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December 21, 2005

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Subject: Duke Energy Corporation
Catawba Nuclear Station, Units 1 and 2
Docket Nos. 50-413 and 50-414
Technical Specification Bases Changes

Pursuant to 10CFR 50.4, please find attached changes to the Catawba Nuclear Station Technical Specification Bases. These Bases changes were made according to the provisions of 10CFR 50.59.

Any questions regarding this information should be directed to L. J. Rudy, Regulatory Compliance, at (803) 831-3084.

I certify that I am a duly authorized officer of Duke Energy Corporation and that the information contained herein accurately represents changes made to the Technical Specification Bases since the previous submittal.

Dhiaa M. Jamil

Attachment

A001

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December 21, 2005

Re: Catawba Nuclear Station
Technical Specifications Bases

Please replace the corresponding pages in your copy of the Catawba Technical Specifications Manual as follows:

REMOVE THESE PAGES	INSERT THESE PAGES
LIST OF EFFECTIVE PAGES	
Page 24	Page 24
TAB 3.6.3	
B 3.6.3-13 – **B** 3.6.3-14	**B** 3.6.3-13 – **B** 3.6.3-14

If you have any questions concerning the contents of this Technical Specification update, contact Debbie Rome at (803)831-3067.

A. Jones-Young
for

Randy Hart
Manager, Regulatory Compliance

Page Number	Amendment	Revision Date
B 3.6.1-5	Revision 1	7/31/01
B 3.6.2-1	Revision 0	9/30/98
B 3.6.2-2	Revision 1	7/31/01
B 3.6.2-3	Revision 1	7/31/01
B 3.6.2-4	Revision 1	7/31/01
B 3.6.2-5	Revision 1	7/31/01
B 3.6.2-6	Revision 1	7/31/01
B 3.6.2-7	Revision 1	7/31/01
B 3.6.2-8	Revision 1	7/31/01
B 3.6.3-1	Revision 1	3/20/02
B 3.6.3-2	Revision 1	3/20/02
B 3.6.3-3	Revision 1	3/20/02
B 3.6.3-4	Revision 1	3/20/02
B 3.6.3-5	Revision 0	9/30/98
B 3.6.3-6	Revision 0	9/30/98
B 3.6.3-7	Revision 0	9/30/98
B 3.6.3-8	Revision 0	9/30/98
B 3.6.3-9	Revision 0	9/30/98
B 3.6.3-10	Revision 0	9/30/98
B 3.6.3-11	Revision 0	9/30/98
B 3.6.3-12	Revision 0	9/30/98
B 3.6.3-13	Revision 3	12/05/05
B 3.6.3-14	Revision 1	7/31/01
B 3.6.3-15	Revision 0	9/30/98
B 3.6.4-1	Revision 0	9/30/98
B 3.6.4-2	Revision 1	2/26/01
B 3.6.4-3	Revision 0	9/30/98
B 3.6.4-4	Revision 0	9/30/98
B 3.6.5-1	Revision 0	9/30/98
B 3.6.5-2	Revision 1	4/26/00
B 3.6.5-3	Revision 1	4/26/00
B 3.6.5-4	Revision 0	9/30/98
B 3.6.6-1	Revision 0	9/30/98

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SURVEILLANCE REQUIREMENTS (continued)

SR 3.6.3.5

Verifying that the isolation time of each automatic power operated containment isolation valve is within limits is required to demonstrate OPERABILITY. The isolation time test ensures the valve will isolate in a time period less than or equal to that assumed in the safety analyses. The isolation time is specified in the UFSAR and the Frequency of this SR is in accordance with the Inservice Testing Program.

SR 3.6.3.6

For the Containment Purge System valves with resilient seals, additional leakage rate testing beyond the test requirements of 10 CFR 50, Appendix J, Option B is required to ensure OPERABILITY. The measured leakage rate for the Containment Purge System and Hydrogen Purge System valves must be $\leq 0.05\ L_a$ when pressurized to P_a. Operating experience has demonstrated that this type of seal has the potential to degrade in a shorter time period than other seal types. Based on this observation and the importance of maintaining this penetration leak tight (due to the direct path between containment and the environment), these valves will not be placed on the maximum extended test interval. Therefore, these valves will be tested in accordance with Regulatory Guide 1.163, which allows a maximum test interval of 30 months.

The Containment Air Release and Addition System valves have a demonstrated history of acceptable leakage. The measured leakage rate for containment air release and addition valves must be $\leq 0.01\ L_a$ when pressurized to P_a. These valves will be tested in accordance with Regulatory Guide 1.163, which allows a maximum test interval of 30 months.

SR 3.6.3.7

Automatic containment isolation valves close on a containment isolation signal to prevent leakage of radioactive material from containment following a DBA. This SR ensures that each automatic containment isolation valve will actuate to its isolation position on a containment

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SURVEILLANCE REQUIREMENTS (continued)

isolation signal. The isolation signals involved are Phase A, Phase B, and Safety Injection. This surveillance is not required for valves that are locked, sealed, or otherwise secured in the required position under administrative controls. The 18 month Frequency is based on the need to perform this Surveillance under the conditions that apply during a plant outage and the potential for an unplanned transient if the Surveillance were performed with the reactor at power. Operating experience has shown that these components usually pass this Surveillance when performed at the 18 month Frequency. Therefore, the Frequency was concluded to be acceptable from a reliability standpoint.

SR 3.6.3.8

This SR ensures that the combined leakage rate of all reactor building bypass leakage paths is less than or equal to the specified leakage rate. This provides assurance that the assumptions in the safety analysis are met. The Frequency is required by the Containment Leakage Rate Testing Program. This SR simply imposes additional acceptance criteria.

Bypass leakage is considered part of L_a.

REFERENCES

1. UFSAR, Section 15.
2. 10 CFR 50.36, Technical Specifications, (c)(2)(ii).
3. UFSAR, Section 6.2.
4. Standard Review Plan 6.2.4.
5. Generic Issue B-24.