

January 13, 2006

Mr. Karl W. Singer
Chief Nuclear Officer and
Executive Vice President
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT, UNIT 1 — REQUEST FOR ADDITIONAL
INFORMATION REGARDING THE CYCLE 13 STEAM GENERATOR TUBE
INSERVICE INSPECTIONS (TAC NO. MC8712)

Dear Mr. Singer:

By letter dated November 22, 2004, the Tennessee Valley Authority (TVA, the licensee) submitted the steam generator (SG) tube plugging report in accordance with Technical Specification (TS) Section 4.4.5.5.a. By letter dated February 15, 2005, TVA submitted the 90-day inservice inspection summary report in accordance with the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Section XI, Article IWA-6230. By letter dated October 20, 2005, TVA submitted the 12-month SG inspection report in accordance with TS Section 4.4.5.5.b. The Cycle 13 SG tube inservice inspections were the first SG tube inspections since SG replacement.

In order for the staff to complete its review of the reports, we request that the licensee provide responses to the enclosed request for additional information (RAI). Based on discussions with your staff, we understand that you intend to respond to this RAI within 60 days of receipt of this letter.

Sincerely,

/RA/

Douglas V. Pickett, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-327

Enclosure: Request for Additional Information

cc w/enclosure: See next page

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Mr. Karl W. Singer
Tennessee Valley Authority

SEQUOYAH NUCLEAR PLANT

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REQUEST FOR ADDITIONAL INFORMATION

SEQUOYAH NUCLEAR PLANT, UNIT 1

CYCLE 13 STEAM GENERATOR TUBE INSERVICE INSPECTIONS

DOCKET NO. 50-327

Questions Regarding the 12-Month Steam Generator (SG) Tube Inservice Inspection Report

1. Your report indicated that 100-percent of the dents and dings greater than or equal to 2-volts between the top of the hot leg tubesheet and the top of the cold leg tubesheet were examined with a +Point™ probe. Discuss the results of these +Point™ probe inspections, including the number of dents/dings in each SG. In addition, discuss if there were any new dents/dings or any “anomalous” dent signals.
2. Please indicate if the 11 wear indications in the area of the 2nd and 4th vertical straps were inspected with bobbin and +Point™ probes. If the wear indications were not inspected with a +Point™ probe, please discuss how you concluded that the indications were wear. In addition, the staff noted that several of the wear indications were located in the same general vicinity. Please provide any insights on the pattern of the wear indications.
3. Your report provided information about flaws in specific tubes and at specific locations within a tube. In order for the U.S. Nuclear Regulatory Commission staff to better understand where your indications are being detected and for future reference, please provide the following information regarding the design of your replacement SGs: a tubesheet map, a tube support plate diagram, the heat transfer surface area, the tubesheet thickness (with and without the clad), the lattice grid thickness, the tube support plate thickness, and a description of the U-bend support system, including the thickness and the tubing supported by the various supports. In addition, discuss whether you have a flow distribution baffle to distribute flow at the top of the tubesheet, including the thickness.

ENCLOSURE