

Operated by Nuclear Management Company, LLC

January 19, 2006

10 CFR 50.73(a)(2)(i)(B) 10 CFR 50.73(a)(2)(v)(D)

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Palisades Nuclear Plant Docket 50-255 License No. DPR-20

<u>Licensee Event Report 05-007, Inoperable Emergency Diesel Generator For A Time</u>
<u>Longer Than Permitted By Technical Specifications</u>

Licensee Event Report (LER) 05-007 is attached. The LER describes the discovery of a component failure during surveillance testing of emergency diesel generator 1-2. This event is reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) and 10 CFR 50.73(a)(2)(v)(D).

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.

Paul A. Harden

Site Vice President, Palisades Nuclear Plant

Nuclear Management Company, LLC

Enclosure (1)

CC Administrator, Region III, USNRC
Project Manager, Palisades, USNRC
Resident Inspector, Palisades, USNRC

ENCLOSURE 1

LER 05-007, Inoperable Emergency Diesel Generator For A Time Longer Than Permitted By Technical Specifications

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TITLE (4)									-		SALA CONTRACTOR AND CONTRACTOR			
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МО	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	МО	DAY	YEAR	FACILITY NAME		DOCKET NUMBER			
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MODE				2201(b)		20.22	2203(a)(3)(ii) 50.73(a)(2)(ii)(B)		50.73(a)(2)(ii)(B)	50.73(a)(2)(ix)(A)				
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		20.2203(a)(2)(iii)				.46(a)(3)(ii)		50.73(a)(2)(v)(C)		Specify in Abstract below or in		ır in		
			20.2203(a)(2)(iv)		50.73).73(a)(2)(i)(A)		X	50.73(a)(2)(v)(D)	NRC Form 366A		эΑ		
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				LICE	NSEE	CONT	ACT FO	OR THIS L	.ER	(12)				

NAME TELEPHONE NUMBER (Include Area Code) Daniel G. Malone (269) 764-2463

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) MANU-REPORTABLE MANU-FA CTURER CAUSE SYSTEM

		В	EK	SNB	F010	Υ						
SUPPLEMENTAL REPORT EXPECTED (14)							EXPECTED		MONTH	DAY	YEAR	
X YES (If yes, complete EXPECTED SUBMISSION DATE).				NO	SUBMISSION DATE (15)		03	31	2006			

REPORTABLE

ABSTRACT

On November 20, 2005, during a monthly surveillance test of emergency diesel generator (EDG) 1-2, a fuel leak was observed from the fuel oil injection system at the discharge of the cylinder 5R fuel pump. Since the spraying fuel oil created a potential fire hazard, the EDG was shutdown and declared inoperable. Subsequent inspection determined that the leak occurred at the cylinder 5R snubber valve, which had cracked.

Review of the cause of failure indicates that the snubber valve was unable to operate satisfactorily for the EDG's required mission time. Therefore, EDG 1-2 should be considered to have been inoperable since installation of this snubber valve. Consequently, EDG 1-2 was inoperable for a period of time longer than allowed by Palisades Technical Specifications. Additionally, during the period EDG 1-2 was inoperable, EDG 1-1 was made inoperable for performance of its monthly surveillance. Therefore, both EDGs were simultaneously inoperable for a period of time longer than allowed by Technical Specifications.

This event is reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications, and 10 CFR 50.73(a)(2)(v)(D) as a condition that could have prevented fulfillment of the safety function of a system needed to mitigate the consequences of an accident. NRC FORM 366A (1-2001)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)		PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 of 3
Palisades	05000-255	2005	007	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION

On November 20, 2005, during a monthly surveillance test of emergency diesel generator (EDG) 1-2 [DG;EK], a fuel leak was observed from the fuel oil injection system at the discharge of the cylinder 5R fuel pump. Since the spraying fuel oil created a potential fire hazard, the EDG was shutdown and declared inoperable.

Subsequent inspection determined that the leak occurred at the cylinder 5R snubber valve, which had cracked. The snubber valve is located at the interface of the fuel injection pump and fuel injection tube. The snubber valve functions to dampen pulsations created by the fuel injection system and also serves as the fuel oil pressure boundary.

This snubber valve had been recently installed during EDG maintenance and had performed satisfactorily during post maintenance test runs on October 27 and 28, 2005, for a total of approximately 11 hours of EDG run time. Subsequently, on November 20, 2005, EDG 1-2 operated for approximately 1.5 hours during the monthly surveillance test before the leak was identified.

Review of the cause of failure indicates that the snubber valve was unable to operate satisfactorily for the EDG's required mission time. Therefore, EDG 1-2 should be considered to have been inoperable since installation of this snubber valve. Consequently, EDG 1-2 was inoperable for a period of time longer than the 7 days allowed by Palisades Technical Specification (TS) 3.8.1.B.4. Additionally, during the period EDG 1-2 was inoperable, EDG 1-1 was made inoperable for approximately 8 hours for the performance of its monthly surveillance. Therefore, both EDGs were simultaneously inoperable for a period of time longer than the 2 hours allowed by TS 3.8.1.E.1.

This event is reportable in accordance with 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by Technical Specifications, and 10 CFR 50.73(a)(2)(v)(D) as a condition that could have prevented the fulfillment of the safety function of a system needed to mitigate the consequences of an accident.

This event involves a safety system functional failure.

CAUSE OF THE EVENT

Examination of the snubber valve revealed that it had cracked axially in 3 separate locations. An additional crack was visible, but the fracture was not yet complete. The apparent cause of the cracking was due to improper heat treatment.

NRC FORM 366A (1-2001)

U.S. NUCLEAR REGULATORY COMMISSION

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The EDG vendor is continuing their investigation of the issue and evaluation of potential 10 CFR 21 reporting.

SAFETY SIGNIFICANCE

The safety significance of the event is considered minimal. For any postulated design basis scenario, the safety function of the EDGs is met with one operating EDG.

EDG 1-2 demonstrated that it would run for approximately 1.5 hours before the actual failure of the snubber valve. EDG 1-1 was operable during the subject period except for approximately 8 hours due to surveillance testing. EDG 1-1 is considered capable of being recovered to a fully operable condition from surveillance testing in a matter of minutes (much less than 1.5 hours).

Therefore, even if a postulated scenario requiring the EDG safety function occurred during EDG 1-1 surveillance testing, EDG 1-2 would have initially been in operation, with recovery of EDG 1-1 to operable status occurring before EDG 1-2 exhibited its failure.

CORRECTIVE ACTIONS

The cylinder 5R snubber valve was replaced. The EDG was satisfactorily test operated for approximately 25 hours. The issue with snubber valve cracking is considered an early life failure and would be expected to self reveal within a 24-hour run of the EDG.

PREVIOUS SIMILAR EVENTS

None