Dominion Energy Kewaunee, Inc.
N490 Highway 42, Kewaunee, WI 54216-9511

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JAN 2 6 2006

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No. 06-030
KPS/LIC/RR: RO
Docket No. 50-305
License No. DPR-43

DOMINION ENERGY KEWAUNEE, INC.
KEWAUNEE POWER STATION
LICENSEE EVENT REPORT 2005-016-00

Dear Sirs:

Pursuant to 10 CFR 50.73, Dominion Energy Kewaunee, Inc., hereby submits the following Licensee Event Report applicable to Kewaunee Power Station.

Report No. 50-305/LER 2005-016-00

This report has been reviewed by the Plant Operating Review Committee and will be forwarded to the Management Safety Review Committee for its review.

If you have any further questions, please contact Mr. Rick Repshas at (920) 388-8217.

Very truly yours,

Michael G. Gaffney
Site Vice President, Kewaunee Power Station

Attachment

Commitments made by this letter: NONE

IE22

cc: Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
2443 Warrenville Road
Suite 210
Lisle, IL 60532-4352

Mr. D. H. Jaffe
Project Manager
U.S. Nuclear Regulatory Commission
Mail Stop O-7-D-1
Washington, D. C. 20555

NRC Senior Resident Inspector
Kewaunee Power Station

bc (*hard copy distribution):

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NRC FORM 366 (6-2004) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104 EXPIRES 6-30-2007

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0066), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Kewaunee Power Station	05000305	1 of 3

TITLE (4)
Automatic Reactor Trip due to Main Feedwater Pump Motor Failure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MO	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
11	28	2005	2005	-- 016 --	00	01	26	2006	FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	N
POWER LEVEL (10)	100

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR : (Check all that apply) (11)

	20.2201(b)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)(B)		50.73(a)(2)(ix)(A)
	20.2201(d)		20.2203(a)(4)		50.73(a)(2)(iii)		50.73(a)(2)(x)
	20.2203(a)(1)		50.36(c)(1)(i)(A)	X	50.73(a)(2)(iv)(A)		73.71(a)(4)
	20.2203(a)(2)(i)		50.36(c)(1)(ii)(A)		50.73(a)(2)(v)(A)		73.71(a)(5)
	20.2203(a)(2)(ii)		50.36(c)(2)		50.73(a)(2)(v)(B)		OTHER
	20.2203(a)(2)(iii)		50.46(a)(3)(ii)		50.73(a)(2)(v)(C)		Specify in Abstract below or in NRC Form 366A
	20.2203(a)(2)(iv)		50.73(a)(2)(i)(A)		50.73(a)(2)(v)(D)		
	20.2203(a)(2)(v)		50.73(a)(2)(i)(B)		50.73(a)(2)(vii)		
	20.2203(a)(2)(vi)		50.73(a)(2)(i)(C)		50.73(a)(2)(viii)(A)		
	20.2203(a)(3)(i)		50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Richard Repshas	(920) 388-8217

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
X	SJ	MO	A180	YES					

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	X NO				

ABSTRACT

On November 28, 2005 at 2219 central standard time (CST), while the plant was operating at 100 percent power with no abnormal indications, the 'B' main feedwater pump tripped due to an overcurrent condition. During the transient, 'B' steam generator reached its low-low level setpoint actuating a reactor protection system reactor trip at 2200 CST. All systems responded as designed. Subsequent inspection of the 'B' main feedwater pump identified a short to ground in all three phases of the motor.

This event was reported as a non-emergency in accordance with 10 CFR 50.72(b)(2)(iv)(B) for actuation of the reactor protection system when the reactor is critical and 10 CFR 50.72(b)(3)(iv)(A) for valid actuation of the auxiliary feedwater system.

Since all safety systems operated as expected, this event is considered to have negligible safety significance. This is not considered a safety system functional failure.

The 'B' main feedwater pump motor was replaced and the reactor startup was completed on November 29, 2005 at 2236 CST with the plant output breaker being closed on November 30, 2005 at 0335 CST. Full power was reached on December 2, 2005 at 1938 CST.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Kewaunee Power Station	05000305	2005	-- 016 --	00	2 of 3

TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

Event Description:

On November 28, 2005 at 2219 central standard time (CST), while the plant was operating at 100 percent power with no abnormal indications, the 'B' main feedwater pump [P][SJ] tripped due to an overcurrent condition. During the transient, the plant responded by initiating an automatic turbine [TRB][TA] runback and automatically increasing the opening of the main feedwater regulating valves [V][SJ] in an attempt to restore steam generator [SG] level. As 'B' steam generator level continued to decrease, operators took manual action to control feedwater to restore steam generator level. 'B' steam generator level continued to decrease and reached its low-low level setpoint actuating a reactor protection system reactor trip [RCT] at 2200 CST. The two motor-driven auxiliary feedwater pumps [P][BA] and the turbine-driven auxiliary feedwater pump [P][BA] started as expected since water level in both steam generators reached their low-low steam generator water level of 17%. All systems responded as designed. Subsequent inspection of the 'B' main feedwater pump identified a short to ground in all three phases of the motor [MO].

This event was reported in accordance with 10 CFR 50.72(b)(2)(iv)(B) for actuation of the reactor protection system when the reactor is critical and 10 CFR 50.72(b)(3)(iv)(A) for valid actuation of the auxiliary feedwater system.

The 'B' main feedwater pump motor had been in service since 1998 when it was last reconditioned.

The 'B' main feedwater pump motor was replaced and the reactor startup was completed on November 29, 2005 at 2236 CST with the plant output breaker [BKR] being closed on November 30, 2005 at 0335 CST. Full power was reached on December 2, 2005 at 1938 CST.

Event Analysis and Safety Significance:

This condition is being reported in accordance with 10 CFR 50.73(a)(2)(iv)(A) for any event or condition that resulted in manual or automatic action of "(B)(1), reactor protection system (RPS) including: reactor scram or reactor trip" and "(B)(6), PWR auxiliary or emergency feedwater system."

Since all safety systems operated as expected, this event is considered to have negligible safety significance.

This is not considered a safety system functional failure.

Cause:

The cause of the main feedwater pump motor fault is a stator winding to ground short due to degraded insulation between the winding coil and the coil slot caused by normal thermal aging wear over time and expected cyclic stresses such as electromagnetic forces and thermal cycling.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Kewaunee Power Station	05000305	2005	-- 016 --	00	3 of 3

TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

Corrective Actions:

1. 'B' main feedwater pump motor was replaced with a spare motor.

2. The station is reviewing the event to determine if changes to the large electric motor maintenance program are required.

Similar Events:

LER 93-001, Phase-to-Phase Fault in Main Feedwater Pump Causes Bus 1 and 2 Undervoltage Condition Resulting in Automatic Reactor Trip.