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January 5, 2006

U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Document Control Desk

Subject: Oconee Nuclear Station
Docket Numbers 50-269, 270 and 287
Implementation of Amendment Associated with New Reactor Building Emergency Sump (RBES) Strainers, TSC Number 2004-08

On November 1, 2005, the Nuclear Regulatory Commission (NRC) issued Amendment Nos. 348 and 350 to Renewed Facility Operating Licenses DPR-38 and DPR-47 for Oconee Nuclear Station (ONS), Units 1 and 2. The amendments consists of changes to Technical Specification (TS) in response to a Duke Energy Corporation (Duke) application dated August 18, 2005, as supplemented by letter dated September 15, 2006. Duke's License Amendment Request (LAR) requested the change for Unit 3, however, supporting calculations for the Unit 3 modification had not been completed prior to the need date for Unit 2. As a result, NRC issued the amendment for Units 1 and 2 only.

During implementation of this change, in order to retain the existing requirements for Unit 3 and implement the new requirements for Units 1 and 2, Duke created separate TS pages applicable to Units 1 and 2 (Amendment Nos. 348 and 350) and to Unit 3 (retained existing Amendment No. for Unit 3). The NRC (Leonard Olshan) agreed that this method of implementation was appropriate and strictly administrative. The NRC requested we submit the revised pages after implementation.

Attachment 1 provides the revised Technical Specification pages.

A001

If there are any additional questions, please contact Boyd Shingleton at (864) 885-4716.

Very truly yours,

Bruce Hamilton

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Oconee Nuclear Site

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January 5, 2006

ATTACHMENT 1

TECHNICAL SPECIFICATION

Remove Page	Insert Page
3.5.2-5	3.5.2-5a
	3.5.2-5b
3.5.3-3	3.5.3-3a
	3.5.3-3b

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.5.2.3	Verify each HPI pump's developed head at the test flow point is greater than or equal to the required developed head.	In accordance with the Inservice Testing Program
SR 3.5.2.4	Verify each HPI automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	18 months
SR 3.5.2.5	Verify each HPI pump starts automatically on an actual or simulated actuation signal.	18 months
SR 3.5.2.6	Verify, by visual inspection, each HPI train reactor building sump suction inlet is not restricted by debris and suction inlet strainers show no evidence of structural distress or abnormal corrosion.	18 months
SR 3.5.2.7	Cycle each HPI discharge crossover valve and LPI-HPI flow path discharge valve.	18 months

SURVEILLANCE REQUIREMENTS (continued)

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SR 3.5.2.6	Verify, by visual inspection, each HPI train reactor building sump suction inlet is not restricted by debris and suction inlet trash racks and screens show no evidence of structural distress or abnormal corrosion.	18 months
SR 3.5.2.7	Cycle each HPI discharge crossover valve and LPI-HPI flow path discharge valve.	18 months

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.5.3.2	----------NOTE---------- Not applicable to operating LPI pump(s). ---------- Vent each LPI pump casing.	31 days
SR 3.5.3.3	Verify each LPI pump's developed head at the test flow point is greater than or equal to the required developed head.	In accordance with the Inservice Testing Program
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