

March 10, 2006

Mr. Gordon Bischoff, Manager  
Owners Group Program Management Office  
Westinghouse Electric Company  
P.O. Box 355  
Pittsburgh, PA 15230-0355

SUBJECT: FINAL SAFETY EVALUATION FOR WESTINGHOUSE OWNERS GROUP (WOG) TOPICAL REPORT WCAP-15791-P, REVISION 1, "RISK-INFORMED EVALUATION OF EXTENSIONS TO CONTAINMENT ISOLATION VALVE COMPLETION TIMES" (TAC NO. MB5751)

Dear Mr. Bischoff:

By letter dated June 6, 2002 (OG-02-022), as supplemented by letters dated February 13 and May 6, 2004, and March 10, 2005 (WOG-04-077, WOG-04-234, and WOG-05-119, respectively), the WOG submitted Topical Report (TR) WCAP-15791-P, "Risk-Informed Evaluation of Extensions to Containment Isolation Valve Completion Times," Revision 1, to the U.S. Nuclear Regulatory Commission (NRC) for review and approval. This TR would support licensee requests for changes to their technical specification (TS) completion times (CTs) for primary containment isolation valves (CIVs). The letters dated February 13, 2004, and March 10, 2005, provided responses to the NRC staff's request for additional information (RAI). The letter dated May 6, 2004, provided WCAP-15791-P, Revision 1, that incorporated changes delineated in the RAI responses.

On September 16, 2005, the NRC issued its draft Safety Evaluation (SE) regarding our approval of WCAP-15791-P, Revision 1. By letter dated October 19, 2005 (WOG-05-438), the WOG provided its comments on the WCAP-15791-P draft SE. The NRC staff has reviewed the comments and incorporated them in the final SE, as described in Attachment 2 to the SE. By a communication (e-mail) with the NRC staff, the WOG stated that the draft SE did not contain proprietary information. A telephone call was held with the WOG on January 27, 2006, to discuss the resolution of its comments.

Enclosed is the final SE for WCAP-15791-P, Revision 1. The NRC staff has found that WCAP-15791-P, Revision 1, is acceptable for referencing in licensing applications regarding extended CIV CTs. The enclosed SE contains conditions on licensees adopting this TR and identifies additional information needed to be submitted in plant-specific applications adopting the TR in Sections 3.5 and 3.6, respectively, of the enclosed SE.

Based on its review of WCAP-15791-P, Revision 1, the NRC staff concludes that the TR provides guidance, and generic and plant-specific analyses, to assist licensees in evaluating changes to CIV CTs. The NRC staff finds the guidance included in the TR to be complementary to staff guidance provided in Regulatory Guides (RGs) 1.174 and 1.177, and in Chapter 19.0 of NUREG-0800. As such, WCAP-15791-P, Revision 1, provides an acceptable basis to evaluate the proposed CIV CTs, when used in conjunction with the guidance provided by RGs 1.174 and 1.177.

In accordance with the guidance provided on the NRC website, we request that the WOG publish an accepted version of this TR within three months of receipt of this letter. The accepted version shall incorporate this letter and the enclosed SE between the title page and the abstract. It must be well indexed such that information is readily located. Also, it must contain historical review information, such as NRC staff questions and accepted responses, draft SE comments, and original TR pages that were replaced. The accepted version shall include a "- A" (designating accepted) following the TR identification symbol.

If future changes to the NRC's regulatory requirements affect the acceptability of this TR, the WOG and/or licensees referencing it will be expected to revise the TR appropriately, or justify its continued applicability for subsequent referencing.

In support of WCAP-15791-P, Revision 1, the Technical Specification Task Force (TSTF) submitted TSTF-446, Revision 1, "Risk-Informed Evaluation of Extensions to Containment Isolation Valve Completion Times (WCAP-15791)," by letter dated January 31, 2005, to the NRC. Although the TSTF is not addressed in the enclosed SE, it is referred to in Sections 3.1 and 3.3.3 of the SE because the WOG referenced the TSTF in its response to an NRC RAI. The acceptability of the technical specifications in the proposed TSTF will be addressed in a separate evaluation.

Sincerely,

/RA/  
Ho K. Nieh, Deputy Director  
Division of Policy and Rulemaking  
Office of Nuclear Reactor Regulation

Project No. 694

Enclosure: Final Safety Evaluation

cc w/encl:  
Mr. James A. Gresham, Manager  
Regulatory Compliance and Plant Licensing  
Westinghouse Electric Company  
P.O. Box 355  
Pittsburgh, PA 15230-0355

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
WCAP-15791-P, REVISION 1, "RISK-INFORMED EVALUATION OF EXTENSIONS TO  
CONTAINMENT ISOLATION VALVE COMPLETION TIMES"  
WESTINGHOUSE OWNERS GROUP  
PROJECT NO. 694

## 1.0 INTRODUCTION

Licensees of nuclear power plants have Technical Specifications (TSs) in accordance with Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, "Technical specifications," that govern the operation of the plants. These TSs have limiting conditions for operation (LCOs) that state the primary containment isolation valves (CIVs) must be operable and the applicable reactor modes of operation in which CIVs are required to be operable. If any of the CIVs are inoperable, the TSs specify the required actions to address the inoperability and the completion times (CTs). The U.S. Nuclear Regulatory Commission (NRC) improved standard TSs (ISTS) for Westinghouse plants are in NUREG-1431, "Standard Technical Specifications Westinghouse Plants," Revision 3, dated June 2004 (NUREG-1431).

By letter dated June 6, 2002 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML021720004) as supplemented by letters dated February 13 and May 6, 2004, and March 10, 2005 (ADAMS Accession Nos. ML052010500, ML051940476, and ML050740020, respectively), the Westinghouse Owners Group (WOG) submitted proprietary and non-proprietary versions of topical report (TR) WCAP-15791-P, "Risk-Informed Evaluation of Extensions to Containment Isolation Valve Completion Times," for NRC staff review. The WOG letters dated February 13, 2004, and March 10, 2005, provided responses to the NRC staff's request for additional information (RAI) and other clarifications. The supplemental letter dated May 6, 2004, provided the proprietary and non-proprietary versions of WCAP-15791, Revision 1 (WCAP-15791) that incorporated changes delineated in the WOG RAI responses. It is this version of WCAP-15791 that is addressed in this safety evaluation (SE). The TR provides technical justification for extending CIV CTs in ISTS LCO 3.6.3, "Containment Isolation Valves," and would be referenced in plant-specific license amendment requests to extend CIV CTs. The WOG also provided comments on the NRC staff's draft SE in its letter dated October 19, 2005 (ADAMS Accession No. ML052940248).

The TR provides a risk-informed justification for extending the CIV CTs from 4 hours to 168 hours (i.e., 7 days). The approach taken in WCAP-15791 in grouping plant CIVs and determining the CTs for the CIVs is addressed in Appendix A to this SE.

For CIVs that did not demonstrate acceptable results for 168 hours, shorter CTs were evaluated in WCAP-15791. The WOG analysis includes a generic-bounding risk assessment of the impact of adopting the proposed CTs. A deterministic approach was used to determine the minimum-containment hole size that would result in a large release from the containment

atmosphere. Penetration flow paths connected to the containment atmosphere smaller than the minimum hole size are screened out of the total list of penetration flow paths (i.e., no further evaluation is made), and are assigned the maximum CT of 7 days. Penetrations larger than the minimum-containment hole size were evaluated using a probabilistic evaluation to verify what CT (i.e., a 7-day or shorter CT) is justified by the evaluation.

The WOG states that the CIV CT extension methodology in WCAP-15791 is consistent with the guidance of Regulatory Guide (RG) 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis," Revision 1, dated November 2002, and RG 1.177, "An Approach for Plant-Specific, Risk-Informed Decisionmaking: Technical Specifications," dated August 1998. However, to be within these guidelines, the 7-day CT had to be reduced for some CIVs. Thus, plant-specific applications of the proposed generic results will lead to some CIV CTs that are less than 7 days.

In support of WCAP-15791, the Nuclear Energy Institute (NEI) submitted TSTF-446, Revision 1, "Risk-Informed Evaluation of Extensions to Containment Isolation Valve Completion Times (WCAP-15791)," by letter dated January 31, 2005 (ADAMS Accession No. ML050460293), to the NRC. Although the TSTF is not addressed in the SE, it is referred to in Sections 3.1 and 3.3.3 of the SE because the WOG referenced the TSTF in its response to an NRC RAI. The acceptability of the technical specifications in the proposed TSTF will be addressed in a separate evaluation.

The WOG stated in WCAP-15791 that the proposed CT extensions will provide flexibility by increasing the time to perform on-line CIV testing, maintenance, or repair. The proposed CTs were stated to provide sufficient time for plant personnel to both address CIV inoperability and to perform preventive maintenance activities on the CIVs during power operation.

## 2.0 REGULATORY EVALUATION

CIVs ensure that adequate primary containment boundaries are maintained during and after accidents by minimizing potential flow paths to the environment and ensure that the primary containment function assumed in the plant-specific safety analysis is maintained. Two barriers (one may be a closed system) in a series are provided for each penetration so that no credible single failure of an active component can result in a loss of isolation or leakage that exceeds the limits assumed in the safety analysis. The associated CIV LCO in the plant TSs ensures that the CIVs will perform their design safety functions to minimize the loss of reactor coolant inventory and establish an adequate containment boundary during an accident.

NUREG-1431 states that CIVs form part of the containment pressure boundary and provide a means for fluid penetrations not serving accident consequence limiting systems to be provided with two isolation barriers that are closed on a containment isolation signal. These isolation devices are either passive or active (i.e., automatic). Manual valves, deactivated automatic valves secured in their closed position (including check valves with flow through the valve secured), blind flanges, and closed systems are considered passive devices. Two barriers in a series are provided for each penetration so that no single credible failure or malfunction of an active component can result in a loss of isolation or leakage that exceeds limits assumed in the safety analysis. CIVs help ensure that the containment atmosphere will be isolated from the environment in the event of a release of fission product radioactivity to the containment

atmosphere as a result of a design-basis accident (DBA). The DBAs that result in a release of radioactive material within containment are a loss-of-coolant accident (LOCA) and a rod ejection accident. The operability requirements for CIVs help ensure that containment is isolated within the time limits assumed in the safety analysis.

## 2.1 Applicable Regulations

The applicable regulations governing CIVs are the following:

Technical Specifications, 10 CFR 50.36, requires that all operating licenses for nuclear reactors must include TSs for the subject plant. The LCOs, along with the required CTs, are specified for each system in the TSs, which includes the CIVs. With the LCOs, there are surveillance requirements specified to check that the system LCO is being met, and conditions, required actions, and CTs specified for when the LCO is not being met and how long the plant can take to restore the LCO or shut down. Although CTs are not specifically stated in 50.36, LCOs are addressed and 50.36(c)(2) states that when a LCO is not met, the licensee shall "shut down the reactor or follow any remedial action permitted by the technical specifications until the condition can be met." The action conditions and required actions in the technical specifications are the remedial actions and the CTs are allowed time for the specified remedial actions before the licensee shall shut down the reactor. If the basis for extending the CTs is acceptable, then the requirements of 10 CFR 50.36 are met. The basis for the CTs specified in the TSs can be deterministic and/or risk-informed.

The Maintenance Rule, 10 CFR 50.65(a)(4), as it relates to the proposed CIV CT configuration, requires the assessment and management of the increase in risk that may result from the proposed maintenance activity.

General Design Criterion (GDC 35), "Emergency core cooling" of Appendix A to 10 CFR Part 50, requires suitable redundancy in components and features, and suitable interconnections, leak detection, isolation, and containment capabilities to assure that the system safety function can be accomplished assuming a single failure.

GDC 54, "Piping systems penetrating containment," requires those piping systems penetrating primary containment shall be provided with leak detection, isolation, containment capabilities having redundancy, reliability, and performance capabilities which reflect the importance to safety of isolating these piping systems.

GDC 55, "Reactor coolant pressure boundary penetrating containment," requires that each line that is part of the reactor coolant pressure boundary and that penetrates primary containment shall be provided with CIVs.

GDC 56, "Primary containment isolation," requires that each line that connects directly to the containment atmosphere and penetrates primary reactor containment shall be provided with CIVs.

GDC 57, "Closed system isolation valves," requires that each line that penetrates primary reactor containment and is neither part of the reactor coolant pressure boundary nor connected directly to the containment atmosphere shall have at least one CIV which shall be either automatic, or locked closed, or capable of remote manual operation.

The proposed CIV CTs do not affect the design or function of these valves; therefore, compliance with the above GDC is not changed by the proposed CTs. Also, if the basis for extending the CTs is acceptable, then 10 CFR 50.36 will be met. The basis in WCAP-15791 for extending the CIV CTs is risk-informed and the criteria for accepting changes to plants utilizing risk information is discussed in the next section.

## 2.2 Applicable Regulatory Criteria/Guidelines

General guidance for evaluating the technical basis of proposed risk-informed changes is provided in Chapter 19.0 of the NRC Standard Review Plan (SRP), NUREG-0800. More specific guidance related to risk-informed TS changes is provided in SRP Section 16.1, "Risk-Informed Decisionmaking: Technical Specifications," which includes CT changes as part of risk-informed decisionmaking. Chapter 19.0 of the SRP states that a risk-informed application should be evaluated to ensure that the proposed changes meet the following key principles:

- The proposed change meets the current regulations, unless it explicitly relates to a requested exemption or rule change.
- The proposed change is consistent with the defense-in-depth philosophy.
- The proposed change maintains sufficient safety margins.
- When proposed changes increase risk (i.e., core damage frequency (CDF) or large early release frequency (LERF)), the increase(s) should be small and consistent with the intent of the Commission's Safety Goal Policy Statement.
- The impact of the proposed change should be monitored using performance measurement strategies.

RG 1.174 and RG 1.177 provide specific guidance and acceptance guidelines for assessing the nature and impact of licensing-basis changes, including proposed permanent TS changes in CTs by considering engineering issues and applying risk insights. RG 1.177 identifies an acceptable risk-informed approach, including additional guidance specifically geared toward the assessment of proposed TS CT changes. Specifically, RG 1.177 identifies a three-tiered approach for the evaluation of the risk associated with a proposed TS CT change as identified below:

- Tier 1 is an evaluation of the plant-specific risk associated with the proposed TS change, as shown by the change in core damage frequency ( $\Delta$ CDF) and incremental conditional core damage probability (ICCDP), change in large early release frequency ( $\Delta$ LERF), and incremental conditional large early release probability (ICLERP). This tier also addresses the technical adequacy of the licensee's plant-specific probabilistic risk assessment (PRA) for the subject application.
- Tier 2 identifies and evaluates, with respect to defense-in-depth, any potential risk-significant plant equipment outage configurations associated with the proposed change. The licensee should provide reasonable assurance that the risk-significant plant

equipment outage configurations will not occur when equipment associated with the proposed TS change is out of service.

- Tier 3 provides for the establishment of an overall configuration risk management program (CRMP) and confirmation that its insights are incorporated into the decisionmaking process before taking equipment out of service prior to or during the CT. Compared with Tier 2, Tier 3 provides additional coverage to ensure risk-significant plant equipment outage configurations are identified in a timely manner and that the risk impact of out-of-service equipment during planned and un-planned maintenance activities is appropriately evaluated prior to performing any maintenance activity over extended periods of plant operation. Tier 3 guidance can be satisfied by the Maintenance Rule (i.e., 10 CFR 50.65(a)(4)), where that program provides an adequate quality basis which requires a licensee to assess and manage the increase in risk that may result from activities such as surveillance, post-maintenance testing, and corrective and preventive maintenance.

If the approach in WCAP-15791 for the evaluation of the risk associated with the proposed CIV CTs addresses Tier 1, Tier 2, and Tier 3 requirements, as described above, and meets the specific guidance and acceptance guidelines in RGs 1.174 and 1.177, for assessing the nature and impact of licensing-basis changes, then the proposed CIV CTs meet 10 CFR 50.36 and are, therefore, acceptable. For WCAP-15791, Tier 1 and Tier 2 are addressed in the TR. Tier 3 is not addressed in the TR, and, therefore, must be addressed in the plant-specific applications. Sections 3.5 and 3.6 of this SE address the conditions and additional information, including Tier 3, that is needed to be submitted by licensees in their plant-specific applications.

### 3.0 TECHNICAL EVALUATION

#### 3.1 NUREG-1431 Technical Specifications Affected

Based on WCAP-15791, the following requirements in ISTS LCO 3.6.3 are affected:

- Condition A - One or more penetration flow paths with one CIV inoperable (only applicable to penetration flow paths with two [or more] CIVs). Required Action A1: Isolate the affected penetration flow path by the use of at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured. In WCAP-15791, this condition was split into two conditions, Condition A for CIV pressure boundary intact and Condition B for CIV pressure boundary not intact. This creates TS conditions to address maintenance activities that impact the CIV and the penetration pressure boundary: one when the pressure boundary is intact and one when it is not intact.
- Condition B - One or more penetration flow paths with two [or more] CIVs inoperable, where the required action is to isolate the affected flow path by a closed and deactivated automatic valve, closed manual valve, or blind flange in a CT of 1 hour, is not being changed, but this condition is renumbered.
- Condition C - One or more penetration flow paths with one CIV inoperable (only applicable to penetration flow paths with only one CIV and a closed system). Required Action C1: Isolate the affected penetration flow path by use of at least one closed and

deactivated automatic valve, closed manual valve, or blind flange. In WCAP-15791, Condition C is deleted to eliminate this condition.

- In WCAP-15791, an additional ISTS LCO 3.6.3 Condition D is proposed to be added stating that for two or more penetration flow paths with one CIV inoperable [for reasons other than Condition[s] E [and F]] the CT is 4 hours. This new condition limits the CTs of inoperable CIVs in more than one penetration flow path as allowed by Note 2 to the Technical Specifications 3.6.3 Actions table. Condition D was addressed in the WOG's response to the NRC staff's RAI 6, in its February 13, 2004, letter in that the WOG stated that the technical specifications in TSTF-446 would be revised to be consistent with the single inoperable CIV assumed in WCAP-15791. The technical specification revision is the proposed additional ISTS LCO 3.6.3 Condition in TSTF-446, Revision 1. The licensee for Wolf Creek Generating Station also submitted this condition in its plant-specific amendment request dated July 23, 2004, to adopt WCAP-15791, Revision 1. This is discussed in Section 3.3.3 of this SE.
- Also, other ISTS LCO 3.6.3 conditions are renumbered to account for deleting Condition C and adding new Conditions B and D, which are discussed above. For example, the existing Condition B is renumbered Condition C.

WCAP-15791 provides justification for extending the CT from 4 hours to up to 168 hours (7 days). For isolation valves that cannot demonstrate acceptable results for 168 hours, shorter times are considered in the WCAP, as shown for LCO 3.6.3 below:

- Condition A - One or more penetration flow paths with one CIV inoperable, and CIV pressure boundary intact.

Required Action A.1: Isolate the affected penetration flow path by the use of at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check the valve with flow through the valve secured.

Change the CT from 4 hours to one of the following seven categories, which are listed in Tables D-1 and D-2 of Appendix D to WCAP-15791, of CIVs:

1. 4 hours for Category 1 CIVs
2. 8 hours for Category 2 CIVs
3. 12 hours for Category 3 CIVs
4. 24 hours for Category 4 CIVs
5. 48 hours for Category 5 CIVs
6. 72 hours for Category 6 CIVs
7. 168 hours for Category 7 CIVs

- Condition B - One or more penetration flow paths with one CIV inoperable and CIV pressure boundary not intact.

Required Action B.1: Isolate the affected penetration flow path by use of at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

Change the CT from 4 hours to one of the following seven categories, which are listed in Tables D-1 and D-2 of Appendix D to WCAP-15791, of CIVs:

1. 4 hours for Category 8 CIVs
2. 8 hours for Category 9 CIVs
3. 12 hours for Category 10 CIVs
4. 24 hours for Category 11 CIVs
5. 48 hours for Category 12 CIVs
6. 72 hours for Category 13 CIVs
7. 168 hours for Category 14 CIVs

As shown above, NUREG-1431 LCO 3.6.3 conditions and notes, which distinguish between penetration flow paths that contain two or more CIVs and penetration flow paths that contain one CIV and a closed system, are eliminated. LCO 3.6.3 conditions are added to address maintenance activities that impact the CIV and penetration pressure boundary: one condition when the pressure boundary is intact and one condition when it is not intact. The difference between Conditions A and B above is whether (1) the CIV is not removed for maintenance and the penetration pressure boundary is intact (Condition A) or (2) the CIV is removed for maintenance and the penetration pressure boundary is not intact (Condition B). This is addressed in Section 3.2 of this SE.

Of the conditions identified in LCO 3.6.3, the risk impact of two CIVs inoperable in one or more penetration flow paths was not evaluated in WCAP-15791. The CT for this configuration is generally limited by the ISTS LCO 3.6.3 Condition B to a CT of 1 hour. This remains unchanged by WCAP-15791 in that the TR does not propose to change the condition, or the CT for this condition. Systems used for accident mitigation that contain CIVs that also function as containment pressure boundaries were evaluated only with regard to the valve impact on loss of containment isolation, and CT limitations with respect to accident mitigation system function remain unchanged. In response to the NRC staff's RAI, the WOG evaluated the potential impact of the CT extensions on the availability of other mitigative functions and the corresponding impact on risk. The WOG results show that this impact is very small.

### 3.2 Detailed Description of the Proposed Changes

The emergency core cooling system (ECCS) is designed to meet the requirements of 10 CFR 50.46, and GDC 35 of Appendix A to 10 CFR 50. Suitable redundancy in components and features, and suitable interconnections, leak detection, isolation, and containment capabilities are provided to assure that the system safety function can be accomplished assuming a single failure. The unavailability of one ECCS train, in addition to one of the injection lines affected by the assumed break, will not compromise the ability of the ECCS to mitigate a LOCA. Thus, with the inoperability of a single ECCS isolation valve to open, the remaining ECCS train is sufficient to perform the design function of ECCS for mitigating a design-basis LOCA. The WOG also confirmed by an RAI response that the ECCS does not contain valves classified as CIVs that would close on a containment isolation signal that would compromise the safety function of the mitigation system. Therefore, the safety function of the ECCS will not be affected by the proposed changes of CIV CTs with respect to a CIV failing to open.

The TR assessment of the risk impact for the proposed changes to extend CIV CTs for up to 7 days during Modes 1, 2, 3, and 4 follows the guidance of RGs 1.177 and 1.174, and includes

the evaluation of the  $\Delta$ CDF,  $\Delta$ LERF, ICCDP, and ICLERP for valves in maintenance where the pressure boundary is or is not maintained during the proposed CT in accordance with the guidance of RGs 1.174 and 1.177. The TR evaluation also includes the interfacing system LOCA (ISLOCA) for valves connected to the reactor coolant system (RCS). The TR evaluated valves that have a dual function of containment isolation and accident consequence mitigation. The WOG considered the risk impact of CIVs installed in systems with non-seismically qualified piping. Although TS LCO 3.6.3 Note 2 allows separate condition entry for each penetration flow path, proposed Condition D addresses an inoperable CIV in more than one penetration flow path and limits the CT, for all but one inoperable CIV, to four hours. This is to say that the proposed Condition D will limit the number of CIVs in an extended CT to no more than one at any given time.

The TR uses a methodology to assess plant risk that involves the grouping of CIVs and the associated penetrations in generic classes. This is addressed in Appendix A of this SE. Each class was then further divided into subgroups of generic configurations. Although the WOG did perform a plant-specific CT risk evaluation for one plant in Chapter 10 of the TR, where Wolf Creek is the lead plant, it primarily selected risk parameters identified as bounding. The risk parameters selected represent a composite plant and are considered bounding values based on data from WOG-member utilities. The risk impact of each configuration was determined by applying the proposed 7-day CT and using the bounding-risk parameters for each LCO. However, for penetration flow paths that do not result in a large early release (i.e., diameters less than or equal to 2 inches) the screening criteria presented in WCAP-15791 provides a default CT of 7 days in lieu of a risk analysis for these CIVs.

The evaluations determined the risk impact on LERF, and ICLERP with one CIV inoperable within a penetration for the 7-day CT. The resulting value represents the risk increase while in a 7-day CIV CT. These estimates were then compared to the acceptance guidelines given in RGs 1.174 and 1.177. For CIV evaluations that met the acceptance guidelines for LERF and ICLERP, a 7-day CT was proposed. For CIVs where the 7-day CT resulted in LERF and/or ICLERP estimates greater than the acceptance guidelines, alternate shorter CTs were evaluated and proposed. Based on the TR evaluation results, not all CIV CTs could be extended to the full 7-day CT; however, a basis for calculating a shorter acceptable CIV CT was presented in Chapter 8 of the TR. Plant-specific applications of the generic analysis in Chapter 8 of WCAP-15791 must show that these TR assumptions are applicable to their facility. The WOG provided a demonstration of the methodology for the Wolf Creek Generating Station in Chapter 9.

The TR stated that because CIVs are used to maintain containment integrity, any change in their availability will directly impact releases from containment following a core damage event. Furthermore, the TR stated that the impact on CDF, as expressed by  $\Delta$ CDF and ICCDP, is not relevant, since containment isolation is not directly related to the prevention or mitigation of core damage. In the RAIs, the NRC staff requested the following:

1. An evaluation of the impact on  $\Delta$ CDF/ICCDP for the TR containment isolation configurations and systems associated with an accident mitigation function (e.g., engineered safety feature actuation system, sample lines, letdown, containment cooling, reactor coolant system inventory control, or containment sprays).

2. Further evaluation of an open system (i.e., an open system inside containment is directly connected to the containment atmosphere and an open system outside containment is directly connected to the outside atmosphere) on CDF during maintenance activities and the ICCDP associated with CIVs that also have a safety function in addition to primary containment isolation that is in the closed position during maintenance.

In response to item 1 above, the WOG stated that systems that are used for accident mitigation that also contain valves that perform a containment isolation function may impact CDF. The WOG provided examples of systems that perform a dual function where the CIV was inoperable and would impact the operability of another system. The impact on operability is a function of the CT of the impacted system. In other words, the CT of the impacted system and the CT of the CIV would be controlled by the shorter of the impacted CTs. The WOG stated that, in all cases, the limiting CT was equal to or less than the CT for the associated mitigation system with a core damage or containment release mitigation function.

With an increase in the CIV CT, CDF is also impacted. The CDF impact depends on the inoperable CIV, its position, and the associated mitigation system function impacted by the inoperable CT. Again, the WOG noted that for this configuration the CT of the CIV would be limited by the mitigation system CT, which can be greater than the current CT of 4 hours. To address the proposed extended CIV CT, the WOG performed a bounding assessment of the CDF impact on ECCS (as an example) with an increased CIV CT. The WOG considered both preventive and corrective maintenance activities (common cause included). The WOG results indicated that the extended CIV CT impacts on CDF are within the acceptance guidelines given in RG 1.174.

For item 2 above, the NRC staff's RAI requested an evaluation of the impact of an open system on CDF, where the CIV has been removed during preventive or corrective maintenance. With a CIV removed for maintenance, the pressure boundary is not maintained and the impacted system is also inoperable (i.e., the CIV pressure boundary not intact). With a CIV in maintenance but still in place, the pressure boundary may remain intact and the impacted system may remain operable (i.e., the CIV pressure boundary remains intact). The TR evaluated CIVs in open systems with respect to LERF and ICLERP but did not evaluate the impact on CDF. The WOG evaluated two CIV configurations with the first being CIVs connected to the containment atmosphere. With these systems, a CIV that is open and inoperable or removed for maintenance cannot isolate the containment penetration. For CIVs in the line but open and inoperable, the associated mitigation system may or may not be operable depending on the required CIV position requirements. If the CIV is associated with a mitigation system then the inoperable CIV and/or ability and time frame necessary to restore the isolation valve to operability may impact CDF. As stated above, the shorter of the CTs for the CIV and associated inoperable system would be applied.

This second evaluation included CIVs connected to the RCS. An open and inoperable CIV impacts the frequency of an ISLOCA that bypasses containment and a core damage event would result in a large release. The TR evaluation based the CTs for these CIVs on the LERF/ICLERP acceptance guidelines of RGs 1.174 and 1.177. The WOG stated that values of LERF and ICLERP were used to determine CTs based on the assumption that the impact on LERF and ICLERP would be the limiting metrics over the CDF metric, and was considered for this configuration of CIVs in the TR.

However, CDF may be the limiting metric for a LOCA inside containment where the CIV has been removed for maintenance. In this configuration of CIVs, the CDF is the concern since the penetration has only one CIV to maintain the RCS pressure boundary. The WOG also evaluated this configuration and found that the ICCDP met the acceptance guidelines of RG 1.177. The impact on CDF/ICCDP was evaluated and the results show that the  $\Delta$ CDF estimates are within the acceptance guidelines of RG 1.174.

The WOG also addressed the ICCDP for CIVs that have an additional safety function that is in the closed position during maintenance. As with the other CIV configurations where CIVs are important to other safety systems, the CT of the impacted systems should also be evaluated. The WOG stated that the shorter of either the CIV or system CT will be applicable. The ICCDP will be equivalent to the limiting system CT estimate.

The TR evaluation uses plant-specific data from WOG plants to demonstrate a bounding methodology that would be applicable to licensees that confirm the bounding assumptions are applicable to their plants. The WOG selected the most limiting value for each input parameter (including valve type) from a plant-to-plant comparison of the WOG plants. The parameters used by the WOG in the calculation of LERF and ICLERP are included in WCAP-15791, Table 8-1. As stated above, the TR grouped CIVs by class and their associated penetration groups based on the type of containment penetration flow path. These assumptions will be assessed by the NRC staff to assure that the TR is applicable for each plant-specific application.

The CIV flow paths that were evaluated in WCAP-15791 are the penetration configuration types (i.e., Class and Group) that are listed in Appendix A to this SE. The TR includes general assumptions in estimating the risk impact for the proposed CIV CT extensions as listed below:

- Only one CIV is in maintenance with an extended CT at any time.
- Maintenance on a valve can be performed such that either the valve is intact and capable of maintaining its pressure boundary function or the valve is not intact and is not capable of maintaining the pressure boundary.
- Before maintenance or corrective maintenance (repair) is performed on a CIV, the TR evaluation assumes that the other CIV(s) in the penetration flow path have been checked to ensure they are in their proper position.
- For penetrations with two or more CIVs of the same valve type, common cause failures (CCFs) are included in the TR evaluation. CIVs of the same type are not differentiated by manufacturer. For CIVs of different valve types, CCFs are not included in the TR evaluation.
- For penetrations with diverse types of CIVs, the TR evaluation was simplified to assume that all CIVs were the same type. Plant-specific applications of the generic analysis and, if used, plant-specific analysis are to be based on the CIV within the penetration with the highest failure rate. Common cause is included when the CIVs in the penetration are the same type (See the previous bullet).
- Multiple systems are not expected to be out of service simultaneously during the extended CTs.

- A deterministic evaluation was used to establish the containment hole size and associated pipe diameter screening threshold value for a large release. The evaluation determined that any CIV in a penetration not connected to the RCS or steam generators (SGs) that has a hole size less than the threshold value would default to a CT of 7 days. Based on NRC staff questions concerning the WOG alternate large release criteria and the WOG response to the NRC staff's RAI, a 2-inch containment hole size is used as the screening threshold for a large release for all three containment types (i.e., sub-atmospheric, ice-condenser, and large dry).

Several studies including NUREG/CR-4330, "Review of Light-Water Reactor Regulatory Requirements," NUREG-1493, "Performance-Based Containment Leak-Test Program," NUREG/CR-6418, "Risk Importance of Containment and Related ESF System Performance Requirements," and NUREG-1765, "Basis Document for Large Early Release Frequency (LERF) Significance Determination Process (SDP)," have been performed to determine the risk significance of various levels of containment leakage. For example, a containment leakage rate of about 100 percent volume per day is approximately equivalent to a hole diameter of 2.5 to 3 inches for a pressurized-water reactor (PWR) large dry containment and 2 inches for a PWR ice condenser containment and is the threshold after which a release may become significant to a LERF.

- Failures (including failure to close on demand, and failure during the CT) for different valve types were evaluated. The TR selected the maximum value for each parameter within each valve type.
- Not all penetration configurations/maintenance situations may be applicable to all plants. Each licensee will determine the applicability of the proposed CTs for their plant following the approach used in Chapter 9 of the TR.
- Pipe failures, not related to a seismic event, were assumed to occur randomly. The frequency of a pipe break was selected based on a review of WCAP-14572-NP-A, Revision 1, "Westinghouse Owners Group Application of Risk-Informed Methods to Piping Inservice Inspection Topical Report." The WOG stated that the largest failure probability was selected for the TR evaluation.
- Non-seismically qualified piping was assumed to fail with a probability of one, given a seismic event.
- The WOG states that because containment isolation is a function that impacts containment response to an event and not the ability of the plant design to prevent or mitigate core damage, the impact on average CDF and ICCDP due to increased CIV unavailability was not evaluated in the TR. However, for CIVs installed in systems associated with accident mitigation, the WOG provided additional evaluations through responses to the NRC staff's RAI.
- Additional class-specific assumptions are stated for each CIV "Class" in WCAP-15791.

Plant-specific applications will need to discuss whether and how the above assumptions are incorporated in their plant (1) operating practices, procedures, and TSs, and (2) PRA model.

Although WCAP-15791 states that it is not expected that multiple systems will be out of service simultaneously during extended CTs, it does not preclude the practice. Because LCO 3.6.3 Note 2 allows separate condition entry for each penetration flow path, proposed Condition D addresses an inoperable CIV in more than one penetration flow path and limits the CT to 4 hours. Plant-specific applications must verify that the potential for any cumulative risk impact of failed CIVs and multiple CIV LCO entries has been evaluated and is acceptable.

The licensee's Tier 3 risk management program (10 CFR 50.65(a)(4)) must confirm that simultaneous entry for more than one inoperable CIV in separate penetration flow paths are evaluated. The purpose of this evaluation is to ensure that the cumulative risk of plant operation with multiple inoperable CIVs, including an CIV with an extended CT, does not exceed the conclusions of WCAP-15791 and this SE, and that defense-in-depth for safety systems is maintained.

Because not all penetrations in specific plants have the same impact on CDF, LERF, ICCDP, or ICLERP, licensees must address in their plant-specific applications whether or not (a) the CIV configurations for the specific plant match the configurations in the TR and (b) the risk parameter values used in the TR are bounding for the specific plant. Any additional CIV configurations or non-bounding risk parameter values in the plant-specific applications that were not evaluated by the TR must be addressed in the plant-specific analyses. Note that CIV configurations and extended CTs not specifically evaluated by the TR, or non-bounding risk parameter values outside the scope of the TR will require NRC staff review of the specific penetrations and related justifications for the proposed CTs.

### 3.3 Review Methodology

The NRC staff reviewed the WOG submittal using the three-tiered approach referenced in RG 1.174, RG 1.177, and SRP Chapters 16.1 and 19.0.

Tier 1 includes assessing the risk impact of the proposed change in accordance with acceptance guidelines consistent with the Commission's Safety Goal Policy Statement, as documented in RGs 1.174 and 1.177. The first tier assesses the impact on operational plant risk based on  $\Delta$ CDF and  $\Delta$ LERF. It also evaluates plant risk while equipment covered by the proposed CT is out of service, as represented by the ICCDP and ICLERP. In addition, Tier 1 should establish whether the quality of the PRA and PRA impact assessment are compatible with the safety implications of a proposed plant-specific TS change and that the scope and level of detail of the PRA are adequate to fully support the evaluation of the proposed TS change. Cumulative risk of the proposed TS change in light of past applications, or additional applications under review, should be considered along with an uncertainty/sensitivity analysis with respect to the assumptions related to the proposed TS change. PRA quality of the plant-specific application will be evaluated by the NRC staff, as discussed below.

Tier 2 involves identifying potential high-risk configurations that may exist if other equipment or systems (in addition to the equipment associated with the proposed change) were also taken out of service simultaneously, or subjected to concurrent testing. The purpose of the Tier 2 evaluation is to ensure that appropriate restrictions will be in place to prevent the occurrence of high-risk configurations.

Tier 3 establishes a risk management program for the overall configuration and confirms that risk insights are incorporated into the decisionmaking process before taking equipment out of service prior to or during the CT. The third tier provides additional coverage to ensure risk-significant plant equipment outage configurations are identified in a timely manner and that the risk impact of out-of-service equipment during planned and un-planned maintenance activities is appropriately evaluated prior to performing any maintenance activity over extended periods of plant operation. Licensees can implement the overall configuration risk management program (as referenced in RG 1.177) through the maintenance rule of 10 CFR 50.65(a)(4) where implementation of 10 CFR 50.65 incorporates PRA risk assessments of sufficient technical quality to support a plant-specific licensing action. Specifically, the rule requires that, before performing any maintenance activity, the licensee must assess and manage the potential risk increase that may result from that activity.

For the quantitative evaluation of the risk impact of extending the current CIV CT from 4 hours up to a maximum proposed duration of 7 days, the WOG developed a methodology to organize various containment penetrations into defined classes and sub-groups. This is addressed in Appendix A of this SE. For each defined class and sub-group, the WOG developed generic configurations of containment penetrations to assess the impact on the plant at-power risk utilizing the proposed 7-day CT, and shorter CTs, for the associated penetration CIVs and addressing Tier 1, Tier 2, and Tier 3, as discussed below.

### 3.3.1 Tier 1: PRA Applicability and Insights

The analyses used in WCAP-15791 are generic and, therefore, each licensee requesting CIV CT extensions will need to justify the applicability of the TR results to their particular plant. It is expected that licensees will evaluate its plant-specific information to confirm the applicability of the WCAP-15791 methodology and results to the plant-specific cases as presented in Chapter 9 or 10 of the TR. The CT changes requested by licensees should correspond to those included in the TR, and any penetration flow path type not specifically included in the TR will require a plant-specific analysis.

#### 3.3.1.1 PRA Applicability

The objective of the NRC staff's PRA review is to determine whether the TR generic risk assessments used in evaluating the proposed CIV extended CTs were of sufficient scope and detail. The NRC staff reviewed the information provided in WCAP-15791 and, based on the above discussion, the NRC staff concludes that the TR adequately addressed the issue of capability, and the risk analysis was of sufficient scope and detail to estimate the risk associated with the proposed CIV extended CTs on a generic basis. The quality of the licensees' PRA is applicable based on the approved methodology in WCAP-15791, the RG 1.174 PRA quality guidance, and the subsequent impact on Tier 2 and Tier 3 evaluations.

To ensure the applicability of WCAP-15791 to a licensee's plant, additional information on the PRA quality will be required of each plant-specific application in the following areas:

1. Assurance that the plant-specific PRA reflects the as-built, as-operated plant.
2. Assurance that the applicable PRA updates include the findings from the individual plant evaluation (IPE) and the IPE for external events. External events may include seismic,

high winds, fires, floods, or other related events applicable to each licensee. Licensees must demonstrate, by either quantitative or qualitative means, that external event risk will not have an adverse impact on the conclusions of the plant-specific analyses with respect to the TR evaluation.

3. Assurance that conclusions from the peer review, including facts and observations (A and B), per NEI 00-02, "Probabilistic Risk Assessment (PRA) Peer Review Process Guidance," Revision A3 and American Society of Mechanical Engineers (ASME) RA-S-2002, "Standard for Probabilistic Risk Assessment for Nuclear Power Plant Applications," that are applicable to the proposed CIV extended CTs were considered and resolved. If not resolved, justification for acceptability of conclusions (e.g., sensitivity studies showing negligible impact) must be provided. The licensee should indicate the PRA revision that underwent the peer review and the PRA revision that was used in the plant-specific application.
4. Assurance that there is PRA configuration control and updating, including PRA quality assurance programs, associated procedures, and PRA revision schedules.
5. Assurance that there is PRA adequacy, completeness, and applicability with respect to evaluating the risk associated with the proposed CIV CT extensions.
6. Assurance that plant design or operational modifications that are related to or could impact the proposed CT extensions are reflected in the PRA revision used in the plant-specific application, or a justification provided for not including these modifications in the PRA.

### 3.3.1.2 PRA Insights

One approach to demonstrate that the risk impact of the proposed change is acceptable is to show that the licensing basis meets the key principles set forth in RG 1.174 for the proposed change. One of these principles is to show that when the proposed change results in an increase in risk, the increased risk is small. In addition, the impact of the proposed change should be monitored using performance measurement strategies. RGs 1.174 and 1.177 provide acceptance guidelines for meeting the above principles. Specifically, those guidelines include  $\Delta$ CDF,  $\Delta$ LERF, ICCDP, and ICLERP. The risk metrics ICCDP and ICLERP, suggested by RG 1.177, are used in addition to the metrics outlined in RG 1.174 for the evaluation of CTs because CTs are entered infrequently and are temporary in nature.

The risk impact of extending CIV CTs is summarized on a generic basis in Table 8-2 of the TR. The results show that the risk impacts of the proposed CIV CTs are within the  $\Delta$ LERF and ICLERP acceptance guidelines of RGs 1.174 and 1.177, respectively. The impacts on average CDF and ICCDP due to increased CIV unavailability were addressed in a response to an NRC staff's RAI, and the response shows that the estimates for  $\Delta$ CDF and ICCDP are also within the acceptance guidelines of RGs 1.174 and 1.177. The intent of WCAP-15791 is to provide a generic analysis applicable to all WOG plants; however, the TR also includes a plant-specific analysis in Chapter 10 of the TR where Wolf Creek is the lead plant. A licensee that implements WCAP-15791 must demonstrate by its plant-specific application, the applicability of WCAP-15791 input parameter assumptions with respect to  $\Delta$ CDF,  $\Delta$ LERF, ICCDP, and ICLERP to their particular plant.

### 3.3.1.3 PRA Uncertainty

As discussed in RG 1.174 and NUREG/CR-6141, "Handbook of Methods for Risk-Based Analyses of Technical Specifications," a licensee can perform sensitivity studies to provide additional insights into the uncertainties related to the proposed CT extension and demonstrate compliance with the guidelines and evaluate uncertainties related to modeling and completeness issues.

Based on the RAI responses, the WOG stated the parameters used (e.g., valve failure rates, CDF, and CCF values) were based on generic WOG plant PRA values. The estimates used were stated in WCAP-15791 to be the most conservative values obtained from the WOG plant-specific PRA models. Because of this, the WOG stated that the values used in the analysis are bounding and no data uncertainty analysis was required. Therefore, WCAP-15791 did not provide sensitivity studies with respect to the CT extension risk analysis. However, based on the TR's use of bounding values for input parameters, a sensitivity analysis using an upper bound value should be inherent in the results. As a further check, the NRC staff reviewed NUREG-1715, Volume 3, Component Performance Study - Air-Operated Valves, 1987 - 1998," and Volume 4, "Component Performance Study-Motor-Operated Valves, 1987-1998 Commercial Power Reactors," data for motor-operated and air-operated valve failures on demand. Although limited to motor-operated and air-operated valves, the data presented in NUREG-1715 show that the CIV failure probability estimates used in WCAP-15791 are consistent with the range of values given in NUREG-1715. NUREG-1715 also indicated a decreasing trend for air-operated valve failures for PWRs in risk-important systems. For motor-operated valves, there were no statistically significant trends noted.

Additional uncertainty due to plant PRA models is not addressed in WCAP-15791, but the use of bounding values from various models should limit model uncertainty in the analysis. In addition, based on responses to the NRC staff's RAI, the WOG generic analysis was re-done assuming a CDF total of  $1.0E-4$ /year to bound internal and external events. The new CDF estimate is greater than the original WOG composite total plant internal CDF of  $7.8E-5$ /year.

However, each plant-specific application to modify CIV CTs, using WCAP-15791 as a basis, must discuss uncertainties in the risk assessment, including propagation of parameter uncertainties, as appropriate. The "state of knowledge" correlation must be accounted for in the uncertainty analysis in cases where a noticeable difference between point estimate and propagated mean value would be expected (e.g., ISLOCA, where parameter uncertainty of two similar valves' failure rates could be 100 percent correlated).

### 3.3.2 Tier 2: Avoidance of Risk-Significant Plant Configurations

For the Tier 2 analysis, a licensee must provide reasonable assurance that risk-significant plant equipment outage configurations will not occur when specific plant equipment is out of service in accordance with the proposed extended CIV CT changes. A Tier 2 program is intended to limit the degradation of plant mitigation capabilities with a CIV out of service (i.e., in a LCO condition) such that defense-in-depth is maintained. The TR evaluation identified no generic Tier 2 conditions as a result of the proposed CT extension for CIVs. For licensees adopting WCAP-15791, an evaluation should be performed to confirm that the conclusions of the TR concerning Tier 2 requirements remain applicable to the licensee's plant.

WCAP-15791 provides minimal guidance on cumulative risk impacts, although risk impact is recognized as part of a risk-informed review. With respect to past plant-specific license amendments, or additional plant-specific applications for a TS change under review, and the plant-specific CIV CT application based on WCAP-15791, the cumulative risk must be evaluated on a plant-specific basis consistent with the guidance given in RG 1.174, and addressed in the plant-specific application. Licensees should consider the guidance given in RG 1.174 for combined TS change requests.

### 3.3.3 Tier 3: Risk-Informed Configuration Risk Management

Because WCAP-15791 did not address Tier 3 requirements and is based on generic plant characteristics, except for the lead plant Wolf Creek plant-specific risk information presented in Chapter 10 of the TR, licensees adopting the TR must address Tier 3 information concerning their plants in their plant-specific applications.

A Tier 3 program ensures that, while a CIV is in a LCO condition, additional activities will not be performed that could further degrade the capability of the plant to respond to a condition the inoperable CIV or system was designed to mitigate, and, as a result, increase plant risk beyond that assumed by the TR analysis. Tier 3 programs do the following: (1) ensure that additional maintenance does not increase the likelihood of an initiating event intended to be mitigated by the out-of-service equipment, (2) evaluate the effects of additional equipment out of service during CIV maintenance activities that would adversely impact CIV CT risk such as from redundant systems or components, and (3) evaluate the impact of maintenance on equipment or systems assumed to remain operable by the CIV CT analysis. WCAP-15791 did not address Tier 3 requirements and, therefore, these requirements must be addressed by licensees in their plant-specific applications.

Accordingly, for extended CIV CTs, a licensee should have a program to ensure that it appropriately evaluates the risk impact of out-of-service equipment before performing a maintenance activity on a CIV. Licensees can utilize the overall CRMP, as referenced in RG 1.177, through the Maintenance Rule (i.e., 10 CFR 50.65(a)(4)) if the PRA risk assessment quality aspects of this program meet the quality needs of a risk-informed licensing action. Specifically, the rule requires that, before performing any maintenance activity, the licensee must assess and manage the potential risk increase that may result from that activity. Plant-specific applications referencing WCAP-15791 must include a discussion on the licensee's CRMP for assessing the risk associated with removal of CIVs from service and their conformance to the requirements of 10 CFR 50.65(a)(4), and the additions and clarifications outlined in Section 2.3.7.2 of RG 1.177 as they relate to the proposed extended CIV CTs. The PRA quality guidance in RG 1.174 provides one method to demonstrate such required quality.

The program used by licensees to meet 10 CFR 50.65(a)(4) and assess and manage the increase in risk that may result from proposed maintenance would be required to address the maintenance of CIVs. For the CIVs with extended CTs, the program would need to assess and manage risk in terms of the LERF and ICLERP metrics because the extended CTs are based on these metrics. The licensees proposing extended CIV CTs in plant-specific applications based on WCAP-15791 must discuss how the LERF and ICLERP would be addressed in these programs.

In addition, RG 1.174 also states that an implementation and monitoring plan should be developed to ensure that the impact of the proposed changes continues to reflect the actual reliability and availability of the CIVs evaluated to support the proposed CIV CT extension. Monitoring performed in conformance with the maintenance rule of 10 CFR 50.65 can be used when such monitoring is sufficient for the structures, systems, and components affected by the risk-informed application. Because WCAP-15791 is based on generic plant characteristics, each licensee adopting the TR must confirm plant-specific implementation and monitoring in their individual applications. Plant-specific CIV reliability and availability will be monitored and assessed by the licensee under the maintenance rule (10 CFR 50.65) to confirm that performance continues to be consistent with the analysis assumptions used to justify the extended CIV CTs.

Although the TR assumes that only one CIV is in maintenance at any time (i.e., there is only one inoperable CIV), the TR notes that the existing ISTS LCO 3.6.3 would allow multiple simultaneous entries into the LCO for inoperable CIVs for which the proposed extended CIV CTs would apply. Although the existing ISTS LCO 3.6.3 Condition B, which is listed in Section 3.1 of this SE, requires action within 1 hour when one or more flow paths with two or more inoperable CIVs; there could be multiple inoperable CIVs because the proposed ISTS LCO 3.6.3 Condition A, which has the proposed extended CIV CTs, would allow one or more flow paths each having an inoperable CIV. This case of multiple LCO entries for a single inoperable CIV, in multiple penetrations, incorporating the proposed extended CTs would result in increased CDF, LERF, ICCDP and ICLERP values from those assumed in the TR. Simultaneous multiple entries and the subsequent impact on risk were not evaluated by the WOG, because, as stated in the TR, CIV inoperability is not expected to occur frequently and single CIV inoperability in multiple penetrations flow paths is expected to occur less frequently.

In response to the NRC staff's RAI 6, in its February 13, 2004, letter, the WOG stated that the technical specifications in TSTF-446 would be revised to be consistent with the single inoperable CIV assumed in WCAP-15791. The intent of the revision is to limit the TS condition entry to a single extended CIV CT such that multiple simultaneous inoperable CIVs, each with an extended CT, would not be allowed. The revision is the proposed addition of TS LCO 3.6.3 Condition D, which states that, for two or more penetration flow paths with one CIV inoperable, the required action of isolating all but one flow path is to be completed within 4 hours, which is the current CT for an inoperable CIV. However, the proposed ISTS LCO 3.6.3 Condition A and the proposed ISTS LCO 3.6.3 Condition D would allow one inoperable CIV in one penetration flow path to be in maintenance in an extended CIV CT (Proposed Condition A) and one or more inoperable CIVs in one or more other penetration flow paths in the 4-hour CT (Proposed Condition D). Therefore, in the case of having multiple CIVs inoperable with no more than one inoperable CIV in any penetration flow path, the first inoperable CIV would be in the proposed Condition A with an extended CIV CT, and the second and any other inoperable CIV would be in proposed Condition D because Note 2 to ISTS LCO 3.6.3 allows a separate condition entry for each penetration flow path.

The Required Action D.1 for proposed Condition D would require all but one penetration flow path to be isolated within 4 hours of when the second CIV was found inoperable, or the plant would be required to shut down because the required action and associated CT were not being met. The remaining inoperable CIV would be in proposed Condition A, and its CT would be that for the appropriate CIV category in WCAP-15791, Appendix D.

Because the required action for proposed Condition D would require all but one penetration flow path to be isolated, within the 4 hours or start shutting down the plant, the case of more than one inoperable CIV would exist for no longer time than the 4 hours. Therefore, there could not exist more than one inoperable CIV in an extended CIV CT for more than 4 hours without the plant having to shut down. Because the longest time period where more than one CIV may be inoperable is the 4-hour CT allowed by existing ISTS LCO 3.6.3 Condition A for an inoperable CIV, the NRC staff concludes that the proposed Condition D meets the TR assumption that only one CIV is in maintenance at the extended CIV CT at any time.

If the plant-specific TSs would allow CIV maintenance that could include multiple simultaneous LCO entries for single inoperative CIVs, in multiple penetrations, then this case must be evaluated in the plant-specific applications to demonstrate that the risk-impact assumptions of CDF, LERF, ICCDP and ICLERP remain less than the RGs 1.174 and 1.177 acceptance guidelines and are consistent with the guidance contained in NUMARC 93.01, "Industry Guidelines for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," Section 11, "Assessment of Risk Resulting from Performance of Maintenance Activities," as endorsed by RG 1.182, "Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants." CIV extended CTs as implemented per the NRC staff findings and conditions of this SE and the maintenance rule (10 CFR 50.65(a)(4)) will limit the overall risk associated with extended CIV CT interval maintenance. As discussed above, the NRC staff has concluded that the proposed ISTS LCO 3.6.3 Condition D would prevent this case of multiple inoperable CIVs with one CIV in maintenance in an extended CIV CT.

### 3.4 Regulatory Commitments

The RG 1.177 Tier 3 program ensures that while a CIV is in a LCO condition, additional activities will not be performed that could further degrade the capabilities of the plant to respond to a condition for which the inoperable CIV or system was designed to mitigate, and as a result, increase plant risk beyond that assumed by the TR analysis. A licensee's implementation of RG 1.177 Tier 3 guidelines generally implies the assessment of risk with respect to CDF. However, the proposed CIV CT impacts containment isolation and consequently LERF and ICLERP, as well as CDF. Because the equations in WCAP-15791 to determine the extended CIV CTs are based on the LERF and ICLERP metrics, the management of risk in accordance with 10 CFR 50.65(a)(4) for these extended CIV CTs must assess LERF and ICLERP.

Therefore, a licensee's CRMP, including those implemented under the maintenance rule of 10 CFR 50.65(a)(4), must be addressed in the plant-specific submittal to explain how LERF/ICLERP is assessed and must be documented in the plant-specific applications as a regulatory commitment (i.e., included in the licensee's commitment tracking system in accordance with NEI 99-04, Revision 0, "Guidelines for Managing NRC Commitment Changes") in the licensees' plant-specific applications referencing WCAP-15791, as well as demonstrating PRA quality as part of the licensee's Tier 3 assessment. Since NUMARC 93-01 implements ICLERP as the quantitative risk metric (i.e., based on a zero maintenance model) and RG 1.177 utilizes ICLERP (i.e., based on an average maintenance model), the licensees, in their implementation of WCAP-15791, will need to demonstrate the equivalence for Tier 3 decisionmaking.

The NRC staff finds that reasonable controls for the implementation and for subsequent evaluation of proposed changes pertaining to regulatory commitment(s) can be provided by the

licensees' administrative processes, including their commitment management program. The NRC staff has agreed that NEI 99-04 provides reasonable guidance for the control of regulatory commitments made to the NRC staff (see Regulatory Issue Summary 2000-17, "Managing Regulatory Commitments Made by Power Reactor Licensees to the NRC Staff," dated September 21, 2000). The NRC staff notes that this establishes a voluntary reporting system for the operating data that is similar to the system established for the ROP PI program. The commitments would be controlled in accordance with the industry guidance or comparable criteria employed by a specific licensee. The NRC staff may choose to verify the implementation and maintenance of these commitments in a future inspection or audit. Should licensees choose to incorporate a regulatory commitment into the final safety analysis report or other document with established regulatory controls, the associated regulations would define the appropriate change-control and reporting requirements.

### 3.5 Conditions on Licensees Adopting WCAP-15791

1. WCAP-15791 is based on only one CIV that is in maintenance at any time. The TR states that it is not expected that multiple systems will be out of service simultaneously during extended CTs, but the TR does not preclude the practice. Although TS LCO 3.6.3 Note 2 allows separate condition entry for each penetration flow path, proposed Condition D (see Section 3.3.3 of this SE) addresses an inoperable CIV in more than one penetration flow path and limits the CT to 4 hours. If the licensees' proposed TS change does not include this Condition D, then the licensees' applications must verify that the potential for any cumulative risk impact of failed CIVs and multiple CIV LCO entries has been evaluated and is acceptable. The licensee must confirm that its Tier 3 risk management program in accordance with 10 CFR 50.65(a)(4) will address the possibility of simultaneous LCO entries of inoperable CIVs in separate penetrations such that this combination will not exceed the RG 1.174 and RG 1.177 acceptance guidelines confirmed by the analysis presented in WCAP-15791, and that defense-in-depth for safety systems is maintained. (See Section 3.2 of this SE.)
2. The existing and proposed TS 3.6.3 must not allow multiple simultaneous extended CIV CTs to occur for more than 4 hours, which is the existing CT for an inoperable CIV in ISTS LCO 3.6.3. This is to meet the TR assumption listed in Section 3.2 of this SE that only one valve within a single penetration can be in maintenance at a time (i.e., for more than the 4 hours allowed by the current ISTS LCO 3.6.3 Condition A). The existing ISTS LCO 3.6.3 Condition B, and the proposed ISTS 3.6.3 Conditions A and D, assure that this assumption is being met. If the TSs do not prevent this case, then this case must be evaluated in the plant-specific applications to demonstrate that the risk impact assumptions of CDF, LERF, ICCDP and ICLERP remain less than the RGs 1.174 and 1.177 acceptance guidelines as discussed in Section 3.3.3 of this SE. Also, the plant-specific application must address if the position and operability of the remaining CIVs in the affected penetration flow path, or another penetration flow path, are confirmed before entering the extended CT for the inoperable CIV. (See Section 3.3.3 of this SE.)

### 3.6 Additional Information Needed in Applications Referencing WCAP-15791

The additional information that must be provided in the plant-specific applications referencing WCAP-15791 is as follows:

1. Address how the general assumptions of WCAP-15791, which are listed in Section 3.2 in the SE, are incorporated in the specific plant practices, procedures, TSs, and PRA. (See Section 3.2 of this SE.)
2. Because not all penetrations have the same impact on CDF, LERF, ICCDP, or ICLERP, verify the applicability of WCAP-15791 to the specific plant, including verification that (a) the CIV configurations for the specific plant match the configurations in the TR and (b) the risk-parameter values used in the TR are bounding for the specific plant. Any additional CIV configurations, CT extensions, or non-bounding risk parameter values not evaluated by the TR should be addressed in the plant-specific analyses. Note that CIV configurations and extended CTs not specifically evaluated by the TR, or non-bounding risk parameter values outside the scope of the TR will require NRC staff review of the specific penetrations and related justifications for the proposed CTs. (See Sections 3.2 and 3.3.1 of this SE.)
3. Confirm that the Tier 2 conclusion of the TR (i.e., no Tier 2 requirements are needed) is applicable to the specific plant, or provide the plant-specific Tier 2 requirements needed for the plant. (Sections 3.3.2 of this SE.)
4. Because WCAP-15791 does not address Tier 3, each plant-specific application must address Tier 3 for the specific plant. The plant-specific application must discuss conformance to the requirements of the Maintenance Rule (i.e., 10 CFR 50.65(a)(4)), as the requirements relate to the proposed CIV CTs and the guidance contained in NUMARC 93.01, Section 11, as endorsed by RG 1.182, including verification that the licensee's maintenance rule program, with respect to CIVs, includes a LERF/ICLERP (i.e., ILERP as defined in NUMARC 93-01) assessment as part of the maintenance rule process, and that the PRA quality is adequate as part of the basis of a risk-informed licensing action. (See Sections 3.3.3 and 3.4 of this SE.)
5. Verify that the plant-specific PRA quality is acceptable for this application in accordance with the guidelines given in RGs 1.174 and 1.177, which are identified in the 6 items listed in Section 3.3.1.1 of this SE. This includes a verification that external event risk, including seismic and fires, either through quantitative or qualitative evaluation, is bounded by the TR assumptions and will not have an adverse impact on the conclusions of the plant-specific analysis for extending the CIV CTs. (See Section 3.3.1.1 of this SE.)
6. Address how plant-specific CIV reliability and availability are monitored and assessed at the plant under the Maintenance Rule (i.e., 10 CFR 50.65) to confirm that performance continues to be consistent with the analysis assumptions used to justify extended CIV CTs, including the assumptions in WCAP-15791 (which are discussed in Section 3.2 of this SE). (See Section 3.3.3 of this SE.)
7. The cumulative risk impact of the proposed CIV CT extensions must be addressed in the plant-specific application in accordance with the acceptance guidelines in RG 1.174. The cumulative risk impact must include both previous plant license changes and additional plant applications still under review. (See Section 3.3.2 of this SE.)

8. Because uncertainty due to plant PRA models is not addressed in WCAP-15791, the plant-specific applications must discuss uncertainties in the risk assessment. (See Section 3.3.1.3 of this SE.)
  9. Address the plant CRMP, including the Maintenance Rule program implemented under 10 CFR 50.65(a)(4), and explain how the LERF/ICLERP is assessed in the program. This assessment is to be documented in a regulatory commitment in the plant-specific application. (See Section 3.4 of this SE.)
- 3.7 NRC Staff Conclusions in Plant-Specific License Amendment Requests Referencing WCAP-15791

The results presented in WCAP-15791 are consistent with the specific guidance and acceptance guidelines given in RGs 1.174 and 1.177, and outlined in SRP Chapters 16.1 and 19.0 of NUREG-0800, and show a small increase in plant risk due to the extension of CIV CTs up to 7 days. The proposed CIV CTs in the plant-specific applications referencing WCAP-15791 would also meet this criteria if the plant-specific submittals showed the following:

- Licensees have confirmed that the generic WOG plant PRA values used for the Tier 1 evaluations in WCAP-15791 envelope the PRA values for their plants,
- Licensees have confirmed that the equations used in WCAP-15791 to justify extended CIV CTs are applicable to their plants,
- Licensees have confirmed that the assumptions (the first, third, and sixth bullets) in Section 3.2 in this SE are met at their plants, including the assumption that not more than one CIV is in maintenance and in an extended CT at a time,
- Licensees have confirmed that the CIV configurations addressed in their plant-specific application match the configurations in the TR and have correlated the CIVs to the 14 categories of CIVs, with CTs from 4 hours to 7 days as described in WCAP-15791 Tables D-1 and D-2, for their plants,
- Licensees have confirmed that (1) the generic WOG PRA values used for the Tier 1 evaluations in WCAP-15791 envelope the PRA values for their plants and (2) no additional Tier 2 requirements are needed, as is stated in WCAP-15791, or have acceptably addressed what are the Tier 2 requirements for their plants, and
- Licensees have acceptably addressed the Tier 3 requirements for their plants.

Based on the licensees demonstrating that the above review elements are met, the NRC staff would conclude in the evaluations of the plant-specific license amendment requests that the calculations of ICLERP and  $\Delta$ LERF for the proposed CIV CTs described in WCAP-15791 are acceptably justified. This includes the calculations performed to justify the CTs for the Category 1 through 14 valves listed in TR Tables D-1 and D-2. Based on this, the NRC staff would conclude that WCAP-15791 provides an acceptable methodology for determining plant-specific CIV CTs of up to 7 days, because the CIV CTs based on WCAP-15791 would meet 10 CFR 50.36.

#### 4.0 CONCLUSIONS ON WCAP-15791

The risk impact of the proposed 7-day CT for the CIV as estimated by  $\Delta$ CDF,  $\Delta$ LRF, ICCDP, and ICLERP, is consistent with the acceptance guidelines specified in RG 1.174, RG 1.177, and staff guidance outlined in SRP Chapters 16.1 and 19.0 of NUREG-0800. However, to be within these guidelines, some CIV CTs had to be less than 7 days. WCAP-15791 showed calculations whereby shorter than 7-day CTs were justified for certain groupings of the CIVs listed in WCAP Tables D-1 and D-2. The NRC staff finds that the risk-analysis methodology and approach used by the WOG to estimate the risk impacts were reasonable and of sufficient quality. The Tier 2 evaluation did not identify any risk-significant plant equipment configurations requiring TSs, procedure, or compensatory measures on a generic basis, but a plant-specific analysis of Tier 2 considerations must be done by licensees for plants adopting WCAP-15791 to confirm or adjust this aspect of the evaluation, as appropriate. WCAP-15791 references a CRMP for Tier 3 using 10 CFR 50.65(a)(4) to manage plant risk when CIVs are taken out of service. CIV reliability and availability will also be monitored and assessed under the maintenance rule (10 CFR 50.65) to confirm that performance continues to be consistent with the analysis assumptions used to justify extended CIVs CTs. Based on the above, and that the licensee demonstrates that PRA quality is adequate as part of the basis of a risk-informed application, the NRC staff finds that the proposed 7-day and shorter CIV CTs are acceptable for the CIVs as described in WCAP-15791. However, the conditions and additional information needed that are identified in Sections 3.5 and 3.6 of this SE must also be addressed by licensees adopting WCAP-15791 in their plant-specific applications.

Although Wolf Creek Generating Station plant-specific information was presented in Chapter 10 of the TR, the NRC staff did not review this data to draw any conclusion about the acceptability of CIV CTs in WCAP-15791 for the Wolf Creek plant. Also, although TSTF-446 is not addressed in the SE, it is referred to in Sections 3.1 and 3.3.3 of the SE because the WOG

referenced the TSTF in its response to an NRC RAI. The acceptability of the technical specifications in the proposed TSTF will be addressed in a separate evaluation.

- Attachments:
1. Appendix A, "WCAP-15791 Analysis of Containment Isolation Valve Completion Times."
  2. Table Addressing Disposition of WOG Comments

Principal Contributor: Cliff Doult

Date:

## APPENDIX A

### WCAP-15791 ANALYSIS OF CONTAINMENT ISOLATION VALVE COMPLETION TIMES

As stated in WCAP-15791, the containment isolation valves (CIVs) are used in plants to isolate the containment penetration flow paths with, typically, one CIV inside and one CIV outside containment performing this function. Depending on the purpose of the system which has CIVs, the CIVs may be normally open or closed. Also, these systems are considered either an open or closed system as follows: (1) an open system inside the containment is directly connected to the containment atmosphere and an open system outside containment is directly connected to the outside atmosphere and (2) a closed system inside containment is not directly connected to the containment atmosphere (e.g., a run of pipe inside containment) and a closed system outside containment is not directly connected to the outside atmosphere. A closed system might not have a CIV.

WCAP-15791 evaluated penetrations that connect directly to containment atmosphere, connect directly to the reactor coolant system (RCS), and connect directly to the steam generators (SGs). The evaluation included both penetration flow paths with multiple isolation valves and penetration flow paths with a single isolation valve and a closed system. The analysis also included CIV maintenance activities that cause the CIV to be inoperable as a pressure boundary or maintenance activities that allow a CIV to remain functional as a pressure boundary.

RG 1.177 provides generally acceptable bases for approving a TS change. Among these bases are improvements in operational safety, the TS change can be supported on a risk basis, and the change may be requested to reduce unnecessary burden caused by complying with current TS requirements. The Westinghouse Owners Group (WOG) stated that the proposed CIV extensions are to improve operational safety, and reduce unnecessary burden. The proposed CIV completion time (CT) extensions are intended to provide for the performance of on-line testing, maintenance, and repair of CIVs declared inoperable during Modes 1, 2, 3, and 4. The WOG stated the proposed changes are acceptable based on the low risk associated with the extended CTs. In addition, the extended CTs provide additional flexibility in the performance of preventive and corrective maintenance during power operation and reduce the potential for plant shutdown and possible plant transients introduced by this reactor mode change. The original intent of the WOG methodology was to extend the CT for an inoperable CIV to 7 days, consistent with the acceptance guidelines given in RGs 1.174 and 1.177; however, based on analysis results, a CT of less than the 7 days was required to meet the acceptance guidelines for some CIVs. Therefore, WCAP-15791 supports CTs of 4 hours to 168 hours for an inoperable CIV.

The approach taken by the WCAP applied both deterministic and probabilistic evaluations. The deterministic approach was used to determine the minimum-containment hole size that would result in a large release from containment and penetration flow paths connected to the containment atmosphere smaller than this size were proposed to have a CT of 7 days. All other penetrations were then evaluated on a probabilistic basis to demonstrate that either a CT of 7 days is acceptable, or to determine a CT that is less than seven days.

For Tier 1, probabilistic risk assessment (PRA) capability and insights, the WCAP assessed the impact of the proposed CT on the incremental conditional large early release probability (ICLERP) and change in large early release frequency ( $\Delta$ LERF) for each type of CIV penetration configuration that was evaluated. The impact of the change in core damage frequency ( $\Delta$ CDF) was not evaluated in the WCAP because containment isolation is a function that impacts the containment response to an event and not the ability of the plant design to prevent or mitigate core damage. However, as addressed in Section 3.2 of this safety evaluation (SE), the core damage frequency (CDF) was addressed in the WOG's response to the NRC staff request for additional information and the NRC staff addressed these CDF responses. (See the 12<sup>th</sup> bullet listed in Section 3.2 of this SE.) For Tier 2, avoidance of risk-significant plant configurations, WCAP-15791 did not identify any risk-significant Tier 2 plant equipment outage configurations requiring TS, procedure, or compensatory measures. The NRC staff has addressed Tier 2 requirements in Section 3.3.2 of this SE, and stated that this evaluation must also be performed by licensees adopting WCAP-15791 and the results must be confirmed on a plant-specific basis. For Tier 3, risk-informed plant configurations, the WCAP did not address Tier 3 requirements and, therefore, these requirements will be addressed by licensees on a plant-specific basis in their plant-specific applications.

For Tier 1, if the ICLERP and  $\Delta$ LERF for the CIV penetration configuration meet the criteria in Regulatory Guides (RGs) 1.174 and 1.177 (less than 5.0E-08 and 1.0E-07/year, respectively), a 168-hour (or 7-day) CT would be acceptable for the penetration configuration. For those penetration configurations that do not meet this criteria, CTs of 72, 48, 24, 12, or 8 hours were evaluated in the equations for ICLERP and  $\Delta$ LERF, and the largest CT where the ICLERP and  $\Delta$ LERF meets the criteria in RGs 1.174 and 1.177 was judged to be acceptable for the penetration configurations.

The probabilistic evaluation is consistent with NRC's approach for using PRA in risk-informed decisions on plant-specific changes to the plant current licensing basis addressed in RGs 1.174 and 1.177. This approach evaluated the risk impact of the CT on a generic basis and on a plant-specific bases using the Wolf Creek Generating Station as the lead plant. Data for both the generic case and for Wolf Creek are in WCAP-15791. Any licensee submitting a plant-specific application that references WCAP-15791 would need to demonstrate that the WCAP analysis is applicable to their plants.

The WCAP grouped the different types of penetration flow path configurations depending on (1) the system of interest and (2) if the system is closed or open with respect to the containment and outside atmospheres. In general, the following penetration configuration types that were evaluated are as follows:

Class I:

Penetrations with flow paths to the containment atmosphere:

- Group IA: Flow paths connected directly to the containment atmosphere and the outside environment (open/open penetration type).
- Group IB: Flow paths closed inside containment and connected directly to the outside environment (closed/open penetration).

- Group IC: Flow paths connected directly to the containment atmosphere and closed outside containment (open/closed penetration).
- Group ID: Flow paths closed inside containment and closed outside containment (closed/closed penetration type).

Class II:

Penetrations with flow paths to the RCS:

- Group IIA: Standby flow paths.
- Group IIB: Normally operating flow paths.

Class III:

Penetrations with flow paths to the SGs:

- Group IIIA: Flow paths connected to the SG secondary side and open to the outside environment.
- Group IIIB: Flow paths connected to the SG secondary side and closed to the outside environment.

In addition, the penetration flow paths within the above classification were further grouped by the following: (1) the arrangement of the CIVs (the penetration description) and (2) the maintenance on the inside and/or outside CIVs (i.e., the IC and/or OC valves) that affected the CIV's function as a pressure boundary and activities that allowed the CIVs to retain their pressure boundary functionality (the maintenance description). ICLERP and  $\Delta$ LERF calculations were then done for each group of CIVs to decide on the maximum CT for the group, and the different groups and calculations were listed in tables as to the generic assessment of the impact on risk (Chapter 8 of the WCAP), the lead plant application of the generic analysis (Chapter 9 of the WCAP), and the re-calculated CT for the lead plant-specific analysis (Chapter 10 of the WCAP).

Examples of the generic calculations of the ICLERP and  $\Delta$ LERF for different groups of CIVs are given in WCAP Section 8.2 for the generic assessment of the impact on risk. The calculation number and CIV group are listed in the table. The calculation number is the specific calculation used to determine the CT for the CIV; however, not all the calculations are in WCAP Section 8.2. There are no calculations in Chapters 9 and 10 of the WCAP because the calculations are the same in terms of the generic assessment of the impact on risk. Only the input numbers would change as the lead plant is considered in these chapters.

Chapter 9 of the WCAP presents the analysis and assumptions used in the lead plant application of the generic assessment addressed in Chapter 8 of the WCAP. This involved identifying the lead plant CIVs and their configuration, using the 2-inch containment hole size criteria to determine the "small lines" that are automatically justified for the 7-day CT, matching the remaining "large line" CIVs to the appropriate generic penetrations in Tables 8.2 through 8.4 of the WCAP, and finally determining the CTs based on the generic calculations of ICLERP and  $\Delta$ LERF for the CIVs. Table 9-2 of the WCAP lists the lead plant penetrations and the CIVs

on the penetration with the grouping explanation, the generic CIV group and calculation number, and the maintenance activity type for the CIV. In Chapter 9 of the WCAP, no plant-specific information was used except for the plant-specific CIVs and their configuration in the plant. The CTs justified for the CIVs are also given in Table 9-2 of the WCAP.

In Chapter 10 of the WCAP, the plant-specific PRA data (i.e., the Wolf Creek data) were also used in what would be the plant-specific calculations of ICLERP and  $\Delta$ LERF to determine the CTs for the CIVs. The plant-specific calculations used the data given in Tables 9-1a through 9-1c of the WCAP, and the justified CTs are listed in Table 10-1 of the WCAP.

For plant-specific applications, WCAP-15791 provides an option to use plant-specific data instead of generic data. The purpose of this option is to provide licensees the ability to further analyze an CIV that did not qualify for the full 7-day CT in the generic results. Chapter 10 of the TR describes the methodology to be used to replace the generic data in the analysis with plant-specific data such that CTs limited to 7 days by the generic methodology may qualify for a further extended CT using a plant-specific approach. The use of plant-specific data by a licensee must be justified when implementing WCAP-15791.

The results in both Tables 9-2 and 10-1 of the WCAP are for both the system pressure boundary maintained and system pressure boundary compromised for the maintenance activity type.

TABLE ADDRESSING

DISPOSITION OF WESTINGHOUSE OWNERS GROUP (WOG) COMMENTS

ON NRC DRAFT WCAP-15791 SAFETY EVALUATION (SE)

There are three sets of WOG comments: General Comments (GC), Specific Comments (SC), and Editorial Comments (EC) submitted in the WOG's letter dated October 19, 2005 (WOG-05-438). Each set of comments was separately numbered starting with 1 through the total number of those comments.

If the disposition of the comment is that the comment is acceptable, the NRC staff Safety Evaluation (SE) was revised to incorporate the comment; however, as identified in the resolution of comment, the revision of the SE may be different from that suggested in the comment. If the disposition of the comment is that the NRC staff disagrees with the comment, there is no change made to the SE. If the comment is partially acceptable, the revision of the SE is as described in the resolution of the comment.

<b>Number</b>	<b>WCAP 15791 SE Reference</b>	<b>Resolution of the Comment</b>
	General Comments	
GC1	N/A	The NRC staff disagrees. The NRC decided that the review of plant-specific information will be in the review of the plant-specific applications.
	Specific Comments	
SC1	Section 1.0 page 1, lines 11 - 23	The comment is acceptable.
SC2	Section 1.0 page 2, line 5	The comment is acceptable.
SC3	Section 1.0 page 2, lines 10 - 12	The comment is acceptable.
SC4	Section 1.0 Page 2, line 12	The NRC staff disagrees. The suggested WOG paragraph would be more appropriate for a WOG implementation guidance document. However, the generic results and methodology of WCAP-15791 have to be confirmed as applicable to a WOG member plant as part of incorporating WCAP-15791 on a plant specific basis. The point of the SER statement is that the methodology and generic results of WCAP-15791 may not be applicable for some plant specific applications.

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC5	Section 2.1 page 3, lines 35 - 36	The NRC staff disagrees. As stated at the beginning of section 2.2, 10 CFR 50.36 is the regulation governing the technical specifications. Although CTs are not specifically stated in 50.36, LCOs are addressed and 50.36(c)(2) states that when a LCO is not met, the licensee shall "shut down the reactor or follow any remedial action permitted by the technical specifications until the condition can be met." The action conditions and required actions in the technical specifications are the remedial actions and the CTs are allowed time for the specified remedial actions before the licensee shall shut down the reactor. If the basis for the extending the CTs is acceptable, then the CT is an acceptable time to complete the remedial action and the CT meets 10 CFR 50.36. If the staff can not conclude that the part of the TSs meets 10 CFR 50.36, then that part can not be in the TSs. The above text on remedial action and the CT having to be an acceptable time to complete the remedial action has been added to SE Section 2.0 on 10 CFR 50.36.
SC6	Section 2.1 page 3, lines 37-38	The comment is acceptable; however, the change should be, "utilizing risk information."
SC7	Section 2.2 page 5, lines 2 - 3	The comment is acceptable; however, the change to the SE will be the following:  "... provides additional coverage to ensure risk-significant plant equipment outage configurations are identified in a timely manner and that the risk impact of out-of-service equipment is appropriately evaluated prior to performing any maintenance activity over extended periods of plant operation."
SC8	Section 2.2 page 5, lines 11 - 12	The NRC staff disagrees. See the response to Comment No. SC5.
SC9	Section 3.1 page 6, line 1	The comment is acceptable, but state "... the CTs for inoperable CIVs in more than one penetration flow path ..." The important point is that the statement in the SE should be to limit the CT, not limit the possibility.
SC10	Section 3.1 page 6, line 2	The comment is acceptable.

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC11	Section 3.1 page 7, lines 13-18	<p>The comment is acceptable, however, replace the last sentence (lines 17 and 18 of the draft SE) with the last two sentences from the WOG response as follows:</p> <p>In response to the staff's RAI, the WOG evaluated potential impact of the CT extensions on the availability of other mitigative functions and the corresponding impact on risk. The WOG results show that this impact is very small.</p>
SC12	Section 3.2 page 7, line 41	<p>The comment is acceptable, but replace the sentence by the following: "Although TS LCO 3.6.3 Note 2 allows separate condition entry for each penetration flow path, proposed Condition D addresses an inoperable CIV in more than one penetration flow path and limits the CT, for all but one inoperable CIV, to four hours This is to say that TSTF 446 will limit the number of CIVs in an extended CT to no more than one at any given time."</p>
SC13	Section 3.2 page 8, line 6	<p>The comment is acceptable; however, the WOG may have misinterpreted line 6. It was intended for CIVs that were greater than the WCAP-15791 screening criteria. The screening of penetration flow paths of less than 2 inches to a default 7-day CT without a risk analysis is not clear in this section. The following text was added to the NRC staff SE for clarification after line 6:</p> <p>However, for penetration flow paths that do not result in a large early release (i.e., # 2 inches) the screening criteria presented in WCAP-15791 provides a default CT of 7 days in lieu of a risk analysis for these PCIVs.</p>
SC14	Section 3.2 page 8, lines 15 - 16	The comment is acceptable.
SC15	Section 3.2 page 9, lines 18 - 19	The comment is acceptable.
SC16	Section 3.2 page 9, lines 39 - 40	The comment is acceptable.
SC17	Section 3.2 page 10, line 10	The comment is acceptable, but state "or corrective maintenance (repair)."
SC18	Section 3.2 page 10, lines 18 - 19	The comment is acceptable.

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC19	Section 3.2 page 11, line 7	The comment is acceptable.
SC20	Section 3.2 page 11, lines 26 - 28	The comment is acceptable, but replace the sentence by the following: "Although TS LCO 3.6.3 Note 2 allows separate condition entry for each penetration flow path, proposed Condition D in TSTF-446, Revision 1, addresses an inoperable CIV in more than one penetration flow path and limits the CT to 4 hours. If the licensees' proposed TS change does not include this Condition D, then the licensees' application must verify that ... has been evaluated and is acceptable." The proposed Condition D is in TSTF-446, Revision 1, and not in WCAP-15791, Revision 1.
SC21	Section 3.2 page 11, lines 28 - 32	The comment is acceptable; however, lines 28 through 32 should read as follows:  The licensee's Tier 3 risk management program (10 CFR 50.65(a)(4)) must confirm that simultaneous entry for more than one inoperable CIV in separate penetration flow paths are evaluated. The purpose of this evaluation is to ensure that the cumulative risk of plant operation with multiple inoperable CIVs, including a CIV with an extended CT, does not exceed the conclusions of WCAP-15791 and this SE, and that defense-in-depth for safety systems is maintained.
SC22	Section 3.3 page 12, lines 27 - 28	The comment is acceptable. See the NRC staff's revised input provided for SC-7.
SC23	Section 3.3.1 page 13, lines 3 - 5	The comment is acceptable with this clarification. It is expected that licensees will evaluate its plant-specific information to confirm the applicability of the WCAP-15791 methodology and results to the plant-specific cases as presented in Chapter 9 or 10 of the TR. The CT changes requested by licensees should correspond to those included in the TR, and any penetration flow path type not specifically included in the TR will require a plant-specific analysis.
SC24	Section 3.3.1 page 13, lines 6 - 7	The comment is acceptable.
SC25	Section 3.3.1.1 page 13, line 15 to page 14, line 4	The comment is acceptable with this clarification. The quality of the licensees' PRA is applicable based on the approved methodology in WCAP-15791, the RG 1.174 PRA quality guidance, and the subsequent impact on Tier 3 evaluations.

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC26	Section 3.3.1.2 page 14, lines 20 - 22	The NRC staff disagrees. The NRC decided that the review of plant-specific information will be in the review of the plant-specific applications. See the response to Comment No. GC1.
SC27	Section 3.3.1.2 page 14, lines 22 - 24	The comment is acceptable, but state that "A licensee that implements WCAP-15791 must demonstrate, in its plant-specific application, the applicability of WCAP-15791 input parameter assumptions ..."
SC28	Section 3.3.1.3 page 15, lines 11 - 16	The NRC staff disagrees. WCAP-15791 provides a methodology. As stated in the SE, each licensee is to discuss the uncertainties in the plant specific application of WCAP-15791 - this is consistent with RG 1.174 and 1.177 guidance.
SC29	Section 3.3.2 page 15, line 29	The comment is acceptable.
SC30	Section 3.3.3 page 16, lines 12 - 13	<p>The NRC staff disagrees with the following clarification. The intent is to state that a CRMP can be implemented by 10 CFR 50.65(a)(4) with respect to risk-informed TS changes subject to the additional guidance in Section 2.3.7.2 of RG 1.177 and RG 1.182. Specifically, that the CRMP, as implemented by the maintenance rule, provides an adequate risk-informed assessment to manage the risk of the proposed risk-informed licensing action. In the case of evaluating CIV maintenance and associated impacts on LERF, the fact that a plant PRA is used to comply with the 10 CFR 50.65 is not, by itself, a basis for acceptability with respect to application of a licensing action.</p> <p>See the change made to the NRC staff SE in the resolution of Comment SC32.</p>
SC31	Section 3.3.3 page 16, lines 15 - 19	<p>The NRC staff disagrees with the following clarification. A plant that implements a license amendment based on WCAP-15791 and credits a CRMP program that implements the 10 CFR 60.65(a)(4) needs to discuss their plant's CRMP program and its acceptability with respect to the proposed license amendment including the acceptance guidelines (i.e. key components) as outlined in Section 2.3.7.2 of RG 1.177 and RG 1.182.</p> <p>See the change made to the NRC staff SE in the resolution of Comment SC32.</p>

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC32	Section 3.3.3 page 16, lines 23 - 27	<p>The NRC staff disagrees with the following clarification on why the licensee is to describe how LERF and ICLERP metrics are to be addressed by a licensee's CRMP per a licensee commitment.</p> <p>NUMARC 93-01 addresses ILERP but does not specify that it will be evaluated with respect to CIVs. The analysis of level 2 is optional. CIVs may be evaluated within the assessment of SSCs important to containment performance and may be covered by the inclusion of safety significant SSCs. Section 9.3.1 of NUMARC 93-01 discusses the importance of containment performance as a consideration in identifying risk significant SSCs. Section 9.3.1 of NUMARC 93-01 is not endorsed by RG 1.182.</p> <p>NUMARC 93-01, Rev. 3, as an option, uses the product of incremental CDF and LERF and CT duration (i.e. incremental core damage probability (ICDP) and incremental large early release probability (ILERP)) as well as the configuration specific CDF to establish the threshold for quantitative risk management actions during maintenance activities. The guidance for NUMARC 93-01 does not provide a risk metric for a configuration specific LERF which will need to be addressed on a licensee's plant-specific basis when adopting WCAP-15791.</p> <p>Appendix E to NUMARC 93-01 states that, "The PSA is not required to be expanded to quantitatively address containment performance (level 2), external events, or conditions other than power operation. Use of such an expanded PSA is an option." (Emphasis added.) The above risk metric "ILERP" is identified in Section 11.3.7 of NUMARC 93-01 as a <u>quantitative</u> risk metric. It appears based on the WOG's comment that the WOG is committing to a quantitative evaluation of condition-specific LERF and ILERP with respect to plant-specific CIV submittals. This commitment, however, should be provided in the plant-specific submittal.</p> <p>However, item 9 of Section 3.6 of the NRC staff SE will be revised to replace "the ... assessment and methodology" to "the ... is assessed."</p>

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC33	Section 3.3.3 page 16, lines 40 - 42	The NRC staff disagrees. There is no need for the WOG's proposed change because the point being made by the WOG is within the paragraphs of SE page 17, line 8 through SE page 18, line 6, where the NRC staff concludes that the proposed Condition D does ensure that the extended CIV CT is limited to one inoperable CIV in one penetration flow path. The proposed Condition D is in TSTF-446, Revision 1, and not in WCAP-15791, Revision 1.
SC34	Section 3.3.3 page 17, lines 24 - 25	The comment is acceptable, but state the following: "The Required Action D.1 for proposed Condition D would require all but one penetration flow path to be isolated within 4 hours of when the second CIV was found inoperable, or the plant would be required to shut down in accordance with the proposed Condition G of the TSTF-446, Revision 1."
SC35	Section 3.3.3 page 17, lines 29 - 30	The comment is acceptable.

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC36	Section 3.4 page 18, lines 15 - 16	<p>The NRC staff disagrees with the following clarification on why the licensee is to describe how LERF and ICLEP metrics are to be addressed by a licensee's CRMP.</p> <p>NUMARC 93-01, Rev. 3, as endorsed by RG 1.182, alternatively uses the product of incremental CDF and LERF and CT duration (i.e. incremental core damage probability (ICDP) and incremental large early release probability (ILERP)) as well as the configuration specific CDF to establish the threshold for quantitative risk management actions during maintenance activities. The guidance for NUMARC 93-01 does not provide a risk metric for a configuration specific LERF which will need to be addressed on a licensee's plant-specific basis when adopting WCAP-15791.</p> <p>Appendix E to NUMARC 93-01 states that, "The PSA is not required to be expanded to quantitatively address containment performance (level 2), external events, or conditions other than power operation. Use of such an expanded PSA is an option." (Emphasis added.) The above risk metric "ILERP" is identified in Section 11.3.7 of NUMARC 93-01 as a <u>quantitative</u> risk metric. It appears based on the WOG's comment that the WOG is committing to a quantitative evaluation of condition-specific LERF and ILERP with respect to plant-specific CIV submittals. This commitment, however, should be provided in the plant-specific submittal.</p> <p>However, the NRC staff SE is revised to state that the plant CRMP, including the Maintenance Rule program implemented under 10 CFR 50.65(a)(4), must be addressed in the plant-specific applications to explain how the LERF/ICLEP is assessed in the program.</p>

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC37	Section 3.4 page 18, line 18	The comment is acceptable with the following clarification. ILERP is a quantitative metric used by NUMARC 93-01. LERF and ICLERP assessment is to be addressed by the plant specific submittal. The following will be added to the end of the paragraph in the SE: Since NUMARC 93-01 implements ILERP as the quantitative risk metric (i.e., based on a zero maintenance model) and RG 1.177 utilizes ICLERP (i.e., based on an average maintenance model), the licensees, in their implementation of WCAP-15791, will need to demonstrate the equivalence for Tier 3 decisionmaking.
SC38	Section 3.4 page 18, lines 18-23	The NRC staff disagrees. See the resolutions for Comments SC32, SC36, and SC37.
SC39	Section 3.4 page 18, line 41	The NRC staff disagrees. The proposed Condition D is in TSTF-446, Revision 1, and not in WCAP-15791, Revision 1. See the resolution of Comment No. SC33.
SC40	Section 3.5 page 18, line 41 to page 19, line 2	The comment is acceptable, but replace the sentence by the following: "Although TS LCO 3.6.3 Note 2 allows separate condition entry for each penetration flow path, proposed Condition D in TSTF-446, Revision 1, addresses an inoperable CIV in more than one penetration flow path and limits the CT to 4 hours. If the licensees' proposed TS change does not include this Condition D, then the licensees' application must verify that ... has been evaluated and is acceptable." The proposed Condition D is in TSTF-446, Revision 1, and not in WCAP-15791, Revision 1. See also Comment No. SC20.

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC41	Section 3.5 page 19, lines 2 - 7	<p>The NRC staff disagrees with the following clarification:</p> <p>NUMARC 93-01 Section 11.3.4 states, "Simultaneous removal from service of multiple SSCs requires that an assessment be performed using quantitative, qualitative, or blended (qualitative and qualitative) methods. Sections 11.3.4.1 and 11.3.4.2 provide guidance regarding quantitative and qualitative considerations, respectively."</p> <p>However, WCAP-15791 <u>did not</u> evaluate multiple simultaneous extended CIV CTs in separate penetrations. The current TS or as modified by WCAP-15791 does not preclude the practice for extended CIV CTs. Therefore, multiple simultaneous extended CIV CTs are not directly applicable to the guidance provided by NUMARC 93-01. This is because NUMARC 93-01 provides qualitative and/or quantitative risk metrics differing from that used to approve WCAP-15791. Multiple simultaneous extended CIV CTs will need to be addressed on a plant-specific basis and accepted based on the methodology presented in WCAP-15791 and not solely on NUMARC 93-01 evaluations alone.</p>
SC42	Section 3.5 page 19, lines 9 - 10	<p>The comment is partially acceptable. It is acceptable to change "TSs" to "TS 3.6.3"; however there is disagreement for the rest of this comment because the SE statement is a statement of fact. The proposed TS changes in the plant-specific applications implementing WCAP-15791 "must not allow multiple simultaneous extended CIV CTs to occur for more than 4 hours." The proposed Condition D is in TSTF-446, Revision 1, and not in WCAP-15791, Revision 1. A change was made to the NRC staff SE.</p>

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC43	Section 3.5 page 19, lines 18 - 21	<p>The NRC staff disagrees with the following clarification:</p> <p>The confirmation of position of the remaining CIVs before maintenance or repair is started on one CIV is a general assumption stated in WCAP-15791 and reads as follows:</p> <p>“3. Before maintenance or repair is started on one CIV, it is assumed that the other CIVs within the penetration are checked to ensure they are in the proper positions. This assumption eliminates the need to include the probability that operable valves were mis-positioned or transferred to the wrong position <u>since they were last checked.</u>”</p> <p>TS requirements were not referenced or considered as satisfying this analysis assumption. Therefore, confirming the position as well as operability of the remaining CIVs was listed as a condition in the SER.</p>

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC44	Section 3.6 page 20, line 6	<p>The comment is partially acceptable with the following clarification:</p> <p>With respect to the analysis presented in WCAP-15791 a LERF and ICLERP assessment provides assurance that the assumptions of WCAP-15791 remain satisfied. A 10 CFR 50.65(a)(4) evaluation using NUMARC 93-01 guidance <u>has the option</u> to address risk with a CIV out-of-service using incremental LERF (ILERP) as the risk metric. ILERP is based on a configuration specific LERF (CIV out of service) and a zero maintenance baseline LERF as opposed to ICLERP which assumes a nominal maintenance configuration.</p> <p>Although, NUMARC 93-01 provides for quantitative consideration of a configuration-specific CDF metric, it does not specifically include consideration of a configuration specific LERF metric. Since the analysis of WCAP-15791 is based on both a LERF and ICLERP analysis for a single extended CIV CT out-of-service a licensee's 10 CDF-50.65(a)(4) evaluation will need to assess both LERF and ICLERP (ILERP) as stated in the staff SER.</p> <p>The following revision was made to lines 5, 6, and 7 on page 20 in the draft NRC staff SE:</p> <p>"...the licensee's maintenance rule program, with respect to CIVs, includes a LERF/ICLERP (i.e. ILERP as defined in NUMARC 93-01) assessment ..." The use of ILERP in NUMARC 93-01 is based on a zero maintenance baseline and should provide results conservative to ICLERP which uses a nominal maintenance configuration.</p>
SC45	Section 3.6 page 20, lines 7 - 8	<p>The NRC staff disagrees with the following clarification:</p> <p>The plant specific Tier 2 and 3 analyses was not part of WCAP-15791 and will need to be discussed in a licensee's plant-specific submittal. The adequacy of the topical reports Tier 2 as well as Tier 3 analysis, will need to be assessed. If a licensee's PRA is to be used for a CRMP / 10 CFR-50.65(a)(4) evaluation, the quality of this PRA needs to be established with respect to the proposed license amendment request.</p>

Number	WCAP 15791 SE Reference	Resolution of the Comment
SC46	Section 3.6 page 20, lines 10 - 16	The NRC staff disagrees. See the response to Comment SC-45.
SC47	Section 3.6 page 20, lines 29 - 32	The NRC staff disagrees. The evaluation of a CRMP is plant specific. The CRMP, including programs implemented under 10 CFR 50.65(a)(4) should be presented by each licensee implementing WCAP-15791. The CRMP should discuss how LERF and ICLERP are evaluated. If the CRMP is implemented per the maintenance rule using the guidance of NUMARC 91-03 then the applicability of the risk metrics (including ILERP) and the methodology used to assess the risk of CIVs out-of-service should be provided in the plant specific submittal.
SC48	Section 3.7 page 21, lines 1 - 2	The NRC staff disagrees. The NRC needs to have the licensees state in their plant-specific applications for TS changes that the generic WOG plant PRA values used for Tier 1 evaluations in WCAP-15791 envelope the PRA values for their plants.
SC49	Section 3.7 page 21, lines 3 - 4	The NRC staff disagrees. The NRC needs to have the licensees state in their plant-specific applications for TS changes that the equations used in WCAP-15791 to justify extended CIV CTs are applicable to their plants.
SC50	Section 3.7 page 21, lines 5 - 7	<p>The comment is partially acceptable as explained below:</p> <p>The NRC staff agrees that some of the assumptions stated in WCAP-15791 and in Section 3.2 of the staff SER were intended to facilitate the analysis. However, the staff disagrees on the assumptions to be confirmed in a licensee's plant specific submittal.</p> <p>A reference to the specific assumptions listed in Section 3.2 (i.e., bulleted items 1, 3, and 6) that need to be confirmed on a plant-specific basis will be added to the NRC staff SE.</p> <p>TSTF-446 and its additional reviewer's note allowing removal of the limitation on no more than one CIV in an extended CT was not part of the WCAP-15791 review and will be reviewed separately.</p>
SC51	Section 3.7 page 21, line 12	The comment is acceptable.

<b>Number</b>	<b>WCAP 15791 SE Reference</b>	<b>Resolution of the Comment</b>
SC52	Section 3.7 page 21, line 23	The NRC staff disagrees. See the response to Comment No. SC5.
SC53	Section 3.7 page 21, lines 33 - 35	The comment was a question asking what the phrase "... but a plant-specific analysis ..." meant in the referenced sentence. The phrase was included in the sentence because the generic Tier 2 evaluation results must be confirmed to be applicable on a plant specific basis. It would be expected that a plant specific analysis consistent with the methodology presented in WCAP-15791 would be used to ensure the WCAP-15791 Tier 2 results are applicable. No revision was made to the staff SE.
SC54	Section 4.0 page 22, lines 5 - 7	The NRC staff disagrees. See the responses to Comment Nos. SC26 and GC1. The NRC decided that the review of plant-specific information will be in the review of the plant-specific applications.
SC55	Appendix A page A-1, line 41	The comment is acceptable, but state that "... to determine a CT which is less than seven days."
SC56	Appendix A page A-2, lines 13 - 14	The NRC staff disagrees. The WOG did not comment on SE page 15, lines 24 and 25 where the NRC staff is making the statement referred to on SE page A-2.
SC57	Appendix A page A-2, lines 27 - 28	The NRC staff disagrees. See the responses to Comment Nos. SC26, SC54, and GC1. The NRC decided that the review of plant-specific information will be in the review of the plant-specific applications.
SC58	Appendix A page A-3, line 25	The comment is acceptable, but state "Examples of the generic calculations ..." Not all of the generic calculations listed in the tables in WCAP-15791 Section 8.2 are given in that section.
	Editorial Comments	
EC1	Cover letter, page 1	The comment is acceptable.
EC2	Cover letter, page 2	The comment is acceptable.
EC3	Cover letter, page 2 page 1, line 19	The comment is acceptable.
EC4	Section 1.0 page 1, line 24	The comment is acceptable.
EC5	Section 1.0 page 2, line 7	The comment is acceptable.

<b>Number</b>	<b>WCAP 15791 SE Reference</b>	<b>Resolution of the Comment</b>
EC6	Section 2.1 page 3, line 13	The comment is acceptable.
EC7	Section 3.1 page 5, line 23	The comment is acceptable.
EC8	Section 3.1 page 5, lines 28 - 36	The comment is acceptable.
EC9	Section 3.1 page 5, line 39	The comment is acceptable.
EC10	Section 3.1 page 6, lines 22 and 36	The NRC staff disagrees. This change could be done; however, all the other times in the NRC staff SE are given in hours.
EC11	Section 3.1 page 7, line 1	The comment is acceptable.
EC12	Section 3.1 page 7, line 10	The comment is acceptable.
EC13	Section 3.1 page 7, line 17	The comment is acceptable, the revision to the NRC staff SE is based on the resolution of Comment SC11.
EC14	Section 3.2 page 9, line 12	The comment is acceptable.
EC15	Section 3.2 page 9, line 25	The comment is acceptable.
EC16	Section 3.2 page 10, line 1	The comment is acceptable.
EC17	Section 3.2 page 10, line 21	The comment is acceptable.
EC18	Section 3.2 page 11, line [38]	The comment is acceptable.
EC19	Section 3.3 page 12, line 30	The comment is acceptable.
EC20	Section 3.3.3 page 16, line 38	The comment is acceptable.
EC21	Section 3.3.3 page 16, line 39	The comment is acceptable.
EC22	Section 3.3.3 page 17, line 3	The comment is acceptable.

<b>Number</b>	<b>WCAP 15791 SE Reference</b>	<b>Resolution of the Comment</b>
EC23	Section 3.7 page 21, line 20	The comment is acceptable.
EC24	Section 4.0 page 21, line 29	The comment is acceptable.
EC25	Appendix A page A-2, line 18	The comment is acceptable.
EC26	Appendix A page A-2, line 24	The comment is acceptable.