December 9, 1997

Mr. Robert A. Williams
Safeguards Coordinator
Westinghouse Electric Corporation
Commercial Nuclear Fuel Division
Drawer R
Columbia, South Carolina 29250

SUBJECT: AMENDMENT OF LICENSE TO AUTHORIZE RELEASE OF INDUSTRIAL WASTE TREATMENT PRODUCTS TO CEMENT MANUFACTURERS (TAC NO. L31003)

Dear Mr. Williams:

In accordance with your application dated July 14, 1997, and supplement dated November 17, 1997, and pursuant to Part 70 to Title 10 of the Code of Federal Regulations, Materials License SNM-1107 is hereby amended to authorize release of industrial waste treatment products (primarily calcium fluoride) without continuing NRC licensing controls to receivers for cement manufacturing and to delete authorization for use of licensed materials at off-site locations. Accordingly, Safety Condition S-1 has been revised to include the dates of July 14 and November 17, 1997. All other conditions of this license shall remain the same.

Enclosed are copies of the revised Materials License SNM-1107 and the Safety Evaluation Report, which includes the Categorical Exclusion determination.

Sincerely,
Original signed by:
Walter S. Schwink, Acting Chief
Licensing Branch
Division of Fuel Cycle Safety
and Safeguards, NMSS

Docket 70-1151
License SNM-1107
Amendment 8

Enclosures: 1. Materials License SNM-1107
2. Safety Evaluation Report

cc: Mr. Wilbur Goodwin, Regulatory Affairs
Westinghouse Electric Corporation
Commercial Nuclear Fuel Division
Drawer R
Columbia, SC 29250

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NAME	SChotoo	SC	CGoelzin		PShea	PS	MAGamberoni		MAdams	MA	WSchwink	WS
DATE	12/ 9 /97		12/ 9 /97		12/ 9 /97		12/ 9 /97		12/ 9 /97		12/ 09 /97	

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	
1. Westinghouse Electric Corporation P.O. Box 355 2. Pittsburgh, Pennsylvania 15230-0355	3. License Number: SNM-1107 Amendment 8 4. Expiration Date: November 30, 2005 5. Docket or Reference No.: 70-1151

6. Byproduct, Source, and/or Special Nuclear Material	7. Chemical and/or Physical Form	8. Maximum Amount that Licensee May Possess at Any One Time Under This License
A. U-235	A. Any	A. 0.35 kg
B. U-235	B. Any, except metal, enriched to not more than 5.0 w/o	B. 75,000 kgs
C. U-233	C. Any	C. 5 grams
D. Pu-238, Pu-239	D. Sealed sources	D. 1.5 grams

9. Authorized place of use: The licensee's existing facilities at Columbia, South Carolina.

10. This license shall be deemed to contain two sections: Safety Conditions and Safeguards Conditions. These sections are part of the license and the licensee is subject to compliance with all listed conditions in each section.

FOR THE NUCLEAR REGULATORY COMMISSION

Date: 12/09/97

By: Walter S. Schwink (Walter S. Schwink)
Division of Fuel Cycle Safety
and Safeguards
Washington, DC 20555

SAFETY CONDITIONS

S-1 Authorized use: For use in accordance with statements, representations, and conditions in the license application dated April 30, 1995, and supplements dated May 11 and 18, August 4 and 25, September 25, 1995; July 14 and 25, August 11, and November 17, 1997.

S-2. The Criticality Safety Evaluations (CSEs) and Criticality Safety Analysis (CSAs) will define the criticality safety bases utilized throughout the plant and will be completed as described in Sections 6.4.1 and 6.4.2 of the license application for each step of the process. A submittal to the NRC will be made by Westinghouse which will include information which the staff will review against the commitments listed in Section 6.2.3 of the license application. These submittals will not be specific commitments, but are intended as a demonstration of the adequacy of the bases of safety itemized in Section 6.2.3. The contents listed in Section 6.4.2 of the license application are commitments. The demonstration will show the logic used by the Columbia Fuel Fabrication Facility (CFFF) in developing the criticality safety bases. It is understood that this demonstration may change over time. The submittal will consist of a summary of the CSE or CSA as annotated below for each system (which shall include fault trees for those systems for which a CSE will be done). The adequacy of the commitments in Section 6.2.3 of the license application will be evaluated and a reply in the form of a letter report will be generated and sent from the staff to Westinghouse. The submittal by the CFFF will be provided according to the following schedule:

- UN tanks - summary of CSE -10/95
- URRS Dissolver - summary of CSE - 10/95
- Solvent Extraction - summary of CSE - TBD*
- URRS Scrap Processing:
- Low-Level Waste Processing System - summary of CSA - 9/96
  Mop Water System Equipment - summary of CSA - 9/96
  Incinerator System - summary of CSE - 9/96
  Ash Recovery System - summary of CSE - 9/96
  Liquid Honing System - summary of CSE - 9/96
  Ultrasonic Cleaning System - summary of CSE - 9/96
  Shredder System - summary of CSE - 9/96
- URRS Waste Treatment - summary of CSA - 11/95
- IDR Conversion - summary of CSE - Prior to Restart
- ADU Conversion - summary of CSE - 4/96
- ADU Pelleting - summary of CSE - 6/96
- Powder Blending - summary of CSE - 1/96
- Miscellaneous:
  Ventilation Systems (Geometry Controlled) -
  summary of CSE - 12/97**

Ventilation Systems (Moderation Controlled) - summary of CSE - 12/97**
Scrubber Systems - summary of CSE - 12/97**
Hoods & Containment - summary of CSA - 9/96
Laboratories - summary of CSA - 9/96

- IFBA - summary of CSA - 3/96
- Storage Pad - NCS is based on DOE reports K-1920 and K-1663
- Rods - summary of CSA - 3/96
- Storage - summary of CSE - 4/97
- Final Assembly - summary of CSE - 6/97*
- UF_6 Cylinder Washing - summary of CSE - 12/95

ERR REC'd

** The CFFF has performed a CSA upgrade and does not have a specific date for schedule of this CSE at this time.*

*** These systems are to be coupled with the Columbia Plant Risk Management Program (RMP) to be done for the EPA.*

S-3 The licensee shall maintain and execute the response measures in the Site Emergency Plan, dated April 30, 1990, and revisions dated March 31, and September 30, 1992; March 25, August 15, and September 30, 1994; January 9, February 17, August 17, and October 23, 1995; or as further revised by the licensee consistent with 10 CFR 70.32(i).

S-4 Notwithstanding the requirements of 10 CFR 20.205(b) to monitor the external surfaces of packaged radioactive material receipts for radioactive contamination, the licensee is exempted from such requirement relative to flatbed trailer shipments of fuel assemblies received from the General Electric Company for interim storage purposes only, provided the constraints, conditions and controls committed to in the licensee's letter, dated November 30, 1993, (identification # NRC-93-036), are satisfied; and further provided that the total number of such fuel assemblies stored at the licensee's site at any given time does not exceed 250.

SAFEGUARDS CONDITIONS

SECTION 1.0 -- MATERIAL CONTROL AND ACCOUNTING

SG-1.1 The licensee shall follow pages i through xvii and Chapters 1.0 through 9.0 of its "Fundamental Nuclear Material Control Plan for the Columbia Fuel Fabrication Facility," with all pages identified as Revision 27 (dated May 23, 1997). Any further revisions to this Plan shall be made only in accordance with, and pursuant to, either 10 CFR 70.32(c) or 70.34.

SG-1.2 Operations involving special nuclear material which are not referenced in the Plan identified in Condition SG-1.1 shall not be initiated until an appropriate safeguards plan has been approved by the Nuclear Regulatory Commission.

SG-1.3 In lieu of the requirements contained in 10 CFR 74.13(a)(1) and (a)(2) to use the Forms DOE/NRC-742 and 742C, the licensee may use computer generated forms provided all information required by the latest printed instructions for completing the particular form is included.

SG-1.4 In lieu of the requirements contained in CFR 70.54 and 74.15 to use the DOE/NRC Form-741, the licensee may use computer generated forms provided all information required by the latest printed instructions for completing the particular form is included.

SG-1.5 Deleted Per Amendment 3, August 1996 Commitment now contained in licensee's Fundamental Nuclear Material Control Plan.

SG-1.6 Notwithstanding the requirements of the FNMC Plan identified in License Condition SG-1.1, the licensee may use (1) a single standard for measurement control (including daily control limit monitoring and bias corrections) for any linear-response tube or rod scales, in any initially demonstrated to be linear over its range of use within the discrimination of the scale by calculating a bias at four levels across the range of use and demonstrating that the four results are not statistically different, and (2) that the continued linearity of response of the scales is verified by monthly calibration against at least four traceable standards covering the range of use.

SG-1.7 Notwithstanding the requirements contained in Sections 5.2.2 and 5.2.3 of the licensee's Fundamental Nuclear Material Control Plan, the licensee is exempted from physical inventory requirements relative to the material identified in Condition S-4; provided the conditions and commitments contained in the licensee's November 30, 1993, letter (identification # NRC-93-036) are satisfied.

MATERIALS LICENSE SUPPLEMENTARY SHEET

License Number: SNM-1107

Docket or Reference Number: 70-1151

Safeguards Conditions

SG-1.8 Notwithstanding the requirement of Section 6.2.1(a).5 of the licensee's Fundamental Nuclear Material Control Plan to unpackage and perform an item count upon receipt of special nuclear material, the licensee is exempted from such requirement relative to the material identified in Condition S-4; provided the conditions and commitments contained in the licensee's November 30, 1993, letter (identification # NRC-93-036) are satisfied.

SG-1.9 Notwithstanding the requirement of Section II.A.7, block U, of NUREG/BR-0006, which is incorporated via 10 CFR 74.15, to complete receiver's measurements of scrap receipts (following recovery processing) within 60 days of receipt, the licensee shall not be subject to any time limit relative to recovering and measuring received UF_6 heels when the block U action code (of DOE/NRC Form 741) is used to book such receipts.

SG-1.10 With respect to Section 5.1.4 (b) of the Plan identified by Condition SG-1.1, *"allowed number"* within the phrase *"allowed number of defects"* is hereby specified as being:

(i) up to two defects when each item within a batch of items has an assigned value equal to or less than 50 grams U-235;

(ii) no more than one defect when each item within a batch of items has an assigned value of less than 500 grams U-235, but one or more items has an assigned value in excess of 50 grams U-235; and

(iii) zero defect when any item within a batch of items contains 500 or more grams U-235.

SG-1.11 Notwithstanding the first paragraph of Section 7.1 of the Plan identified by Condition SG-1.1, the licensee shall conduct shipper-receiver comparisons on all SNM materials received (regardless of whether booked on the basis of receiver's or shipper's values), except for those materials identified in Section 7.1 of NUREG-1065 (Rev. 2) as being exempted from shipper-receiver comparisons.

SECTION 2.0 -- PHYSICAL PROTECTION OF SNM OF LOW STRATEGIC SIGNIFICANCE

SG-2.1 The licensee shall follow the "Site Physical Security Plan - Westinghouse Electric Corporation, Columbia, S.C.," dated March 1980; Revision 10.0 dated October 1, 1987; the "Revision Record" contained in the approved Physical Security Plan (current to October 1, 1994); and as may be revised in accordance with the provisions of 10 CFR 70.32(e).

SECTION 3.0 -- INTERNATIONAL SAFEGUARDS

SG-3.1 The licensee shall follow Codes 1 through 6 of Transitional Facility Attachment No. 5A dated August 31, 1988, to the US/IAEA Safeguards Agreement. Such Transitional Facility Attachment shall be interpreted in accordance with Conditions SG-3.1.1 through SG-3.1.7.

SG-3.1.1 With respect to Transitional Facility Attachment Code 2:

The reference design information is that dated by the licensee on October 14, 1985. "Information on the Facility" also includes other facility information submitted via Concise Notes in accordance with 10 CFR 75.11(c).

SG-3.1.2 With respect to Transitional Facility Attachment Code 2.2:

Substantive changes to the information provided in the Columbia Plant Design Information Questionnaire (DIQ) means those changes requiring amendment of the Transitional Facility Attachment. Such changes shall be provided by letter to the NRC Office of Nuclear Material Safety and Safeguards at least 70-days in advance of implementation.

Non-substantive changes to the information in the DIQ means those changes not requiring amendment of the Transitional Facility Attachment. Such changes shall be provided by Concise Note (From DOE/NRC-740M) within 30 days of receiving notification from the NRC that the facility has been identified under Article 39(b) of the US/IAEA Safeguards Agreement.

The types of modifications with respect to which information is required under 10 CFR 75.11, (to be submitted in advance), are those items stated in Code 2.2, specifically:

(a) "Any change in the purpose of type of facility" means:

Any deviation from the described activities involving special nuclear material and any change to the maximum enrichment and/or quantities of U-235 currently authorized by License No. SNM-1107, and/or as described in Paragraph 5 of the Design Information Questionnaire (DIQ) dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c). Included also is any deviation from the described special nuclear material (SNM) production activities described in paragraph 6 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

(b) "Any changes in the layout of the facility which affects safeguards implementation of the provisions of the Protocol" means:

Any change in the existing facility and/or site layout or new addition affecting any activity involving SNM as described in Paragraphs 10 and 11

(per the referenced attachments of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c). Included also is any modification to, or deviation from, the data provided in Paragraphs 13 and 14 (per the referenced attachments) of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

(c) "Any change that makes the selected Key Measurement Points (KMPs) (as described in Code 3.1.2) inadequate for the Agency's accounting purpose" means:

Any change to the KMPs as described in Code 3.1.2 of the Westinghouse-Columbia Transitional Facility Attachment to the US/IAEA Safeguards Agreement, or as modified in accordance with 10 CFR 75.11(c), that results in any KMP alteration affecting the purpose of KMPs as stipulated by 10 CFR 75.4(m)

(d) "Any change in the physical inventory procedures that would adversely affect the inventory taking for the Agency's accounting purposes" means:

Any change to the description data contained in Paragraph 34 (per the referenced attachments) of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c), that would not permit the Agency to conclude an SNM material balance for the Westinghouse-Columbia facility.

(e) "Introduction of a significantly less accurate analytical method for accounting purposes" means:

Any recalculation of the "Relative Errors-Random and Systematic" as listed in Attachment 36.2 referenced in Paragraph 36 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c), that results in the estimates of the random and systematic errors being affected by a factor of two or more.

(f) "Decrease in the frequency of calibrating measuring equipment if it significantly decreases the accuracy of the materials accounting system" means:

Any change that results in the estimates of the systematic error being affected by a factor of two or more.

(g) "Any change in the statistical procedures used to combine individual measurement error estimates to obtain limits of error for shipper/receiver (S/R) differences and material unaccounted for (MUF)" means:

Any deviation from (or modification of) the equations and/or calculations outlined in Attachments 37.1, 37.2, and 37.3 referenced in Paragraph 37 of the DIQ dated October 14, 1985, or as modified in accordance with 10 CFR 75.11(c).

SG-3.1.3 With respect to Transitional Facility Attachment Code 3.1.2:

KMP* -- This is a KMP in which all shipper receiver differences (SRDs) must be recorded and reported even if numerically zero. SRDs are computed and reported by the Nuclear Materials Management and Safeguards System upon receipt of the receiver's measurement results.

SG-3.1.4 With respect to Transitional Facility Attachment Code 4:

The licensee shall use the material composition codes documented in the DIQ dated October 14, 1985, and as modified by Concise Notes. Further, notwithstanding any other requirements for advance notification and/or reporting, the licensee may add or delete composition codes for nuclear material routinely processed and on inventory at CFFF immediately upon telephone notification to the Office of Nuclear Material Safety and Safeguards. Follow-up documentation, in the form of a Concise Note accompanied by appropriate changes to Table 1 of Attachment 34.8 to the DIQ shall be submitted within three regular workdays of the telephone notification.

SG-3.1.5 With respect to Transitional Facility Attachment Code 4.1:

Measured discards should be reported as an SN (Shipment to non-safeguards facility) when shipped off-site to an authorized burial ground. (The IAEA system will not process measured discards as loss/disposal (LDs) when they are shipped off-site).

SG-3.1.6 With respect to Transitional Facility Attachment Code 5.1.1:

For inventory changes, time of recording, "upon" means: No later than the next regular workday (Monday through Friday).

For those occasions where natural or depleted uranium is inadvertently enriched above 0.711 percent through commingling with residual enriched uranium in process equipment, the resultant product shall be considered as being produced through a blending operation and the material category change shall be recorded upon obtaining measurement confirmation that a material category change has occurred.

SG-3.1.7 With respect to Transitional Facility Attachment Code 6.2.2:

For Concise Notes describing the anticipated operational programme, "anticipated operational programme" means: Anticipated physical inventory schedule.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

December 9, 1997

DOCKET: 70-1151

LICENSEE: Westinghouse Electric Corporation
Columbia, SC

SUBJECT: SAFETY EVALUATION REPORT: APPLICATION DATED JULY 14, 1997, RELEASE OF INDUSTRIAL WASTE TREATMENT PRODUCTS TO CEMENT MANUFACTURERS

BACKGROUND

By letter dated July 14, 1997, Westinghouse Electric Corporation requested an amendment to License No. SNM-1107 to authorize the release of industrial waste treatment products (primarily calcium fluoride) to cement manufacturers. The July 14, 1997, submittal also requested withdrawal of the authorization to use up to 15 grams of ^{235}U for testing and demonstration purposes at off-site locations. The licensee indicated that this authorization was used infrequently and that future requirements for this type of activity could be handled more efficiently on a case-by-case basis.

By letter dated November 7, 1997, the NRC staff requested additional information on the licensee's proposal to release waste treatment products to cement manufacturers. The licensee responded by letter dated November 17, 1997, which included a dose assessment on the anticipated use of the material.

DISCUSSION

In 1985, the NRC authorized the licensee to dispose of industrial waste treatment products, in which the mean concentration of uranium does not exceed 30 pCi/g, in a chemical landfill. The NRC granted further authorization in 1987 to allow release of the material to be used in the production of steel. By letter dated July 14, 1997, the licensee also requested authorization to release the waste treatment products to cement manufacturers. The waste treatment products will be used as a filler and as a fluxing agent in the cement.

The licensee has committed to reducing the radioactive contents of all such transferred materials to levels as low as reasonably achievable below the 30 pCi/g limit. To ensure that the concentration of materials released is less than 30 pCi/g, the licensee has committed to implementation of a sampling plan to characterize the material transferred in accordance with

NUREG/CR-2082, "Monitoring for Compliance with Decommissioning Termination Survey Criteria."

The licensee has also committed to maintaining records pertaining to the release of the waste treatment products, including identities of receivers. In addition, the licensee will notify each receiver in writing of the uranium content by including the radionuclide concentration on the Material Safety Data Sheet for each batch of material.

The NRC staff performed a conservative dose assessment to estimate the potential risks to a worker from exposure to uranium during the manufacture of cement with the licensee's material. A conservative assessment was performed because, once the material is released from the licensee's control, the NRC can no longer place restrictions on its use to reduce potential doses.

The staff assumed that the worker is exposed to dust with an insoluble uranium concentration of 30 pCi/g for 40 hours per week, 50 weeks per year. The dust concentration in air was assumed to be 5 mg/m^3, which is the maximum exposure level (without a respirator) to nuisance dust of a respirable fraction established by the Occupational Health and Safety Administration in 29 CFR 1910.1000, Table Z-3. The worker's breathing rate was assumed to be 2.4×10^6 L/yr for a man engaged in light activity, taken from the "Report of the Task Group on Reference Man," International Commission on Radiation Protection (ICRP) No. 23, 1974.

This scenario results in a committed effective dose equivalent of 0.45 mSv/yr (45 mrem/yr) to the worker. Because the external dose to the worker is orders of magnitude less than the inhalation dose, the total effective dose equivalent for the worker does not exceed the NRC dose limit of 1 mSv/yr (100 mrem/yr) specified in 10 CFR 20.1301(a)(1) for a member of the public. As previously stated, the staff considers the assessment described above to be extremely conservative because it assumes that all of the worker's occupational dust inhalation in a year is composed entirely of the licensee's waste treatment products.

Other exposure scenarios were considered by the staff including (1) exposure to the cement user, (2) exposure during an accidental spill of waste treatment products during transport, and (3) external exposure during use of concrete products. The dose estimated for the worker during the manufacture of cement was determined to be bounding for all of these scenarios.

The cement containing waste treatment products will be provided by the cement manufacturer to a concrete manufacturer. The dose to a worker during the

manufacture of concrete is expected to be orders of magnitude less than the occupational dose during the manufacturer of cement due to the low concentration of uranium in the cement product. As previously stated, the waste treatment products will be used as a filler and as a fluxing agent in the cement, and the licensee expects that the waste treatment products will constitute a maximum of 0.25% of each cement batch. Inhalation exposure during an accident will also be significantly less, due to a relatively short exposure time. Finally, external exposure to a member of the public from finished concrete products was estimated to be 0.05 mSv/yr (5 mrem/yr) after ingrowth of the uranium daughters (up to radon), which is also significantly less than the dose to the worker during the manufacture of cement.

The staff's review of the amendment application has been coordinated with Rodney Wingard from the Bureau of Land Management/Division of Radioactive Waste Management of the South Carolina Department of Health and Environmental Control (DHEC). Although DHEC does not license Westinghouse for radioactive materials, the Westinghouse fuel fabrication facility is located in South Carolina, where DHEC has the responsibility to ensure the public is protected from radiological hazards in accordance with South Carolina Materials Regulations 61-63, Title A. DHEC considers the NRC the lead agency in this matter but provided assistance in this review.

DHEC provided an independent dose assessment using the computer code RESRAD. Using a resident farmer scenario and RESRAD default parameters, DHEC staff estimated that the dose to the maximally exposed individual will be less than 10 mrem/yr for the first 400 years. After 400 years, the dose will increase to approximately 28 mrem/yr at 1000 years due to the ingrowth of radon. The dose estimate remains below 10 mrem/yr if radon is excluded. Although the waste treatment products will be made into cement products and not directly applied to land, this scenario illustrates possible risks in the event that the material is spilled or accidentally disposed of on a land surface. In addition, the concrete products will eventually crumble leaving debris containing uranium in place.

On July 21, 1997, the NRC promulgated a new rule, 10 CFR Part 20, Subpart E, "Radiological Criteria for License Termination." This rule establishes a dose limit of 25 mrem/yr, excluding radon, for unrestricted release of sites. Although the rule is not applicable in this case, the above scenario does result in a dose which meets this limit. On October 16, 1997, Mr. Wingard indicated that DHEC had no objection to the NRC granting the licensee's amendment request.

ENVIRONMENTAL REVIEW

The staff has determined that the following conditions have been met:

1. There is no significant change in the types or significant increase in the amounts of any effluents that may be released offsite.

2. There is no significant increase in individual or cumulative occupational radiation exposure.

3. There is no significant construction impact.

4. There is no significant increase in potential for, or consequences from, radiological accidents.

Accordingly, pursuant to 10 CFR 51.22(c)(11), neither an environmental assessment nor an environmental impact statement is warranted for this action.

CONCLUSION

The staff has determined that release to cement manufacturers of industrial waste treatment products, such as calcium fluoride, which contain less than 30 pCi/g of uranium will not result in any adverse effect on public health and safety or the environment. In addition, the staff does not have any objection to withdrawal of the authorization to use up to 15 grams of ^{235}U for testing and demonstration purposes at off-site locations. Therefore, the staff recommends that the license amendment be granted.

The Region II project inspector has no objection to the proposed action.

Principal Contributor
Susan Chotoo