclide	nf	dpm/100cm2	nf/DCGL	% Dose	Beta f
Co-60	1.099E-01	7.050E+03	1.559E-05	3.049E+01	1.099E-01
Nb-94 <	1.181E-02	8.290E+03	1.425E-06	2.788E+00	1.181E-02
Ag-108m <	1.394E-02	1.020E+04	1.367E-06	2.675E+00	1.394E-02
Sb-125 <	2.944E-02	4.430E+04	6.645E-07	1.300E+00	2.944E-02
Cs-134 <	6.129E-04	1.270E+04	4.826E-08	9.442E-02	6.129E-04
Cs-137	7.860E-01	2.800E+04	2.807E-05	5.492E+01	7.860E-01
Eu-154 <	3.060E-02	1.270E+04	2.409E-06	4.714E+00	3.060E-02
Eu-155 <	1.770E-02	1.150E+04	1.539E-06	3.011E+00	1.770E-02
			5.111E-05	1.000E+02	1.000E+00
DCGL Total		1.957E+04			



Grout Level Using NRC Generic Screening Level

Location Code	NRC Screening cpm/100 cm ²
1	2.306E+04
2	2.021E+04
3	2.519E+04
4	7.631E+03
5	1.992E+04
6	1.804E+04
7	2.598E+04
8	2.545E+04
9	2.511E+04
10	2.461E+04
11	2.374E+04
12	2.519E+04
13	2.110E+04
14	2.480E+04
15	1.035E+04
16	1.844E+04
17	8.833E+03
18	2.790E+04
19	2.532E+04
20	1.957E+04
mean	2.102E+04
Standard Deviation	5.900E+03
1.96σ	1.156E+04
lcl	9.458E+03
ucl	3.259E+04