

G. ENO shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822), and to the authority of 10 CFR 50.90 and CFR 50.54(p). The combined set of plans¹ for the Indian Point Energy Center, which contain Safeguards Information protected under 10 CFR 73.21, is entitled: "Physical Security, Training and Qualification, and Safeguards Contingency Plan, Revision 0," and was submitted by letter dated October 14, 2004. Letter of 10-28-04

H. ENO shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Final Safety Analysis Report for Indian Point Nuclear Generating Unit No. 3 and as approved in NRC fire protection safety evaluations (SEs) dated September 21, 1973, March 6, 1979, May 2, 1980, November 18, 1982, December 30, 1982, February 2, 1984, April 16, 1984, January 7, 1987, September 9, 1988, October 21, 1991, April 20, 1994, January 5, 1995, and supplements thereto, subject to the following provision:

ENO may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

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| I. | (DELETED) | Amdt. 205
2/27/01 |
| J. | (DELETED) | Amdt. 205
2/27/01 |
| K. | (DELETED) | Amdt. 49
5-25-84 |
| L. | (DELETED) | Amdt. 205
2/27/01 |
| M. | (DELETED) | Amdt. 205
2/27/01 |
| N. | (DELETED) | Amdt. 49
5-25-84 |

¹ The Training and Qualification Plan and Safeguards Contingency Plan are Appendices to the Security Plan.

trustee has first given the NRC 30 days prior written notice of the payment. In addition, the trust agreement shall state that no disbursements or payments from the trust shall be made if the trustee receives prior written notice of objection from the Director, Office of Nuclear Reactor Regulation.

- U. The decommissioning trust agreement shall provide that the trust agreement shall not be modified in any material respect without the prior written consent of the Director, Office of Nuclear Reactor Regulation. Amdt. 203
11/27/00
- V. The decommissioning trust agreement shall state that the trustee, investment advisor, or anyone else directing the investments made in the trust shall adhere to a "prudent investment" standard, as specified in 18 CFR 35.32(a)(3) of the Federal Energy Regulatory Commission's regulations. Amdt. 203
11/27/00
- W. For purposes of ensuring public health and safety, ENIP3, upon the transfer of this license to it, shall provide decommissioning funding assurance for the facility by the prepayment or equivalent method, to be held in a decommissioning trust fund for the facility, of no less than the amount required under NRC regulations at 10 CFR 50.75. Any amount held in any decommissioning trust maintained by PASNY for the facility after the transfer of the facility license to ENIP3 may be credited towards the amount required under this paragraph. Amdt. 203
11/27/00
- X. ENIP3 shall take all necessary steps to ensure that the decommissioning trust is maintained in accordance with the application for the transfer of this license to ENIP3 and ENO and the requirements of the order approving the transfer, and consistent with the safety evaluation supporting such order. Amdt. 203
11/27/00
- AA. The following conditions relate to the amendment approving the conversion to Improved Standard Technical Specifications: Amdt. 205
2/27/01
1. This amendment authorizes the relocation of certain Technical Specification requirements and detailed information to licensee-controlled documents as described in Table R, "Relocated Technical Specifications"

Table 3.3.1-1 (page 1 of 8)
Reactor Protection System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Manual Reactor Trip	1,2	2	B	SR 3.3.1.14	NA
	3 ^(a) , 4 ^(a) , 5 ^(a)	2	C	SR 3.3.1.14	NA
2. Power Range Neutron Flux					
a. High	1,2	4 ^(j)	D	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.7 SR 3.3.1.11	≤111% RTP
b. Low	1 ^(b) , 2	4 ^(j)	E	SR 3.3.1.1 SR 3.3.1.8 SR 3.3.1.11	≤25% RTP
3. Intermediate Range Neutron Flux	1 ^(b) , 2 ^(c)	1	F	SR 3.3.1.1 SR 3.3.1.8 SR 3.3.1.11	NA

(continued)

- (a) With Rod Control System capable of rod withdrawal or one or more rods not fully inserted.
- (b) Below the P-10 (Power Range Neutron Flux) interlocks.
- (c) Above the P-6 (Intermediate Range Neutron Flux) interlocks.
- (j) Only 3 channels required during Mode 2 Physics Tests, LCO 3.1.8

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.16 RCS Specific Activity

LCO 3.4.16 The specific activity of the reactor coolant shall be within limits.

APPLICABILITY: MODES 1 and 2,
MODE 3 with RCS loop average temperature (T_{avg}) \geq 500°F.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. DOSE EQUIVALENT I-131 > 1.0 μCi/gm.</p>	<p>----- NOTE ----- LCO 3.0.4.c is applicable -----</p> <p>A.1 Verify DOSE EQUIVALENT I-131 within the acceptable region of Figure 3.4.16-1.</p> <p><u>AND</u></p> <p>A.2 Restore DOSE EQUIVALENT I-131 to within limit.</p>	<p>Once per 4 hours</p> <p>48 hours</p>
<p>B. Gross specific activity of the reactor coolant not within limit of SR 3.4.16.1.</p>	<p>B.1 Be in MODE 3 with T_{avg} < 500°F.</p>	<p>6 hours</p>

(continued)