



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

January 5, 2006

Duke Energy Corporation
ATTN: Mr. G. R. Peterson
Vice President
McGuire Nuclear Station
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: NOTIFICATION OF MCGUIRE NUCLEAR STATION COMPONENT DESIGN
BASES INSPECTION-NRC INSPECTION REPORT 05000369/2006007 AND
05000370/2006007

Dear Mr. Peterson:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your McGuire Nuclear Station during the weeks of March 20-24, 2006, April 3-7, 2006, and April 17-21, 2006. The inspection team will be led by Larry Mellen, a Senior Reactor Inspector from the NRC's Region II Office. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued December 2, 2005.

The inspection will evaluate the capability of risk significant / low margin components to function as designed and support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on January 4, 2006, I confirmed with Ms. K. Crane of your staff, arrangements for an information gathering site visit and the three-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of March 6, 2006.
- Onsite weeks: March 20, 2006; April 3, 2006; and April 17, 2006.

The purpose of the information gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. Rudy Bernhard, a Region II Senior Reactor Analyst, may accompany Mr. Mellen and the inspection team during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection.

The enclosure lists documents that will be needed prior to the information gathering visit. Please contact Mr. Mellen prior to preparing copies of the materials listed in the enclosure.

The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for the inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space, supplemental documents requested to be made available to the team in the Region II office prior to the inspection preparation week of March 13, 2006; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Mellen at (404) 562-4531 or me at (404) 562-4605.

Sincerely,

/RA/

Charles R. Ogle, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-369, 50-370
License Nos.: NPF-9, NPF-17

Enclosure
cc w/enc/: (See page 3)

cc w/encl:

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INFORMATION REQUEST FOR MC GUIRE NUCLEAR STATION COMPONENT DESIGN BASES INSPECTION

Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Information in “lists” should contain enough information to be easily understood to someone who has a knowledge of pressurized water reactor technology.

1. Risk ranking of components from your site specific probabilistic safety analysis (PSA) sorted by Risk Achievement Worth (RAW) and sorted separately by Birnbaum Importance. If possible, please provide a correlation between actual plant components in plain language, and RAW values.
2. Provide a list of the top 500 cutsets from your PSA.
3. Risk ranking of operator actions from your site specific PSA sorted by RAW. Provide copies of your human reliability worksheets for these items.
4. If you have an External Event or Fire PSA Model, provide the information requested in Items 1 and 2 for external events and fire.
5. Any pre-existing evaluation or list of components and calculations with low design margins (i.e., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, MOV risk-margin rankings, etc.).
6. A list of the last two years of operating experience evaluations, modifications, and corrective actions sorted by component or system.
7. Information of any common cause failure of components experienced in the last 5 years at your facility.

Enclosure