

Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road
Waterford, CT 06385



Dominion™

DEC 13 2005

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No. 05-804
MPS Lic/MAE R0
Docket No. 50-423
License No. NPF-49

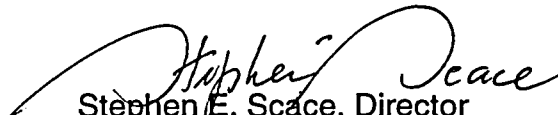
DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION UNIT 3
STARTUP TEST REPORT FOR CYCLE 11

Pursuant to Section 6.9.1.1 of the Millstone Unit 3 Technical Specifications, Dominion Nuclear Connecticut, Inc. hereby submits the enclosed Startup Test Report for Cycle 11.

There are no regulatory commitments contained within this letter.

If you have any questions or require additional information, please contact Mr. David W. Dodson at (860) 447-1791, extension 2346.

Very truly yours,


Stephen E. Scace, Director
Nuclear Station Safety and Licensing

JE26

Enclosures: (1)

Commitments made in this letter: None.

cc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406-1415

Mr. V. Nerses
Senior Project Manager
U.S. Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike
Mail Stop 8C2
Rockville, MD 20852-2738

Mr. S. M. Schneider
NRC Senior Resident Inspector
Millstone Power Station

Serial No. 05-804
Docket No. 50-423

Enclosure 1

Startup Test Report

Cycle 11

**Millstone Power Station 3
Dominion Nuclear Connecticut, Inc. (DNC)**

Table of Contents

	<u>Page</u>
1.0 SUMMARY	2
2.0 INTRODUCTION	2
3.0 FUEL DESIGN	3
4.0 LOW POWER PHYSICS TESTING	3
4.1 Critical Boron Concentration	3
4.2 Moderator Temperature Coefficient	4
4.3 Control Rod Reactivity Worth Measurements	4
5.0 POWER ASCENSION TESTING	5
5.1 Power Distribution, Power Peaking and Tilt Measurements ...	5
5.2 Boron Measurements	7
5.3 Reactor Coolant System Flow Measurement	7
6.0 REFERENCES	8
7.0 FIGURES	8

1.0 SUMMARY

Low Power Physics Testing and Power Ascension Testing for Millstone Unit 3 Cycle 11 identified no unusual core response or reactivity anomalies. All measured core parameters were determined to be within their acceptance criteria. All Technical Specification surveillance requirements were met.

2.0 INTRODUCTION

The Millstone Unit 3 Cycle 11 fuel reload was completed on October 14, 2005. The attached core map (Figure 1) shows the final core configuration. Reference 6.3 documents that Cycle 11 uses a low leakage loading pattern (L3P) consisting of 73 new Region 13 fuel assemblies, 72 Region 12 once-burned fuel assemblies, and 48 Region 11 twice-burned fuel assemblies. The 73 feed fuel assemblies, 64 of the 72 once-burned fuel assemblies and all 48 twice-burned assemblies are the Westinghouse 17x17 Robust Fuel Assembly (RFA) design. Eight of the once-burned fuel assemblies are Westinghouse 17x17 Next Generation Fuel (NGF) Lead Test Assemblies (LTAs).

The 73 Region 11 assemblies are comprised of 37 assemblies enriched to 4.00 weight percent Uranium-235 (w/o U^{235}) and 36 assemblies enriched to 4.95 w/o U^{235} . The top and bottom regions of all fuel assemblies in the Cycle 11 core are comprised of a 6-inch annular blanket region enriched to 2.6 w/o U^{235} . The fuel assembly locations for the fresh fuel were randomly assigned to prevent power tilts across the core due to systematic deviations in the fresh fuel composition.

Every fuel assembly in Cycle 11 contains an insert from the following list of items:

2 secondary sources, 61 RCCAs, and 130 thimble plugs.

Subsequent operational and testing milestones were completed as follows:

Initial Criticality	October 27, 2005
Low Power Physics Testing completed	October 27, 2005
Main Turbine Online	October 27, 2005
30% Power Testing completed on	October 28, 2005
75% Power Testing completed on	October 30, 2005
100% Power Testing completed on	November 07, 2005

3.0 FUEL DESIGN

The Robust Fuel Assembly (RFA) design comprises 185 out of the 193 assemblies in the Cycle 11 core. This fuel design differs from the previous fuel design in that it incorporates the Westinghouse protective bottom grid (P-Grid), thicker walled control rod guide tubes and instrument tube, and modifications to the mixing vane grids and Intermediate Flow Mixer (IFM) grids. The P-Grid improves the fuel assembly's resistance to debris and thus debris related failures. The thicker walled guide and instrument tubes make the fuel assembly more resistant to bowing and twisting, thereby further reducing the possibility of an incomplete rod insertion event. The modifications to the mixing vanes grids and IFM's improve the fuel assembly thermal performance and increase the margin to fuel-related design limits.

The final 8 assemblies in the Cycle 11 core are Next Generation Fuel (NGF) Lead Test Assemblies (LTA's). These LTAs, designated Region 12C, have several mechanical differences from the RFA assemblies. The LTAs have an Integral Top Nozzle, enhanced structural and IFM grids, two additional IFM grids per assembly, and utilize a tube-in-tube design for the guide tubes. The LTAs also have reduced pressure drop Debris Filter Bottom Nozzles (DFBNs), optimized ZITLO™ cladding, and have had the plenum spring used on an RFA replaced by a spring clip.

4.0 LOW POWER PHYSICS TESTING

The low power physics testing program for Cycle 11 was completed using the procedure in reference 6.1 based on the Westinghouse Dynamic Rod Worth Measurement (DRWM) Technique described in reference 6.4. This program consisted of the following: Control and Shutdown Bank Worth measurements, Critical Boron Endpoint measurements for All Rods Out (ARO), and ARO Moderator/Isothermal Temperature Coefficient measurements. Low power physics testing was performed at a power level below the point of nuclear heat to avoid nuclear heating reactivity feedback effects.

4.1 Critical Boron Concentration

The critical boron concentration was measured for the All Rods Out configuration. The measured values include corrections to account for differences between the measured critical rod configuration and the ARO configuration. The review and acceptance criteria of ± 500 and ± 1000 percent milliRho (pcm) respectively were met for the ARO configuration.

Summary of Boron Endpoint Results

	Measured (ppm)	Predicted (ppm)	M-P (ppm)	Acceptance Criteria (pcm)
All Rods Out (ARO)	2160	2156	4 (23 pcm)	± 1000

4.2 Moderator Temperature Coefficient

Isothermal Temperature Coefficient (ITC) data was measured at the All Rods Out configuration. Controlled heat-ups and cool-downs were performed and the reactivity change was measured. These measurements were corrected for ARO conditions and the averages of the corrected results are presented below. They were then compared to the design predictions and review criteria. The review criteria of ± 2 pcm/ $^{\circ}$ F of the predictions were met.

The ARO Moderator Temperature Coefficient (MTC) of -0.01 pcm/ $^{\circ}$ F was calculated by subtracting the design Doppler Temperature Coefficient (-1.78 pcm/ $^{\circ}$ F) from the measured ARO Isothermal Temperature Coefficient of -1.79 pcm/ $^{\circ}$ F. The Technical Specification Limit of MTC $< +5.0$ pcm/ $^{\circ}$ F at ARO Hot Zero Power (HZP) was met.

Isothermal/Moderator Temperature Coefficient Results

	Measured (pcm/ $^{\circ}$ F)	Corrected Predicted (pcm/ $^{\circ}$ F)	M-P (pcm/ $^{\circ}$ F)	Acceptance Criteria (pcm/ $^{\circ}$ F)
ARO ITC	-1.79	-2.18	+0.39	NA
ARO MTC	-0.01	NA	NA	MTC $< +5.0$

4.3 Control Rod Reactivity Worth Measurements

The integral reactivity worths of all RCCA Control and Shutdown Banks were measured using the Dynamic Rod Worth Measurement Technique (DRWM). The review criteria is that the measured worth is $\pm 15\%$ or 100 pcm of the individual predicted worth, whichever is greater and sum of the measured worths is $\pm 8\%$ of the predicted worths. The DRWM rod worth acceptance criteria is defined as: the sum of the measured worths (M) of all banks shall be greater than or equal to 90% of the sum of their predicted worths (P).

Control Bank Integral Worth Results

	Measured (pcm)	Predicted (pcm)	M-P (pcm)	% Difference (M-P) / P
Control Bank A	828.0	836.3	-8.3	-1.0
Control Bank B	606.5	584.8	21.7	3.7
Control Bank C	860.2	870.2	-10.0	-1.1
Control Bank D	505.2	481.7	23.5	4.9
Shutdown Bank A	342.9	341.2	1.7	0.5
Shutdown Bank B	1071.0	1047.4	23.6	2.3
Shutdown Bank C	322.4	309.0	13.4	4.3
Shutdown Bank D	330.0	313.9	16.1	5.1
Shutdown Bank E	59.0	56.4	2.6	4.6
Totals.	4925.2	4840.9	84.3	1.7

The measured results of the individual bank worths and the total control bank worth showed excellent agreement with the predicted values. All individual and total worth review criteria were met. The acceptance criteria for sum of the measured rod worths (greater than or equal to 90% of the sum of the predicted worths) was met.

5.0 POWER ASCENSION TESTING

5.1 Power Distribution, Power Peaking and Tilt Measurements

The core power distribution was measured through the performance of a series of flux maps during the power ascension as specified in reference 6.2. The results from the flux maps were used to verify compliance with the power distribution Technical Specifications.

A low power flux map, at approximately 29% rated thermal power (RTP), was performed to determine if any gross neutron flux abnormalities existed. At the 30% power plateau flux map, data necessary to perform an INCORE to EXCORE calibration via the single point methodology was obtained. Per Technical Specification Surveillance 4.3.1.1, Table 4.3-1 Functional Unit 2 Note 6, a flux map at approximately 98% power was performed for INCORE to

EXCORE calibration. Once hot full power equilibrium conditions were reached, another flux map was performed to verify core power distributions were within the design limits.

A summary of the Measured Axial Flux Difference (AFD) and INCORE Tilt for the flux maps performed during the power ascension is provided below. Additional tables provide comparisons of the most limiting measured Heat Flux Hot Channel Factor (F_Q) and Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta h}$), including uncertainties, to their respective limits from each of the flux maps performed during the power ascension. The most limiting F_Q is based on margin to the limit which varies as a function of core height.

As can be seen from the data presented, all Technical Specification limits were met and no abnormalities in core power distribution were observed during power ascension.

Summary of Measured Axial Flux Difference and INCORE Tilt

Power (%RTP)	Burnup (MWD/MTU)	Rod Position (steps)	AFD (%)	INCORE Tilt
29.2	9.0	216	5.027	1.0022
72.0	34.0	216	4.568	1.0021
100.0	343.0	216	1.027	1.0029

Comparison of Measured F_Q to F_Q^{RTP} limit

Power (%RTP)	Burnup (MWD/MTU)	Measured F_Q	F_Q^{RTP} steady state limit	Margin to Transient Limit
29.2	9.0	2.156	4.973	N/A
72.0	34.0	1.890	3.491	32.2 %
100.0	343.0	1.860	2.600	14.1 %

Comparison of Measured $F_{\Delta h}$ to $F_{\Delta h}$ limit for each Fuel Type

Power (%RTP)	Burnup (MWD/MTU)	Type 1 (NGF)	Type 1 Limit	Type 2 (RFA)	Type 2 Limit
29.2	9.0	1.413	1.827	1.492	1.912
72.0	34.0	1.391	1.637	1.474	1.713
100.0	343.0	1.362	1.510	1.447	1.580

Presented in Figures 2, 3 and 4 are measured Power Distribution Maps showing percent difference from the predicted power for the 30%, 75% and 100% power plateaus. From these data it can be seen that there is good agreement between the measured and predicted assembly powers.

5.2 Boron Measurements

Hot full power all rods out boron concentration measurements were performed after reaching equilibrium conditions. The measured All Rods Out, Hot Full Power, equilibrium xenon, boron concentration was 1496 ppm with a predicted value of 1478 ppm. The predicted to measured difference was +106 pcm which met the acceptance criteria of ± 1000 pcm.

5.3 Reactor Coolant System Flow Measurement

The Reactor Coolant Flow rate was determined using a secondary calorimetric heat balance for each loop using the steam generators as the control volumes. The following parameters were measured:

- Reactor Coolant System Pressure
- Hot Leg Temperatures
- Cold Leg Temperatures
- Feedwater Temperatures
- Feedwater Flow Rates
- Feedwater Pressure
- Steam Generator Pressure

Steam generator blowdown was not isolated during the data acquisition period.

Per Technical Specification Surveillance 4.2.3.1.2, the Reactor Coolant System Flow was measured prior to operation above 75% rated thermal power. The measured flow at approximately 72% rated thermal power was 400,244 gallons per minute (gpm) with a minimum required flow of 372,292 gpm. The reactor coolant system flow measurement was re-performed after reaching 100% rated thermal power. The measured flow at 100% power was

398,401 gpm with a minimum required flow of 372,292 gpm. All Technical Specification limits were met.

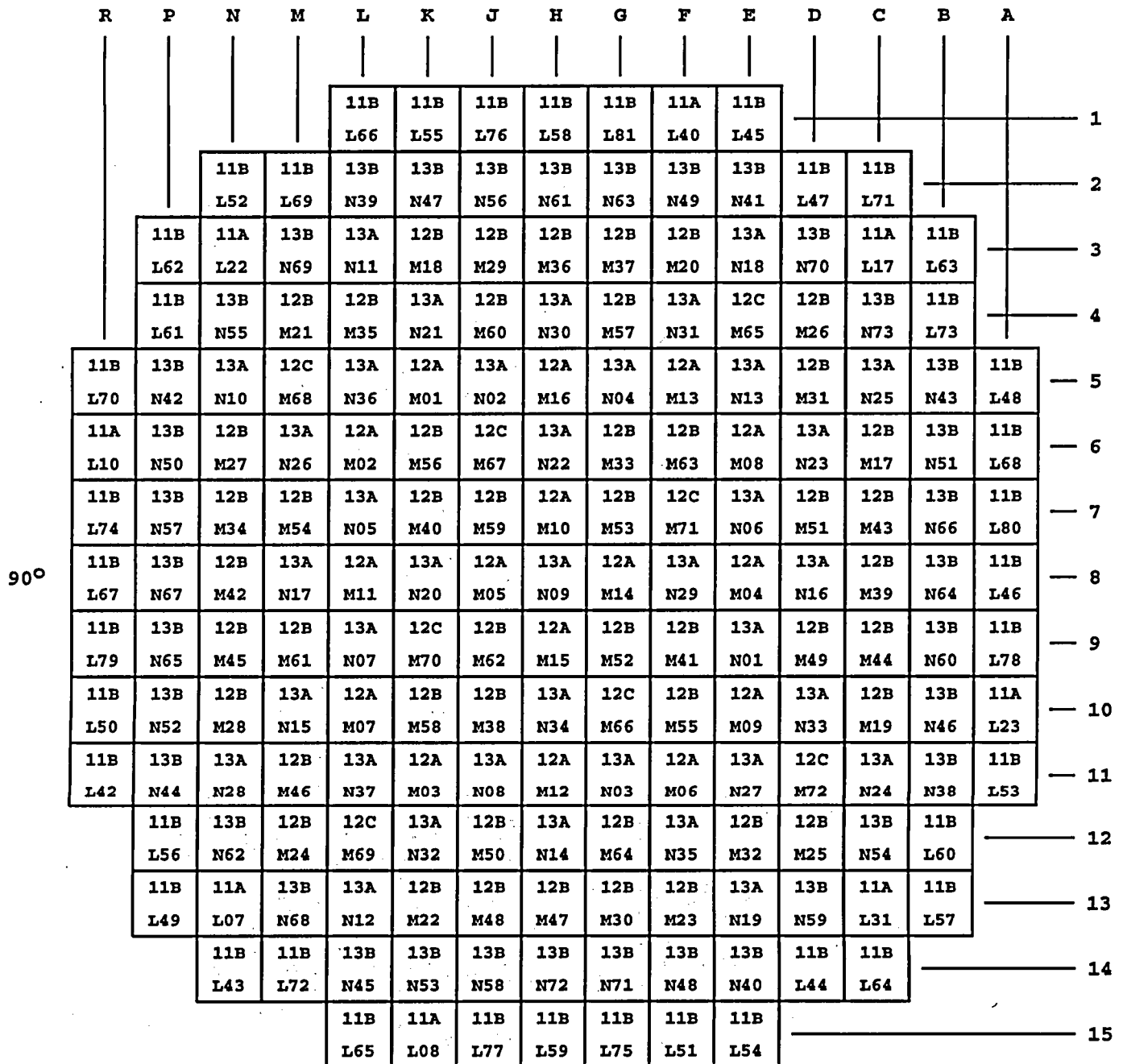
6.0 REFERENCES

- 6.1 SP 31008, Rev. 002-06, "Low Power Physics Testing (IPTE)"
- 6.2 EN 31015, Rev. 001-00, "Power Ascension Testing of Millstone Unit 3"
- 6.3 Nuclear Design and Core Physics Characteristics of the Millstone Generating Station Unit 3, Cycle 11
- 6.4 WCAP-13360-P-A, Revision 1, "Westinghouse Dynamic Rod Worth Measurement Technique"
- 6.5 NEU-05-36, Letter from W. F. Staley (Westinghouse) to Robert Borchert, "Dominion Nuclear Connecticut Millstone Unit 3 Low Power Physics Tests (LPPT)," dated November 11, 2005.

7.0 FIGURES

	<u>Page</u>
1 Cycle 11 Loading Plan	9
2 INCORE Power Distribution - 29%	10
3 INCORE Power Distribution - 72%	11
4 INCORE Power Distribution - 100%	12

FIGURE 1
CORE LOADING PATTERN
MILLSTONE UNIT 3 - CYCLE 11



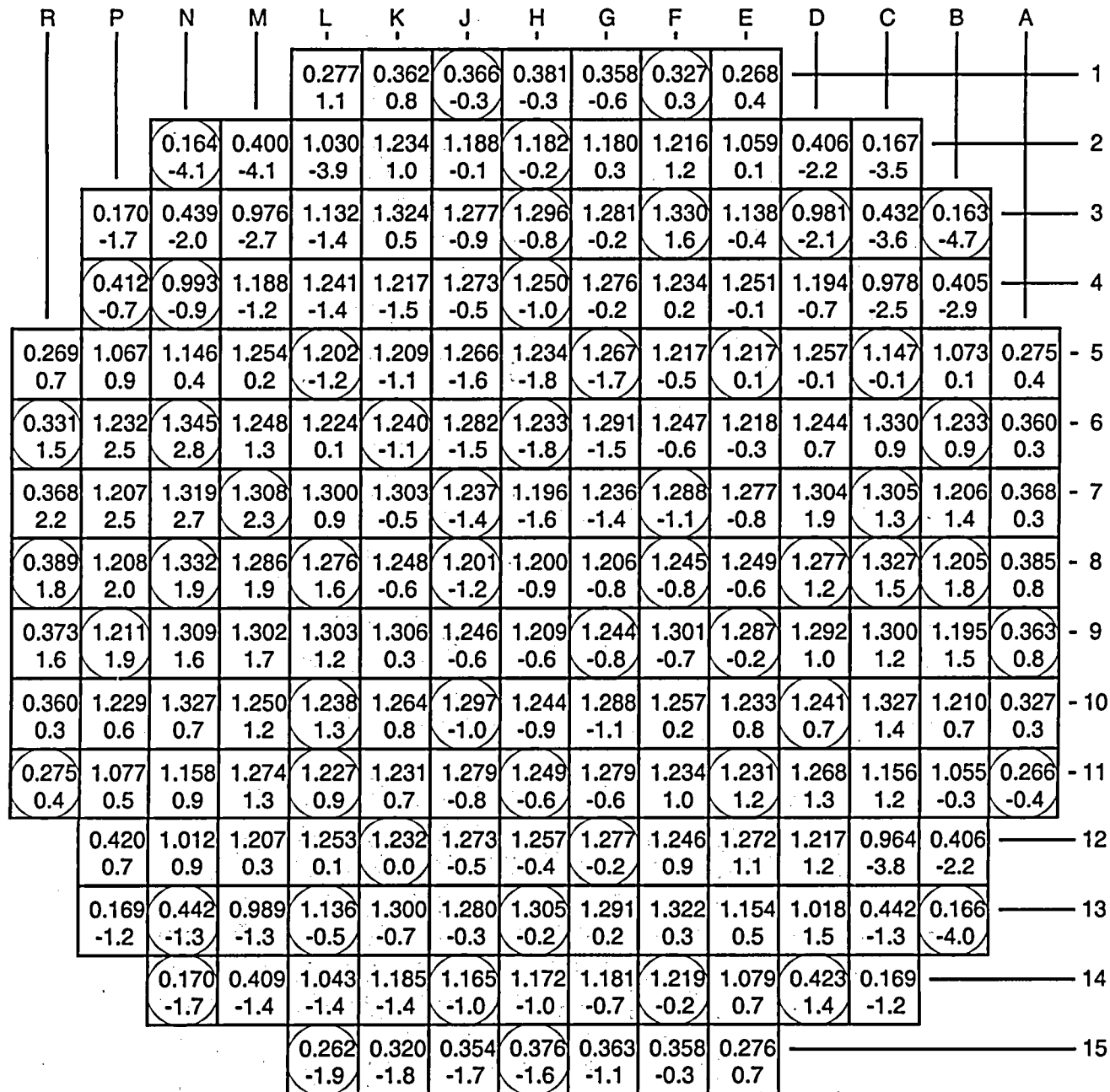
LEGEND

R	Region Identifier
ID	Fuel Assembly Identifier

REGION ASSEMBLIES ENRICHMENT

11A	8	4.20
11B	40	4.70
12A	16	4.70
12B	48	4.95
12C	8	4.95
13A	37	4.00
13B	36	4.95

FIGURE 2
INCORE Power Distribution - 29%
MILLSTONE UNIT 3 - CYCLE 11





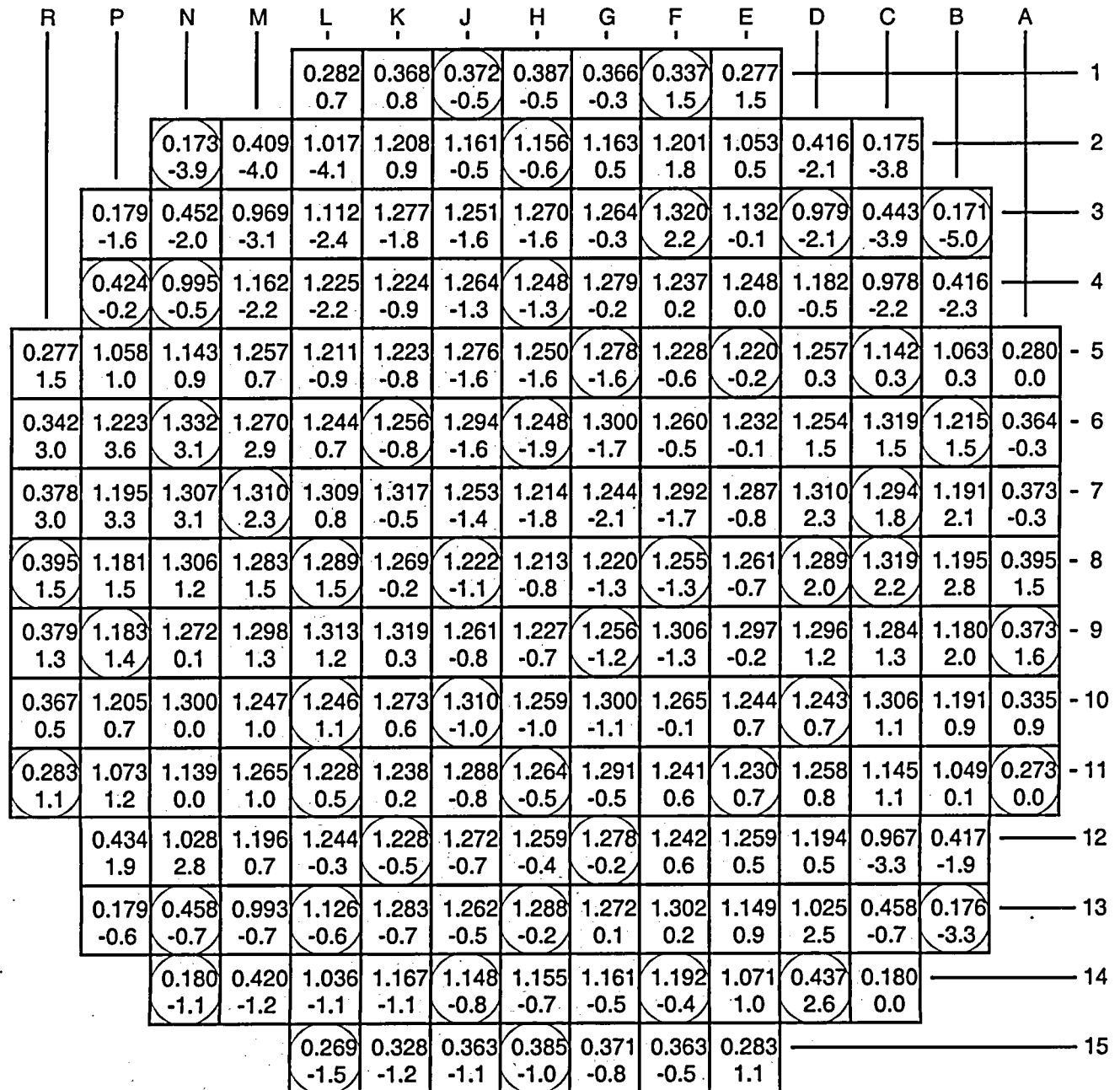
 Measured Power
 % Difference (M-P)/P
 Measured Location

FIGURE 3
INCORE Power Distribution - 72%
MILLSTONE UNIT 3 - CYCLE 11





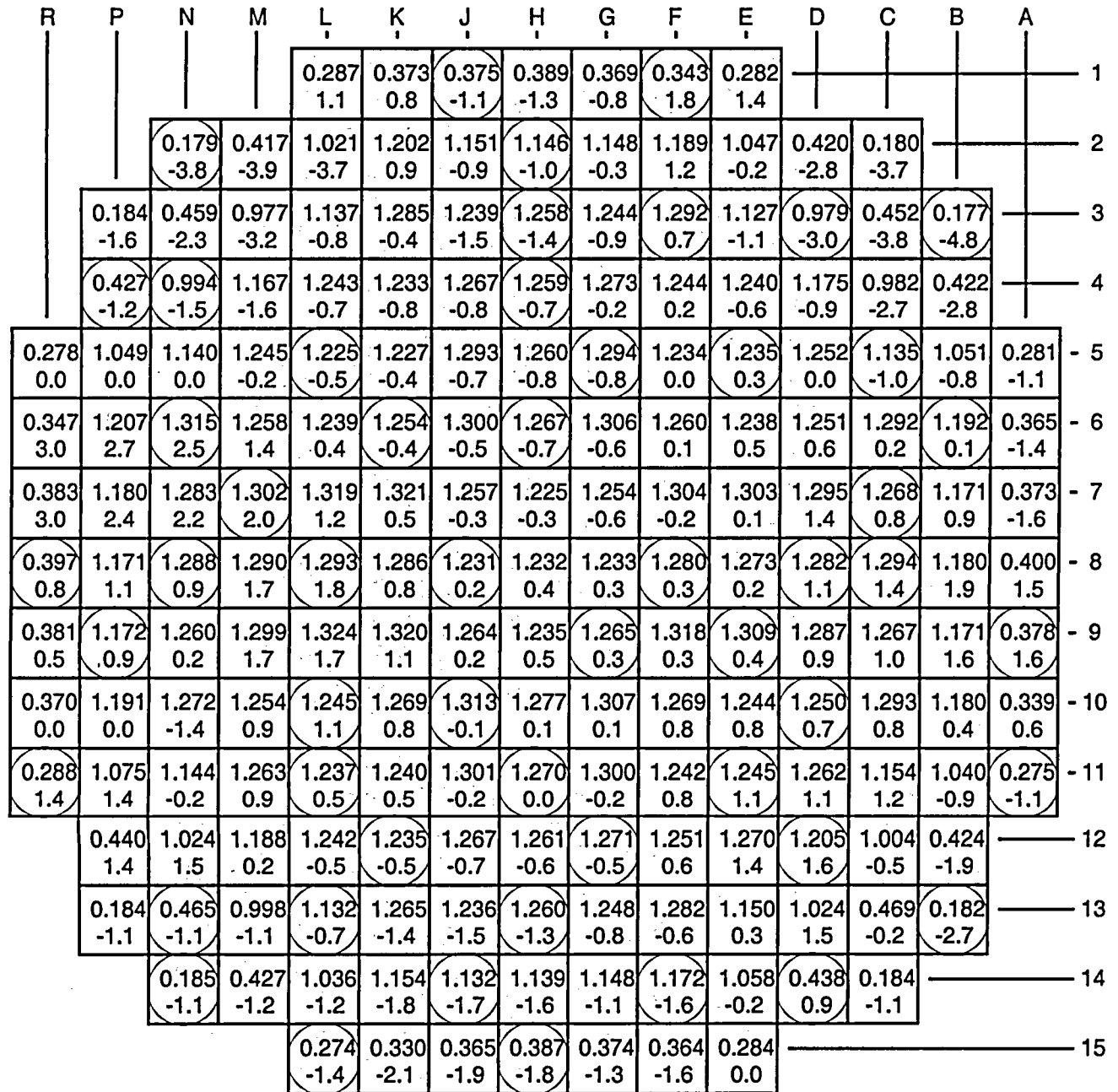


 Measured Power
 % Difference (M-P)/P
 Measured Location

FIGURE 4
INCORE Power Distribution - 100%
MILLSTONE UNIT 3 - CYCLE 11



 Measured Power
 % Difference (M-P)/P
 Measured Location