

November 14, 2005 (5:00pm)

UNITED STATES OF AMERICA
BEFORE THE NUCLEAR REGULATORY COMMISSION
OFFICE OF THE SECRETARY

OFFICE OF SECRETARY
RULEMAKINGS AND
ADJUDICATIONS STAFF

In the Matter of)	
)	Docket No. 50-0219
AMERICAN ENERGY COMPANY,LLC (ALSO KNOWN AS AMERGEN))	
)	
OYSTER CREEK NUCLEAR GENERATING STATION)	
)	NOVEMBER 14, 2005
Regarding the Renewal of Facility Operating License No. DPR-16 for a 20-Year Period)	

CERTIFICATE OF SERVICE FOR PETITIONERS' EXHIBITS

I hereby certify that the enclosed PETITIONERS' EXHIBITS were sent this 14th day of November, 2005 via U.S. Postal Service as designated to each of the following:

Secretary of the Commission
United States Nuclear Regulatory Commission
Washington, DC 20555-0001
Attention: Rulemaking and Adjudications Staff
Email: HEARINGDOCKET@NRC.GOV

Office of General Counsel
United States Nuclear Regulatory Commission
Washington, DC 20555-0001
Email: OGCMailCenter@nrc.gov

Kathryn Sutton, Esq.
Morgan, Lewis, & Boikus LLP
1111 Pennsylvania Avenue, NW
Washington, DC 20004

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Nuclear Information and Resource Service
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Certificate of Service (continued)

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Signed,
For Michele Donato, Esq.


Michele Donato, Esq. /PG

11.14.2005

Novmeber 14, 2005

PETITIONERS' EXHIBIT 1

SSINS No.: 6835
IN 86-99

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

December 8, 1986

IE INFORMATION NOTICE NO. 86-99: DEGRADATION OF STEEL CONTAINMENTS

Addressees:

All nuclear power reactor facilities holding an operating license or a construction permit.

Purpose:

This notice is to provide recipients with current information of a potentially significant safety problem regarding the degradation of a steel containment resulting from corrosion. It is expected that recipients will review this information for applicability to their facilities and consider actions, as appropriate, to promptly recognize or prevent a similar problem from occurring. However, suggestions contained in this notice do not constitute NRC requirements; therefore, no specific action or written response is required.

Description of Circumstances:

The Oyster Creek Nuclear Generating Station first discovered water in the gap between the boiling-water-reactor drywell and the concrete shield in 1980 and began investigation of the cause in 1983. It appeared that the collection of water varied from a few drops to 2 gallons per minute, depending on whether the unit was in operation or an outage for refueling. During the spring and summer of 1986, the licensee planned work to identify and eliminate this water problem. The bellows at the drywell to cavity seal was repaired and a gasket was replaced, thus stopping the leakage. Since the bellows is located at the top of the drywell and the region above the bellows is flooded during refueling, it would explain why leakage was high during refueling and low during operation.

To determine if the water in the gap had caused damaged to the steel containment, the licensee measured the wall thickness, using an ultrasonic testing (UT) technique at two elevations. The 51-ft level near the drywell seal was sound, but there appeared to be loss of metal on the gap side at the 11-ft 3-in. level immediately above the concrete floor. In this area, the gap is packed with sand and contains five equally spaced drain pipes (see attached Figure 1). A total of 143 measurements were made at this level and 60 indicated a reduction in thickness of more than 1/4 in. from the drawing thickness of 1.154 in. These readings were found throughout seven of the ten downcomer bays. The licensee plans to cut the steel containment and remove about 12 samples to confirm and evaluate the corrosion damage.

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The licensee plans to remove a section of the drain pipe to perform a visual examination of the outside of the drywell. Wipe samples will be taken from several areas and a chemical analysis will be performed. Sand samples will be taken adjacent to the core holes and will be analyzed for chemicals, bacteria, and water composition. Some channels are being cut in the concrete floor that is inside the drywell to provide access for further UT examination of the containment-sand interface.

Discussion:

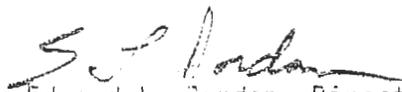
The purpose of the sand is to act as a cushion and allow expansion of the drywell during operation. The steel containment is in contact with sand in those areas where corrosion has been detected. The containment material is ASTM A-212 Grade B carbon steel plate. The licensee stated that the outside surface was protected with a red lead coating from above the drywell down to about the 10-ft. level, which means that the interface between the lead paint and the unprotected steel was in contact with wet sand. Red lead protects steel by providing a stable and impenetrable surface, but the steel is sacrificial with respect to the lead in dilute, acidic water conditions.

It is possible that condensation during initial construction, moisture pickup through the drain line during operation, and the leaking bellows wetted the sand, thereby causing corrosion of the containment steel plates. During construction, water was seen running down the outside of containment into the sand. The five drain lines, as well as other penetrations in the concrete shield, are open during operation and would allow moist air to enter and rise up the gap and later cool and condense as water. Water also was able to enter the gap through the holes in the bellows during refueling until repairs were made.

A related matter is discussed in IE Information Notice 86-35, "Fire in Compressible Material at Dresden Unit 3," where a large amount of water was used to extinguish the slowly burning fire between the drywell and the concrete shield. Oyster Creek uses different filler material.

The NRC is continuing to obtain and evaluate pertinent information. If specific actions are required, an additional notification will be made.

No specific action or written response is required by this information notice. If you have any questions about this matter, please contact the Regional Administrator of the appropriate regional office or this office.



Edward L. Jordan, Director
Division of Emergency Preparedness
and Engineering Response
Office of Inspection and Enforcement

Technical Contact: Paul Cortland, IE
(301) 492-4175

Attachments:

1. Figure 1. Sketch of Possible Degraded Area

Sketch of Possible Degraded Areas

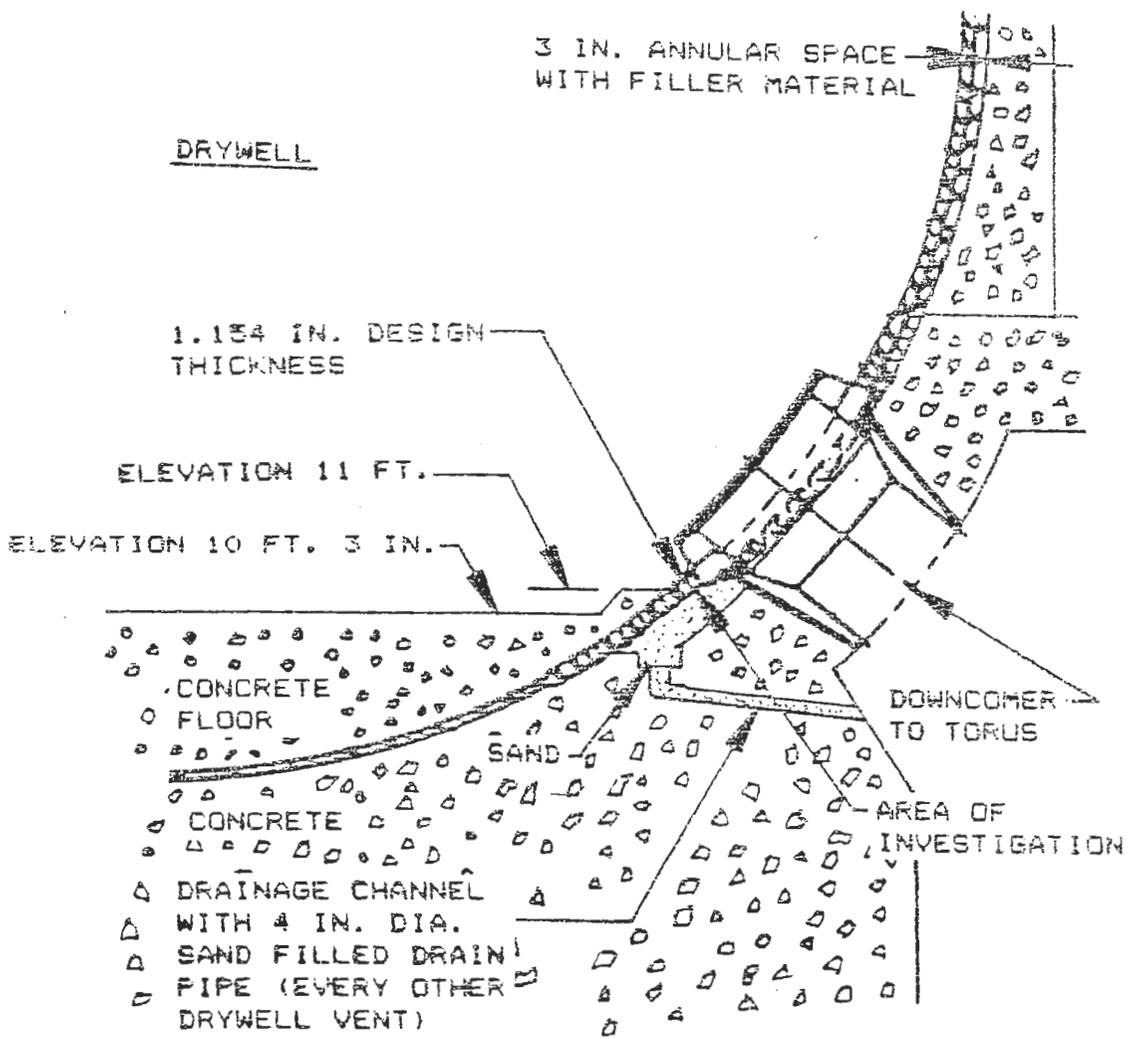


Figure 1

EXHIBIT 2

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF NUCLEAR REACTOR REGULATION
WASHINGTON, D.C. 20555

February 14, 1991

Information Notice No. 86-99, SUPPLEMENT 1: DEGRADATION OF STEEL
CONTAINMENTS

Addressees:

All holders of operating licenses or construction permits for nuclear power reactors.

Purpose:

This supplement to Information Notice (IN) 86-99 is intended to alert addressees to additional information about a potential degradation problem regarding corrosion in steel containments. It is expected that recipients will review the information for applicability to their facilities and consider actions, as appropriate, to avoid similar problems. However, suggestions contained in this supplement to the information notice do not constitute NRC requirements; therefore, no specific action or written response is required.

Discussion:

IN 86-99 was issued on December 8, 1986, in response to the discovery of significant corrosion on the external surface of the carbon steel drywell in the sand bed region of the Oyster Creek plant. This supplement updates the status of Oyster Creek containment corrosion and the licensee's mitigation program.

Since drywell corrosion was detected in 1986, the licensee instituted periodic wall thickness measurements by the ultrasonic testing (UT) technique to determine corrosion rates. The most severe corrosion was found in the sand bed region at a nominal elevation of 11'-3". The highest corrosion rate determined was 35.2 +/- 6.8 mils per year. To mitigate the corrosion in the sand bed region, water was drained from the sand bed and cathodic protection (CP) was installed in the bays with the greatest wall thinning in early 1989. Subsequent UT thickness measurements in these bays indicated that CP was ineffective. The licensee's consultants indicated that it would be necessary to flood the sand bed and to install CP in all the bays to make the CP system effective. The licensee decided that large amounts of water in the sand bed would be counterproductive.

In the spherical portion of the drywell above the sand bed region, the highest corrosion rate determined was 4.6 +/- 1.6 mils per year at a nominal elevation of 51'. In the cylindrical portion of the drywell above the spherical portion, where minor corrosion was discovered and was thought to have originated mostly during construction, no significant wall thinning was detected (at a nominal elevation of 87'). However, this is the region in which the nominal thickness of the wall has the least margin, thus requiring periodic monitoring of actual thickness.

The licensee has instituted a drywell program to arrest corrosion and to ensure containment integrity for the full licensed term of the plant. The licensee has taken action to investigate, identify, and correct leak paths into the drywell gap and plans to take more action to survey leakage and prevent it. The stainless steel liners in the refueling cavity and the equipment pool developed cracks along the perimeter of the liner plates where they were welded to embedded channels. For the refueling cavity, all potential leakage pathways have been thoroughly checked and liner cracks are sealed with adhesive stainless steel tape before a strippable coating is applied. Since the refueling cavity is flooded only during refueling, no leakage concerns exist at other times. At the end of an outage, the refueling cavity is drained, and the tape and strippable coating are removed. The licensee found leaks related to the equipment pool and stopped them with liner weld repairs. The equipment pool also will be protected with a strippable coating during flooded periods of operation.

The licensee believes that a thorough program has been established for managing leakage that could affect drywell integrity due to corrosion from moisture ingress into the drywell gap. Recent surveillance of the sand bed drains indicates that the sand bed is free of water. To further mitigate drywell corrosion, the licensee is considering removing the sand, insulation, gap filler material, and corrosion film and applying a protective coating to the exterior drywell surface. The licensee is proceeding with the analysis, engineering and planning to support removing the sand from the drywell sand bed region in the near future. Removal of the insulation and gap filler material from the drywell gap is being evaluated for future consideration.

The BWR Owners Group is surveying its members to determine whether other plants are experiencing water leakage into the drywell gap and possible corrosion of the exterior surfaces in the sand bed region as well as in the spherical and cylindrical parts of the drywell.

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IN 86-99, Supplement

February 14, 1991

Page 3 of 3

This supplement requires no specific action or written response. If you have any questions about the information in this notice, please contact one of the technical contacts listed below or the appropriate NRR project manager.

Charles E. Rossi, Director
Division of Operational Events Assessment
Office of Nuclear Reactor Regulation

Technical Contacts: Frank J. Witt, NRR
(301) 492-0767

C.P. Tan, NRR
(301) 492-3315

Attachment: List of Recently Issued NRC Information Notices

PETITIONERS' EXHIBIT 3

April 14, 1992

Docket No. 50-219

Mr. John J. Barton
 Vice President and Director
 GPU Nuclear Corporation
 Oyster Creek Nuclear Generating Station
 Post Office Box 388
 Forked River, New Jersey 08731

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Dear Mr. Barton:

SUBJECT: EVALUATION REPORT ON STRUCTURAL INTEGRITY OF THE OYSTER CREEK
 DRYWELL (TAC NO. M79166)

The staff has completed the review and evaluation of the stress analyses and stability analyses reports of the corroded drywell with and without the sand bed. Our evaluation report is contained in the enclosure. GPUN used the analyses to justify the removal of the sand from the sand bed region. Even though the staff, with the assistance of consultants from Brookhaven National Laboratory (BNL), concurred with GPUN's conclusion that the drywell meets the ASME Section III Subsection NE requirements, it is essential that GPUN continue UT thickness measurements at refueling outages and at outages of opportunity for the life of the plant. The measurements should cover not only areas previously inspected but also accessible areas which have never been inspected so as to confirm that the thickness of the corroded areas are as projected and the corroded areas are localized.

We request that you respond within 30 days of receipt of this letter indicating your intent to comply with the above requirements as discussed in the Safety Evaluation.

The requirements of this letter affect fewer than 10 respondents, and therefore, are not subject to Office of Management and Budget review under E.O. 12812.

Sincerely,

101

Alexander W. Dromerick, Sr. Project Manager
 Project Directorate 1-4
 Division of Reactor Projects - 1/11
 Office of Nuclear Reactor Regulation

9204030078 920406
 PDR ADOCK OSCORP
 PDR

Enclosure:
 As stated

cc w/enclosure:
 See next page

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Document Name: M79166

DOC	CLASS	PH	D	DATE	TIME	STATUS
1	1A:PD1-4	1PH:PD1-4	1D:PD1-4	4/14/92	10:00	INDEXED
2	1A:PD1-4	1PH:PD1-4	1D:PD1-4	4/14/92	10:00	INDEXED
3	1A:PD1-4	1PH:PD1-4	1D:PD1-4	4/14/92	10:00	INDEXED



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

DRYWELL STRUCTURAL INTEGRITY

OYSTER CREEK NUCLEAR GENERATING STATION

GPU NUCLEAR CORPORATION

DOCKET NO. 50-219

I. INTRODUCTION

In 1986 the steel drywell at Oyster Creek Nuclear Generating Station (OCNGS) was found to be extensively corroded in the area of the shell which is in contact with the sand cushion around the bottom of the drywell. Since then GPU Nuclear Corporation, (GPUN, the licensee of OCNGS), has instituted a program of periodic inspection of the drywell shell sand cushion area through ultrasonic testing (UT) thickness measurements. The inspection has been extended to other areas of the drywell and some areas above the sand cushion have been found to be corroded also. From the UT thickness measurements, one can conclude that corrosion of the drywell shell in the sand cushion area is continuing. In an attempt to eliminate corrosion or reduce the corrosion rate, the licensee tried cathodic protection and found it to be of no avail. An examination of the results of consecutive UT measurements, confirmed that the corrosion is continuing. There is concern that the structural integrity of the drywell cannot be assured. Since the root cause of the corrosion in the sand cushion area is the presence of water in the sand, the licensee has considered sand removal to be an important element in its program to eliminate the corrosion threat to the drywell integrity.

In the program, the licensee first established the analysis criteria and then performed the analyses of the drywell for its structural adequacy with and without the presence of the sand. The licensee performed stress analyses and stability analyses for both with and without the sand cases and concluded the drywell with or without the sand to be in compliance with the criteria established for the reevaluation. It is to be noted that the original purpose of the sand cushion is to provide a smooth transition of stresses from the fixed portion to the free-standing portion of the steel drywell.

II. EVALUATION

The staff with the assistance of consultants from Brookhaven National Laboratory (BNL) has reviewed and evaluated the information (Refs. 1,2,3,4,5) provided by the licensee.

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level of stress intensity and the extent of the discontinuity to be considered localized. A stress intensity limit of 1.1 Smc is specified at the boundary of the region within which the membrane stress can be higher than 1.1 Smc. The region where the stress intensity varies from 1.1 Smc to 1.0 Smc is not defined in the Code because of the fact that it varies with the loading. In view of this, the licensee rationalized that the 1.1 Smc can be applied beyond the region defined by NE-3213.10 for localized discontinuity without any restriction throughout the drywell. The staff disagreed with the licensee's interpretation of the Code. The staff pointed out that for Oyster Creek drywell, stresses due to internal pressure should be used as the criterion to establish such a region. The interpretation of Section XI Subsections IWE-3519.3 and IWE-3122.4 can be made only in the same context. It is staff's position that the primary membrane stress limit of 1.1 Smc not be used indiscriminately throughout the drywell.

In order to use NE-3213.10 to consider the corroded area as a localized discontinuity, the extent of the reduction in thickness due to corrosion should be reasonably known. UT thickness measurements are highly localized; however, from the numerous measurements so far made on the Oyster Creek drywell, one can have a general idea of the overall corroded condition of the drywell shell and it is possible to judiciously apply the established re-analysis criteria.

2. Re-analyses

The re-analyses were made by General Electric Company for the licensee, one reanalysis considered the sand present and the other considered the drywell without the sand. Each re-analysis comprises a stress analysis and stability analysis. Two finite element models, one axisymmetric and another a 25° pie slice model were used for the stress analysis. The ANSYS computer program was used to perform the analyses. The axisymmetric model was used to determine the stresses for the seismic and the thermal gradient loads. The pie slice model was used for dead weight and pressure loads. The pie slice model includes the vent pipe and the reinforcing ring, and was also used for buckling analysis. The same models were used for the cases with and without sand, except that in the former, the stiffness of sand in contact with the steel shell was considered. The shell thickness in the sand region was assumed to be 0.700" for the with-sand case and to be 0.736" for the without-sand case. The 0.70" was, as claimed by the licensee, used for conservatism and the 0.736" is the projected thickness at the start of fuel cycle 148. The same thicknesses of the shell above the sand region were used for both cases. For the with-sand case, an analysis of the drywell with the original nominal wall thicknesses was made to check the shell stresses with the allowable values established for the re-analyses.

The licensee used the same load combinations as specified in Oyster Creek's final design safety analysis report (FDSAR) for the re-analyses. The licensee made a comparison of the load combinations and corresponding allowable stress

limits using the Standard Review Plan (SRP) Section 3.9.2 and concluded that they are comparable.

The results of the re-analyses indicated that the governing thicknesses are in the upper sphere and the cylinder where the calculated primary membrane stresses are respectively 20,360 psi and 19,850 psi vs. the allowable stress value of 19,300 psi. There is basically no difference, in the calculated stresses at these levels, between the with and without sand cases. This should be expected, because in a steel shell structure the local effect of the edge effect is damped in a very short distance. The stresses calculated exceed the allowable by 3% to 6%, and such exceedance is actually limited to the corroded area as obtained from UT measurements. However, in order to perform the axisymmetric analysis and analysis of the pie slice model, uniform thicknesses were assumed for each section of the drywell. Therefore, the calculated over-stresses may represent only stresses at the corroded areas and the stresses for areas beyond the corroded areas are less and would most likely be within the allowable as indicated in results of the analyses for nominal thicknesses. The diagram in Ref. 6 indicated such a condition. It is to be noted that the stresses for the corroded areas were obtained by multiplying the stresses for nominal thicknesses by the ratios between the corroded and nominal thicknesses.

The buckling analyses of the drywell were performed in accordance with ASME Code Case N-284. The analyses were done on the 36" pie slice model for both with-sand and without-sand cases. Except in the sand cushion area where a shell thickness of 0.7" for the with-sand case and a shell thickness of 0.330" for the without-sand case were used, nominal shell thicknesses were considered for other sections. The load combinations which are critical to buckling were identified as those involving refueling and post accident conditions. By applying a factor of safety of 2 and 1.67 for the load combinations involving refueling and the post-accident conditions respectively, the licensee established for both cases the allowable buckling stresses which are obtained after being modified by capacity and plasticity reduction factors. It is found that the without-sand, case for the post-accident condition is most limiting in terms of buckling with a margin of 14%. The staff and its Greenhaven National Laboratory (BNL) consultants concur with the licensee's conclusion that the Oyster Creek drywell has adequate margin against buckling with no sand support for an assumed sanded region shell thickness of 0.330 inch.

A copy of BNL's technical evaluation report is attached to this safety evaluation.

III. CONCLUSION

With the assistance of consultants from BNL, the staff has reviewed and evaluated the responses to the staff's concerns and the detailed re-analyses of the drywell for the with-sand and without-sand cases. The re-analyses by the licensee indicated that the corroded drywell meets the requirements for

containment vessels as contained in ASME Section III Subsection NE through summer 1977 addenda. This Code was adopted in the Mark I containment program. The staff agrees with the licensee's justification of using the above mentioned Code requirements with one exception, the use of 1.1 Smc throughout the drywell shell in the criteria for stress analyses. It is the staff's position that the primary membrane stress limit of 1.1 Smc not be used indiscriminately throughout the drywell. The staff accepted the licensee's reanalyses on the assumption that the corroded areas are highly localized as indicated by the licensee's UI measurements. The stresses obtained for the case of reduced thickness can only be interpreted to represent those in the corroded areas and their adjacent regions of the drywell shell. In view of these observations, it is essential that the licensee perform UT thickness measurements at refueling outages and at outages of opportunity for the life of the plant. The measurements should cover not only areas previously inspected but also accessible areas which have never been inspected so as to confirm that the thicknesses of the corroded areas are as projected and the corroded areas are localized. Both of these assumptions are the bases of the reanalyses and the staff acceptance of the reanalysis results.

References:

1. "An ASME Section VIII Evaluation of the Oyster Creek Drywell Part 1, Stress Analysis" GE Report No. 9-1 DRF #00664 November 1990, prepared for GPUN (with sand).
2. "Justification for use of Section III, Subsection NE, Guidance in Evaluating the Oyster Creek Drywell" TR-7377-1, Teledyne Engineering Services, November 1990 (Appendix A to Reference 1).
3. "An ASME Section VIII evaluation of the Oyster Creek Drywell, Part 2, Stability Analysis" GE Report No. 9-2 DRF #00664, Rev. 0, & Rev. 1, November 1990, prepared for GPUN (with sand).
4. "An ASME Section VIII Evaluation of Oyster Creek Drywell for without sand case, Part 1, stress analysis" GE Report No. 9-3 DRF #00664, Rev. 0, February 1991. Prepared for GPUN.
5. "An ASME Section VIII Evaluation of Oyster Creek Drywell, for without sand case, Part 2 Stability Analysis" GE Report No. 9-4, DRF #00664 Rev. 0, Rev. 1 November 1990, prepared for GPUN.
6. Diagram attached to a letter from J. C. Davine Jr. of GPUN to NRC dated January 17, 1992 (0321-92-2020, 5000-92-2094).

Principal Contributor: C.P. Tan

Date: April 26, 1992

Attachment:
DNL Technical Evaluation
Report

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 17, 1993

Docket No. 50-219

LICENSEE: GPU Nuclear Corporation

FACILITY: Oyster Creek Nuclear Generating Station

SUBJECT: SUMMARY OF MAY 5, 1993, MEETING WITH GPU NUCLEAR CORPORATION (GPUN)
TO DISCUSS MATTERS RELATED TO THE OYSTER CREEK DRYWELL CORROSION
MITIGATION PROGRAM.

On Wednesday, May 5, 1993, a meeting was held at the NRC, One White Flint North, Rockville, Maryland, with GPUN, the licensee to discuss the Oyster Creek Drywell Corrosion Mitigation Program. Enclosure 1 is the list of participants.

Enclosure 2 is the licensee's agenda and the information presented regarding the Oyster Creek Drywell Corrosion Mitigation Program. The following is a summary of the significant items discussed.

GPUN updated the staff regarding the Oyster Creek Drywell Program since the last meeting held in July 1991.

During operating Cycle 13, the licensee performed structural analyses related to sand removal which were approved by the NRC. The licensee also stated that a concrete mockup of the drywell shield was built to demonstrate that the sand could be removed by cutting access holes in the shield wall which permit workers to remove the sand from the sand cushion area. During this period the licensee cut holes in all 10 bays of the shield wall and removed 60% of the sand in the area.

During the recent 14R refueling outage the remaining sand and corrosion products were removed from the sand bed region together with rust and scale from the drywell shell. The licensee also cleaned and pressure washed the shell and provided protective coatings.

GPUN provided a summary of 14R outage UT thickness measurements taken from the inside drywell. The current thinnest thickness of the (1) cylinder is 0.614," thick, (2) upper sphere is 0.691" thick, (3) middle sphere is 0.743" thick, and (4) sand bed region is 0.800" thick. The licensee stated that the minimum required ASME Code thickness for this area is 0.736" thick. All UT thickness measurements for Bays 3, 5, 7, and 19 are greater than 0.736" thick. All UT thickness measurements for Bays 11, 15 and 17 are greater than 0.736" except for one measurement for each bay. Bays 1 and 13 each have several locations where the measured thicknesses are below 0.736". GPUN stated that these locations are isolated. GPUN further stated that on the basis of the measurements the shell was re-evaluated using ASME local stress criteria.

1993

1993

Based on the above, GPUN concluded that (1) sand removal was successfully completed in refueling outage 14R, (2) external drywell surface in sand bed region was cleaned and coated, (3) corrosion in sand bed region is now stopped, (4) the 14R inspections from the sand bed region confirmed that GPUN's previous inspections from inside the drywell were in the area of highest corrosion, (5) the integrity of the Oyster Creek drywell remains a priority concern of GPUN management and GPUN will continue UT thickness measurements for the life of the plant, and (6) low corrosion rates in sphere and cylinder portions of the drywell, in conjunction with pressure reduction approval, will ensure containment integrity for the full licensed life of the plant. The licensee proposed to use 1.1 Smc, to evaluate the overall shell thickness requirements. The staff disagrees with this interpretation. However, 1.1 Smc could be used in evaluating local areas of shell.

GPUN indicated that priority 1 & 2 UT readings from inside drywell during refueling outage 15R will be taken and that there would be a visual inspection of coating in Bays I and II at the beginning of 15R. This inspection may be expanded to other bays if found necessary. GPUN will monitor for water leakage during operating cycles and refueling outages, and take corrective action as appropriate. GPUN further indicated that post refueling outage UT inspection program scope and frequency will be defined based on 15R inspection results.

Original signed by:

Alexander W. Dromerick, Sr. Project Manager
 Project Directorate I-4
 Division of Reactor Projects - 1/11
 Office of Nuclear Reactor Regulation

Enclosures:

1. Attendance List
2. Licensing Agenda

cc w/enclosures:
 see next page

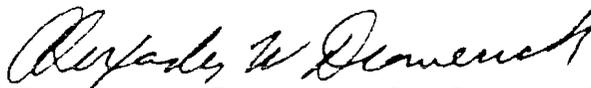
OFFICE:	LA:PD#I-4	PM:PD#I-4	D:PD#I-4		
NAME:	Storris	A Dromerick:b	JStolz		
DATE:	5/17/93	5/17/93	5/17/93		

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May 17, 1993

Based on the above, GPUN concluded that (1) sand removal was successfully completed in refueling outage 14R, (2) external drywell surface in sand bed region was cleaned and coated, (3) corrosion in sand bed region is now stopped, (4) the 14R inspections from the sand bed region confirmed that GPUN's previous inspections from inside the drywell were in the area of highest corrosion, (5) the integrity of the Oyster Creek drywell remains a priority concern of GPUN management and GPUN will continue UT thickness measurements for the life of the plant, and (6) low corrosion rates in sphere and cylinder portions of the drywell, in conjunction with pressure reduction approval, will ensure containment integrity for the full licensed life of the plant. The licensee proposed to use 1.1 Smc, to evaluate the overall shell thickness requirements. The staff disagrees with this interpretation. However, 1.1 Smc could be used in evaluating local areas of shell.

GPUN indicated that priority 1 & 2 UT readings from inside drywell during refueling outage 15R will be taken and that there would be a visual inspection of coating in Bays 1 and 11 at the beginning of 15R. This inspection may be expanded to other bays if found necessary. GPUN will monitor for water leakage during operating cycles and refueling outages, and take corrective action as appropriate. GPUN further indicated that post refueling outage UT inspection program scope and frequency will be defined based on 15R inspection results.



Alexander W. Dromerick, Sr. Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Attendance List
2. Licensing Agenda

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G. Bagchi

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J. Davis

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cc w/both enclosures

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Generating Station

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Resident Inspector
c/o U.S. Nuclear Regulatory Commission
Post Office Box 445
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Kent Tosch, Chief
New Jersey Department of
Environmental Protection
Bureau of Nuclear Engineering
CN 415
Trenton, New Jersey 08625

Mr. John J. Barton
Vice President and Director
GPU Nuclear Corporation
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

ENCLOSURE 1

Oyster Creek Nuclear Generating Station

Docket No. 50-219

Meeting - May 5, 1993

Attendance List

NAME

ORGANIZATION

Alexander Dromerick	NRC/NRR/PD I-4
Robert McBrearty	NRC/RG-I
Goutam Bagchi	NRC/NRR/DE/ECGB
Chen Tan	NRC/NRR/DE/ECGB
Kris Parczewski	NRC/NRR/DE/EMCB
Jim Davis	NRC/NRR/DE/EMCB
Mark Yekta	GPUN
Dana Govill	GPUN-NDE/ISI
Phil Manning	GPUN-NDE/ISI
Mike Laggart	GPUN-Licensing
Steve Tumminelli	GPUN-Engineering Mechanics
Sam Saha	GPUN-Material Engineering
Julien Abramovici	GPUN-Mechanical
John Flynn	GPUN-Project Manager
Joe Colitz	GPUN-Director Engineer Project
Ron Zak	GPUN-Licensing

ENCLOSURE 2

OYSTER CREEK DRYWELL CORROSION MITIGATION

NRC PRESENTATION - MAY 5, 1993

AGENDA

J. J. COLITZ

- MEETING OBJECTIVES

J. C. FLYNN

- CYCLE 13 ACTIVITIES

- Analysis
- Work Accomplished
- Training

- 14R OUTAGE ACTIVITIES

- SUMMARY OF 14R OUTAGE UT THICKNESS MEASUREMENTS (Taken from Inside Drywell)

S. SAHA

- SAND BED REGION

- Shell Condition
- Concrete Floor Condition
- Determining UT Locations

- EVALUATION OF SHELL THICKNESS (UT) MEASUREMENTS S. TUMMINELLI

- CONCLUSIONS & FUTURE DRYWELL INSPECTIONS/PLAN J. J. COLITZ

MEETING OBJECTIVES

- **UPDATE THE STAFF REGARDING THE OYSTER CREEK DRYWELL**
- **PRESENT THE ACTIVITIES ACCOMPLISHED IN CYCLE 13 AND 14R FOR THE DRYWELL CORROSION MITIGATION PROJECT**
- **DISCUSS 14R OUTAGE INSPECTION RESULTS**
- **DISCUSS FUTURE DRYWELL INSPECTIONS/PLAN**

OYSTER CREEK DRYWELL PROGRAM

Phase 1	Phase II	Phase III
Phase: Develop Success Path	Phase II Solve the Problem	Phase III Keep It Solved
Objective: Through 1990	Through 1992	Long Term
Timing: <ul style="list-style-type: none"> ● Examine all information in-hand regarding physical condition and design of the drywell and confirm that shell integrity is adequate to support plant operation through Phase II. 	<ul style="list-style-type: none"> ● Implement plans/engineering developed in Phase I. <ul style="list-style-type: none"> — Characterize shell condition and corrosion mechanism. — Arrest corrosion where necessary by <u>positive</u> means (e.g., remove sand, clean and resurface). 	<ul style="list-style-type: none"> ● Implement life-of-plant monitoring program. ● If required, strengthen shell.
Focus: <ul style="list-style-type: none"> ● Develop detailed plan and engineering (for implementation in Phase II) to: <ul style="list-style-type: none"> — Completely characterize shell condition. — Arrest corrosion in any location where continued corrosion would not permit operation for full term plant life. 	<ul style="list-style-type: none"> ● Stop leakage of water/take steps to ensure that it stays stopped. ● Complete analysis of shell strength, design basis and margin. ● Determine if shell strengthening is required, engineer methods to do so (if required). ● Develop life-of-plant monitoring program. 	
Licensing: Obtain NRC understanding and agreement.	Change licensed design basis if appropriate.	

CYCLE 13 ACTIVITIES

(JUNE 1991 - NOVEMBER 1992)

ANALYSIS

- COMPLETED STRUCTURAL ANALYSIS
 - o Submitted to the NRC
 - o SER Issued
 - o Analyzed Shield Wall for 20' Access Holes

WORK ACCOMPLISHED

- REMOVED 60% OF SAND
- BUILT CONCRETE MOCKUP
- CUT ACCESS HOLES IN ALL TEN BAYS

TRAINING

- ACCESS TRAINING FOR WORKERS AND EMTS
- PROCESS TRAINING FOR WORKERS

14R OUTAGE ACTIVITIES

(NOVEMBER 1992 - FEBRUARY 1993)

- WORK FORCE OF APPROXIMATELY 50 PEOPLE - COMPLETED ALL TEN BAYS
- REMOVED SAND AND CORROSION PRODUCTS FROM SAND BED REGION
- REMOVED RUST / SCALE FROM DRYWELL SHELL
- CLEANED AND PRESSURE WASHED THE SHELL
- APPLIED THE COATING SYSTEM
 - o One Pre-Primer Coat • Devoe 167
 - o Two Top Coats • Devoe 184
 - o Caulked InnerRadius • Devoe 142

SAND BED REGION

14R OUTAGE DRYWELL CONDITION

SHELL CONDITION

- ALL BAYS WERE ENTERED, INSPECTED AND COATED, ENTRIES WERE MADE DURING VARIOUS STAGES OF WORK
- CORROSION OBSERVED ON DW SHELL IN ALL BAYS BELOW VENT PIPE REINFORCEMENT PLATE
- VENT PIPE AND SHELL ABOVE THE REINFORCEMENT PLATE SHOWED NO CORROSION; ORIGINAL RED LEAD PRIMER WAS STILL VISIBLE
- ALL BAYS SHOWED PRESENCE OF A "BATHTUB RING" - AN 8 TO 18 INCH WIDE BAND ABOUT 30 TO 40 INCHES LONG JUST BELOW VENT PIPE REINFORCEMENT PLATE CONTAINING HEAVILY CORRODED AREAS
 - "Ring" very prominent in some bays and less striking in others
 - Above this "Ring," shell had no corrosion
 - Below this "Ring," uniform corrosion was seen
 - Laterally, beyond this "Ring," uniform corrosion was seen
 - The "Ring" is believed to be the air-water interface when sand bed was saturated with water
- ALL DW SHELL PLUGS INSTALLED IN SAND BED REGION WERE FOUND IN THE CORRODED "BATHTUB RING" AREAS

CONCRETE FLOOR CONDITION

- UPON SAND REMOVAL - THE CONCRETE FLOOR WAS FOUND TO BE UNFINISHED IN ALL BAYS
 - Drainage channel, as shown in drawing, was completely missing
 - Drain pipes were 6 to 8 inches above floor level and some were clogged
 - Floor was cratered with some craters adjacent to shell. A few craters were big, about 12-13 feet long, 12-20 inches deep and 8-12 inches wide
 - Concrete reinforcement bars for the floor could be seen bare in many bays

- TO SUMMARIZE, THE CONCRETE FLOOR CONDITION(S) PREVENTED PROPER DRAINAGE OF WATER WHICH, IN TURN, AGGRAVATED THE CORROSION OF DW SHELL

DETERMINING UT LOCATIONS

- PURPOSE - TO OBTAIN AND MEASURE THINNEST AREAS ON DRYWELL SHELL NOW THAT ENTIRE SHELL SURFACE AREA WAS ACCESSIBLE
- UT "A" SCAN CONSISTENTLY SHOWED THAT INSIDE DRYWELL SHELL SURFACE WAS UNIFORM
- THEREFORE, DEEPLY CORRODED AREAS ON OUTSIDE SURFACE OF THE DRYWELL SHELL SHOULD BE THE THINNEST AREAS
- BASED UPON VISUAL OBSERVATION, 6 TO 12 DEEPLY CORRODED SPOTS PER BAY WERE MARKED FOR GRINDING AND UT, ADDITIONAL SPOTS WERE MARKED IN SOME BAYS BASED ON FINDINGS
- MOST OF THESE SPOTS WERE SCATTERED OVER THE ENTIRE BAY AND LOCALIZED
- ABOVE METHOD OF LOCATING AND GRINDING INTRODUCED CONSERVATISM IN SHELL THICKNESS MEASUREMENT
 - We forced a bias in selecting the thinnest spots
 - Grinding of these spots, to obtain flat surface for UT, removed additional good metal

EVALUATION OF SHELL THICKNESS (UT) MEASUREMENTS

- THE BASIS FOR EVALUATION IS A SHELL THICKNESS OF 0.736 INCH WHERE BUCKLING IS THE GOVERNING CRITERION.
- ALL UT THICKNESS MEASUREMENTS FOR BAYS 3, 5, 7, 9 AND 19 ARE GREATER THAN 0.736 INCH.
- ALL UT THICKNESS MEASUREMENTS FOR BAYS 11, 15 AND 17 ARE GREATER THAN 0.736 INCH EXCEPT FOR ONE MEASUREMENT FOR EACH BAY.
THESE MEASUREMENTS WERE EVALUATED USING ASME LOCAL STRESS CRITERIA.
- BAYS 1 AND 13 EACH HAVE SEVERAL LOCATIONS WHERE THE MEASURED THICKNESS IS BELOW 0.736 INCH. THESE LOCATIONS ARE ISOLATED.
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- ONE AREA IN EACH BAY WAS IDENTIFIED WHERE THE MEASURED THICKNESS IS BELOW 0.736 INCH. THESE AREAS ARE 4 X 4 INCHES FOR BAY 1 AND 6 X 6 INCHES FOR BAY 13. IN BAY 13 THIS AREA IS APPROXIMATELY 0.33 % OF THE AREA OF THE BAY.

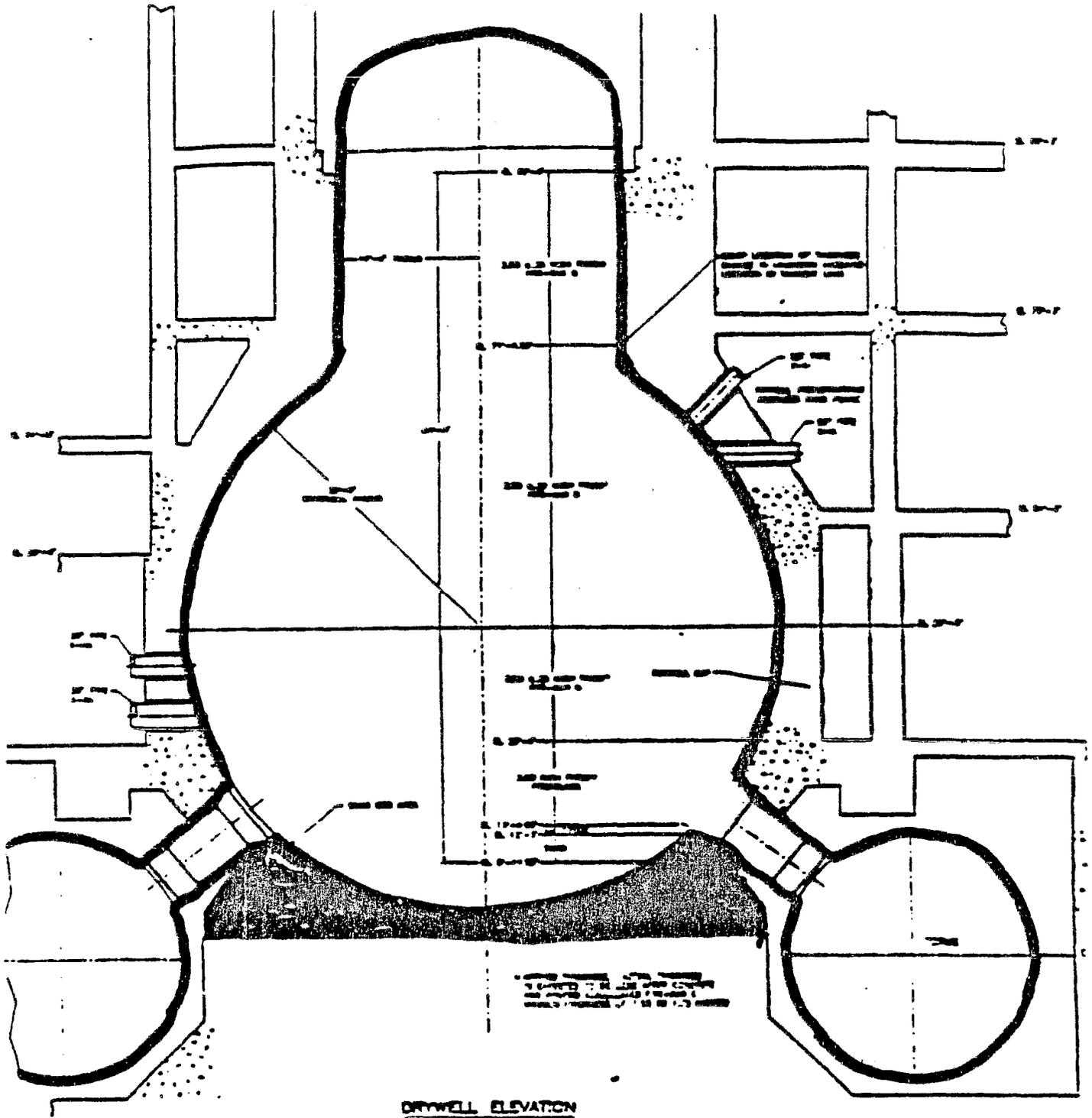
- THESE AREAS WERE EVALUATED USING ASME LOCAL ACCEPTANCE CRITERIA AND ANALYSIS WHICH ACCOUNTS FOR THE FOLLOWING FACTORS:
 - Shell Roughness
 - Local nature of individual thickness measurements
 - Adjustment of buckling capacity to account for general wall thickness found
 - Adjustment of buckling capacity to account for local thinned area

CONCLUSIONS

- SAND REMOVAL WAS SUCCESSFULLY COMPLETED IN 14R
- EXTERNAL DRYWELL SURFACE IN SAND BED REGION WAS CLEANED AND COATED
- CORROSION IN THE SAND BED REGION NOW STOPPED
- THE 14R INSPECTIONS FROM THE SAND BED REGION CONFIRMED THAT OUR PREVIOUS INSPECTIONS FROM INSIDE THE DRYWELL WERE IN THE AREA OF HIGHEST CORROSION. PLUGS WERE IN THE "BATHTUB RING"
- THE INTEGRITY OF THE OYSTER CREEK DRYWELL REMAINS A PRIORITY CONCERN OF GPUN MANAGEMENT, WE WILL CONTINUE UT THICKNESS MEASUREMENTS FOR THE LIFE OF THE PLANT
- LOW CORROSION RATES IN SPHERE AND CYLINDER PORTIONS OF THE DRYWELL, IN CONJUNCTION WITH PRESSURE REDUCTION APPROVAL, WILL ENSURE CONTAINMENT INTEGRITY FOR THE FULL LICENSED LIFE OF THE PLANT

FUTURE DRYWELL INSPECTIONS/PLAN

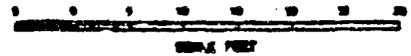
- PRIORITY 1 & 2 UT READINGS FROM INSIDE DRYWELL DURING 15R (1994)
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- MONITOR FOR WATER LEAKAGE DURING OPERATING CYCLES AND REFUELING OUTAGES, TAKE CORRECTIVE ACTION AS APPROPRIATE
- POST 15R UT INSPECTION PROGRAM SCOPE AND FREQUENCY TO BE DEFINED BASED ON 15R INSPECTION RESULTS



DRYWELL ELEVATION
LOOKING NORTH

MATERIAL VOLUMES:

- FIRE-BAR 0 2700 FT³
- FIRECLAY 200 FT³
- SAND 900 FT³



OYSTER CREEK DRYWELL

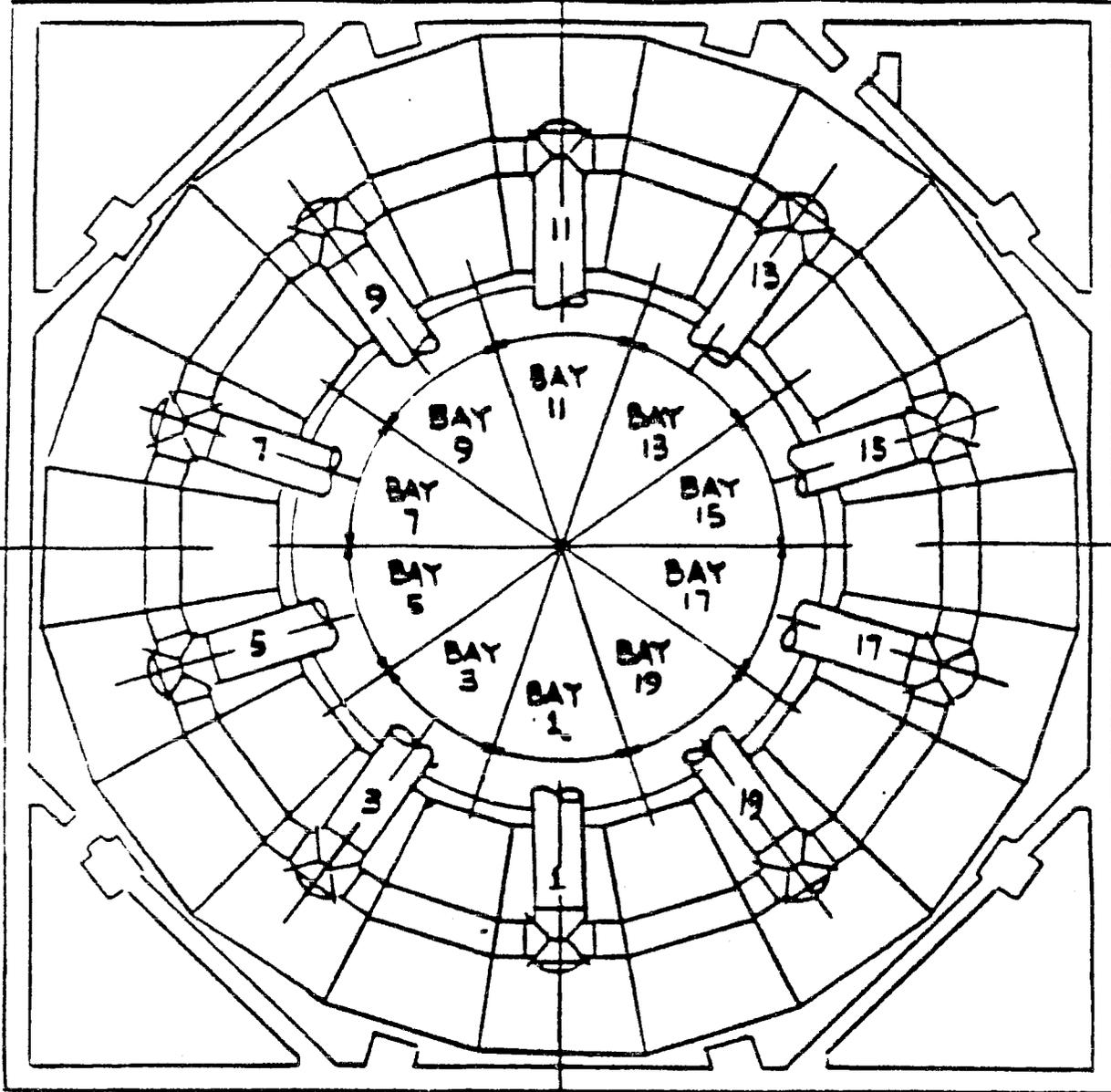


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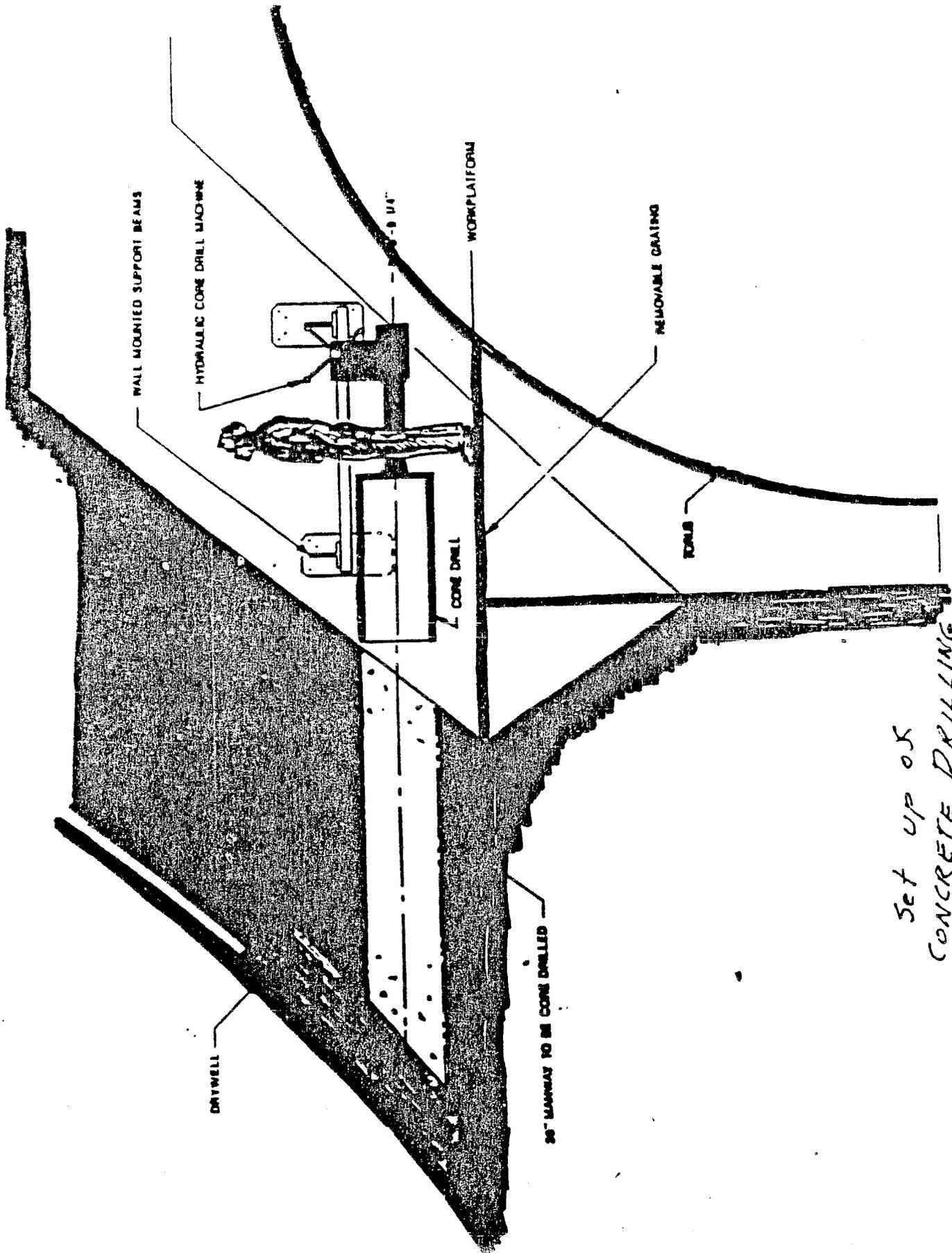
90°

270°

0°



PLAN VIEW
TORUS & DRYWELL



Set up OK
 CONCRETE DRILLING
 EQUIPMENT

ENCLOSURE 2

PETITIONERS' EXHIBIT 5

OYSTER CREEK DRYWELL CORROSION MITIGATION

NRC PRESENTATION - MAY 5, 1993

ENCLOSURE 1

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Docket No. 50-219

Meeting - May 5, 1993

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Phase:	Phase I	Phase II	Phase III
Objective:	Develop Success Path	Solve the Problem	Keep It Solved
Timing:	Through 1990	Through 1992	Long Term
Focus:	<ul style="list-style-type: none"> ● Examine all information in-hand regarding physical condition and design of the drywell and confirm that shell integrity is adequate to support plant operation through Phase II. ● Develop detailed plan and engineering (for implementation in Phase II) to: <ul style="list-style-type: none"> — Completely characterize shell condition. — Arrest corrosion in any location where continued corrosion would not permit operation for full term plant life. 	<ul style="list-style-type: none"> ● Implement plans/engineering developed in Phase I. <ul style="list-style-type: none"> — Characterize shell condition and corrosion mechanism. — Arrest corrosion where necessary by positive means (e.g., remove sand, clean and resurface). ● Stop leakage of water/take steps to ensure that it stays stopped. ● Complete analysis of shell strength, design basis and margin. ● Determine if shell strengthening is required, engineer methods to do so (if required). ● Develop life-of-plant monitoring program. 	<ul style="list-style-type: none"> ● Implement life-of-plant monitoring program. ● If required, strengthen shell.
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(JUNE 1991 - NOVEMBER 1992)

ANALYSIS

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 - o Submitted to the NRC
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o	One Pre-Primer Coat	-	Devos 167
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o	Cauked InnerRadius	-	Devos 142

SUMMARY OF 14R OUTAGE UT THICKNESS MEASUREMENTS

(TAKEN FROM INSIDE DRYWELL)

	Vessel Thickness (inches)			
Drywell Region	As Designed (inches)	Minimum Required at 1.1 Smc (inches)	Current Thinnest (12/92) (inches)	Previous Thinnest (7/91) (inches)
Cylinder	0.640	0.580	0.614	0.612
Upper Sphere (El. 51' to 65')	0.722	0.650	0.691	0.695
Middle Sphere (El. 23' to 51')	0.770	0.670	0.743	0.745
Sand Bed	1.154	0.736	0.800	0.803

SAND BED REGION

14R OUTAGE DRYWELL CONDITION

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EVALUATION OF SHELL THICKNESS (UT) MEASUREMENTS

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- Local nature of individual thickness measurements
- Adjustment of buckling capacity to account for general wall thickness found
- Adjustment of buckling capacity to account for local thinned area

CONCLUSIONS

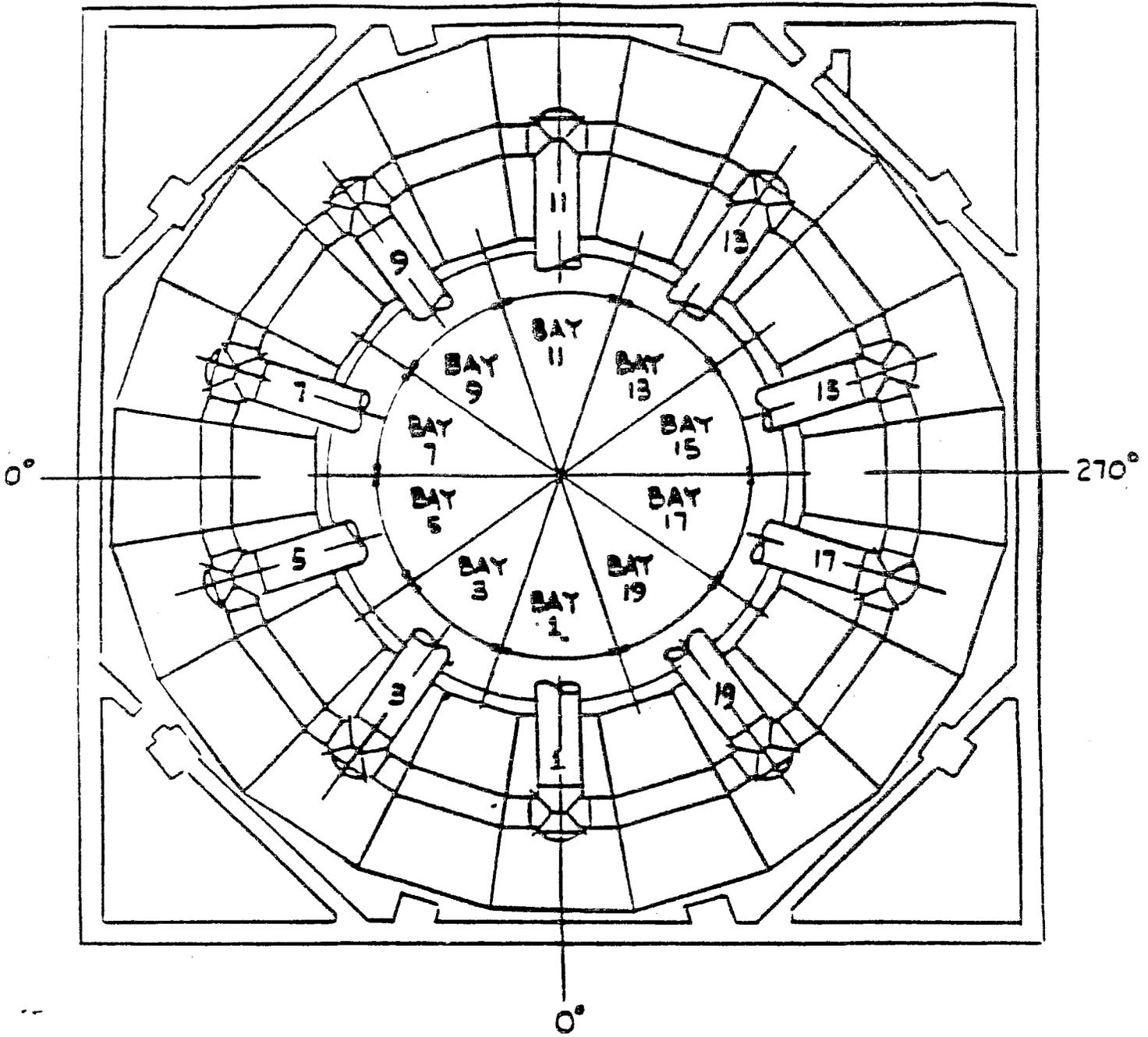
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FUTURE DRYWELL INSPECTIONS/PLAN

- PRIORITY 1 & 2 UT READINGS FROM INSIDE DRYWELL DURING 15R (1994)
- VISUAL INSPECTION OF COATING IN BAYS 1 & 11 AT BEGINNING OF 15R
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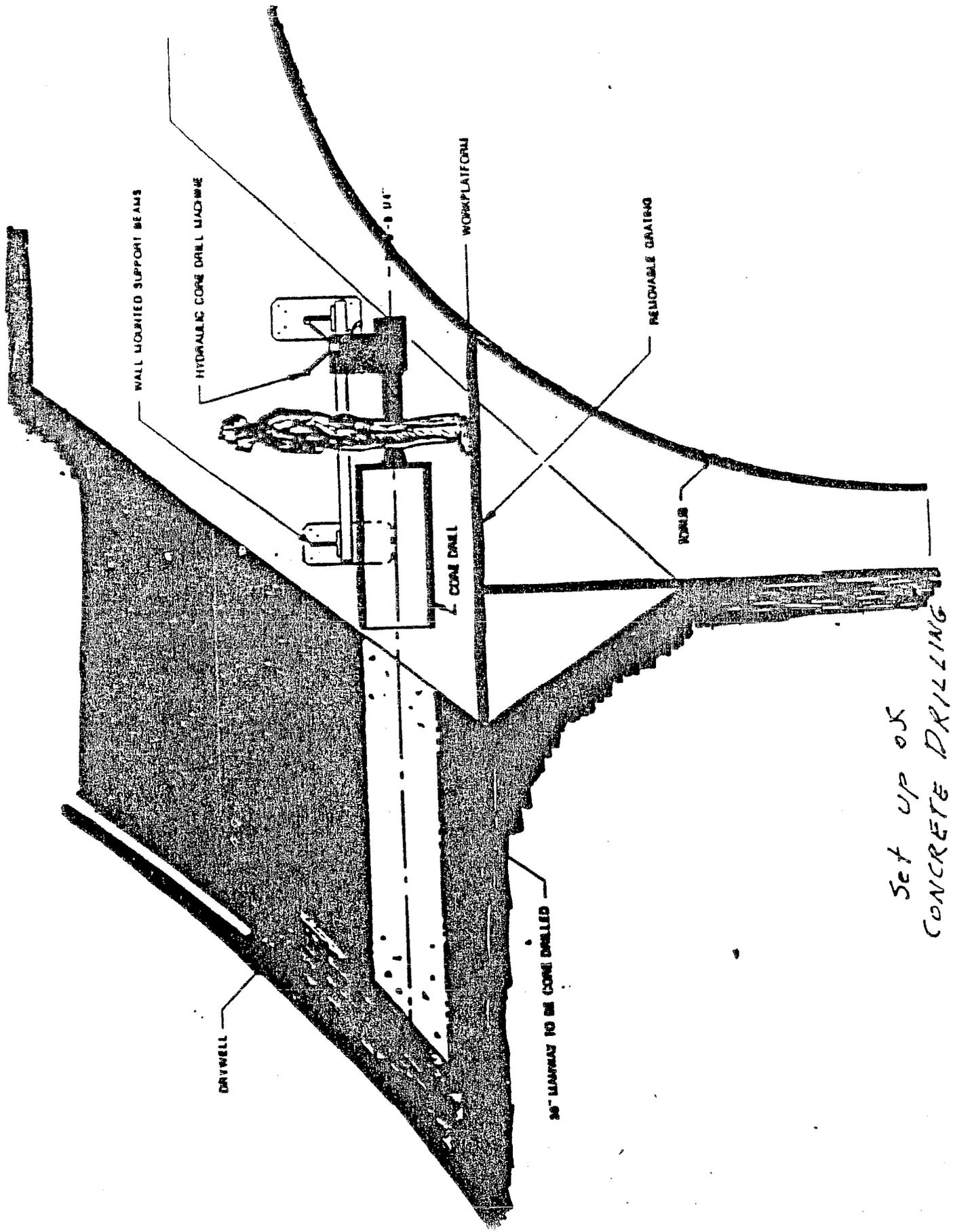


180°



PLAN VIEW

TORUS & DRYWELL



*Set up of
CONCRETE DRILLING
EQUIPMENT*



GPU Nuclear Corporation
 One Upper Pond Road
 Parsippany, New Jersey 07054
 201-316-7000
 TELE X 136 482
 Writer's Direct Dial Number

September 15, 1995
 C321-95-2235
 5000-95-0088

U. S. Nuclear Regulatory Commission
 Attn: Document Control Desk
 Washington, DC 20555

Gentlemen:

Subject: Oyster Creek Nuclear Generating Station (OCNGS)
 Docket No. 50-219
 Facility Operating License No. DPR-16
 Drywell Corrosion Monitoring Program

- References:
- (1) GPU Nuclear Letter C321-92-2163, "Oyster Creek Drywell Containment," May 26, 1992.
 - (2) NRC Letter dated June 30, 1992, "Oyster Creek Drywell Containment."
 - (3) GPU Nuclear Letter C321-93-2100, "Oyster Creek Drywell Inspection," March 25, 1993.

In compliance with Item (3) of References 1 and 2, and Reference 3, GPU Nuclear has (1) assessed the condition of the drywell based upon inspections performed at Oyster Creek during the 15R Outage and is (2) submitting an extended drywell inspection plan for the remaining life of the plant. GPU Nuclear remains committed, as stated in Reference 1, to continue taking drywell thickness measurements for the life of the plant.

Through the 15R Outage, GPU Nuclear's drywell containment vessel thickness monitoring program, Item (2) of References 1 and 2, consisted of ultrasonic thickness (UT) measurements taken at the sandbed region and upper elevations (cylinder, sphere) of the drywell during refueling outages and other outages of opportunity.

Assessment of the most recent UT data taken during the 15R Outage has determined that there is no evidence of ongoing corrosion in the upper elevations of the drywell and that corrosion has been arrested in the sandbed region of the drywell which was cleaned of sand and rust and coated during the 14R Outage (December 1992). The attached table summarizes the 15R Outage UT inspection results for both the sandbed region and upper

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 PDR ADDCK 05600219
 PDR

surfaces indicates that, after 21 months of service, the coating is performing satisfactory with no signs of deterioration such as blisters, flakes, discoloration, etc.

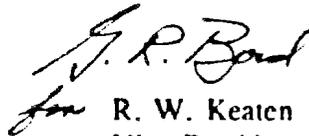
GPU Nuclear's extended inspection plan for the Oyster Creek drywell containment vessel covers both the upper elevations of the drywell and the coated sandbed region.

For the upper elevations of the drywell, this program will perform UT measurements during the 16R Outage (currently scheduled to begin September, 1996) and, as a minimum, again during every other refueling outage (18R, 20R, etc.). The UT measurement locations will be the nine areas identified as most severely corroded. Assessment of the most recent UT data taken during the 15R Outage has determined (and will be reconfirmed by the 16R inspections) that there is no evidence of ongoing corrosion in the upper elevations of the drywell. After each inspection, a technical assessment of the drywell condition will be made, any appropriate corrective action will be taken, and any necessary additional inspections would be scheduled to ensure that drywell integrity is maintained for the remaining life of the plant.

For the sandbed region of the drywell, this program will perform visual inspection of the external epoxy coating during the 16R Outage and, as a minimum, again during the 18R Outage (year 2000). The epoxy coating has an estimated life of 8-10 years which makes the current projected end of life between December, 2000 and December, 2002. Coating inspection shall be by direct (physical) and/or remote methods on a sample basis. Based upon these inspections, a technical assessment of the coating condition will be made, any appropriate corrective action will be taken, and the need for additional (post 18R) inspections will be determined to ensure that drywell integrity is maintained for the remaining life of the plant. In addition, while not technically required based upon the performance of the epoxy coating, UT thickness measurements will be taken one more time in the sandbed region during the 16R Outage, to the same extent as the 15R Outage inspections.

In compliance with Reference 3, GPU Nuclear remains committed to inform the NRC prior to implementing any changes to this drywell inspection program.

Very truly yours,



for R. W. Keaten
Vice President and Director
Technical Functions

Attachment
RTZ/plp

c: Administrator, Region I
Senior Resident Inspector
Oyster Creek NRC Project Manager

TABLE 1

ACCEPTABLE MEAN DRYWELL THICKNESSES				
15R OUTAGE INSPECTION DRYWELL THICKNESSES				
LOCATION	NOMINAL	UT MEASURED MINIMUMS (1)	CODE REQUIRED	MARGIN
Sandbed Region	1.154"	0.806"	.736" (2)	.070" (3)
Sphere (el. 50' - 2")	0.770"	0.733"	0.541"	0.192"
Sphere (el. 51' - 10")	0.722"	0.695"	0.518"	0.177"
Sphere (el. 60' - 11")	0.722"	0.709"	0.518"	0.191"
Cylinder (el. 87' - 5")	0.640"	0.613"	0.452"	0.161"

- (1) Thinnest Location as measured during the 15R outage, September, 1994.
- (2) Controlled by buckling.
- (3) Corrosion arrested (sandbed region coated in 14R outage).

ASME Code, Section XI
10CFR50.55a

February 16, 2005

2130-05-20037

PETITIONERS' EXHIBIT 7

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Oyster Creek Generating Station
Facility License No. DPR-16
Docket No. 50-219

Subject: Oyster Creek Generating Station Refueling Outage 20 (1R20) Inservice Inspection (ISI) Summary Report

Reference: United States Nuclear Regulatory Commission (USNRC) letter dated November 12, 2004, "Oyster Creek Nuclear Generating Station – Alternative Repair of Control Rod Drive Housing Interface With Reactor Vessel (TAC NO. MC1099)"

Submittal of the Owner's Data Reports for Inservice Inspections is required by the American Society of Mechanical Engineers (ASME) Code, Section XI. Attachment 1, "NIS-1 Owner's Data Report for Inservice Inspections," covers the Oyster Creek Generating Station Refueling Outage 20 (1R20) examinations conducted between October 28, 2002 and November 22, 2004. The report includes the first period, of the Fourth Inservice Inspection (ISI) interval examinations performed in accordance with the ASME Code, Section XI, 1995 Edition with 1996 Addenda.

Attachment 2, "Form NIS-1 for Containment ISI Program - IWE," includes the NIS-1 Form covering OCGS Data Report for Containment Inservice Inspections where documentation was completed between October 28, 2002 and November 22, 2004. The report includes the second period of the First Inservice Inspection (ISI) interval examinations performed in accordance with the ASME Code, Section XI, 1992 Edition with 1992 Addenda.

Attachment 3, "Form NIS-2 Owner's Report for Repairs or Replacements," includes the NIS-2 Forms covering repairs or replacements where documentation was completed between October 28, 2002 and November 22, 2004.

In the Referenced letter, the U. S. Nuclear Regulatory Commission (NRC) provided approval of a proposed alternative to 10 CFR 50.55a(g) concerning the roll repair of control rod drive penetrations at Oyster Creek Generating Station. In the Safety Evaluation Report, the NRC requested that we provide the results of the VT-2 inspections

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of all CRD penetrations, including the rolled penetrations, and the inspection results of the housing inside diameter of drive 46-39, performed this past outage.

The VT-2 inspection of all the drives was performed during shutdown and startup did not identify any leakage. Additionally, the UT inspection of the inside diameter of control rod drive housing 46-39 did not identify any indications.

If you should have any questions, please contact the Oyster Creek ISI Coordinator, Mr. Greg Harttraft at 609-971-2287.

Sincerely,

A handwritten signature in black ink, appearing to read "C. N. Swenson". The signature is stylized with large, sweeping loops.

C. N. Swenson
Vice President, Oyster Creek Generating Station

CNS/DIF

Enclosure: OCGS ISI Post 1R20 Outage Summary Report

cc: S. J. Collins, USNRC, Regional Administrator, Region I
P. S. Tam, USNRC, Senior Project Manager, OCGS
R. J. Summers, USNRC, Senior NRC Resident Inspector, TMI
Scott Laley, Hartford Steam Boiler I&I Co. of CT
File No. 05014

Attachment 1

Oyster Creek Generating Station

**ISI Post 1R20 Outage
Summary Report**

FORM NIS-1 (back)

- 8. Examination Dates 10/28/02 to 11/22/04
- 9. Inspection Period Identification: First Period
- 10. Inspection Interval Identification: Fourth Inspection Interval
- 11. Applicable Editions for Section XI 1995 Addenda 1996
- 12. Date / Revision of Inspection Plan: 9/30/04 Rev. 1
- 13. Abstract of Examinations and Tests. Include a list of examinations and tests and a statement concerning status of work required for the Inspection Plan. See attached
- 14. Abstract of Results of Examinations and Tests. See attached
- 15. Abstract of Corrective Measures. See attached

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (if applicable) None Expiration Date N/A
Date 2/9/05 Signed AmerGen Energy Co., L.L.C. By Greg Hawthorn ISI PROG. ENG.

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of New Jersey and employed by HSB of CT have inspected the components described in this Owners' Data Report during the period 10/28/02 to 11/22/04 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Date Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Scott R. Galey Commissions NB 9364 (I) (N) NJ766
Inspector's Signature National Board, State, Province and No.
Date 2/10/2005

FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS
Form NIS-1 for ISI Program

13. Abstract of Examinations and Tests

This submittal is for the ISI examinations performed during the Oyster Creek Generating Station 1R20 refuel outage. This is Oyster Creek's first submittal for the fourth inspection interval. The exams were completed in accordance with the 1995 Edition of the ASME Code with 1996 Addenda.

One hundred sixty three (163) exams were completed on Class 1 and 2 systems for ASME Section XI requirements, ten (10) exams were completed for augmented requirements, and seventeen (17) exams were completed for Generic Letter 88-01 requirements. In addition, visual and UT examinations were completed on the drywell and torus in accordance with ASME Section IWE (Containment Program). In summary, all examinations were completed successfully.

Refueling Outage Inspection Results for the CRD Housing Inspections

During the 1R20 outage, AmerGen performed inspections of the CRD housings to identify any leakage from the RPV bottom head penetrations. No leakage was observed during the drywell 1000 lb. VT-2 inspection performed during the initial shutdown. During the outage, undervessel inspections were performed on all CRD housings during set-up activities for CRD exchange (RPV vented) and no stub tube leakage was identified from the RPV bottom head.

Based on a target of opportunity for access to one of the roll repaired CRD housings, the CRD housing bore at core location 46-39 was UT inspected. Both the roll repair area and the J weld area above it were UT inspected with no indications. Undervessel VT-2 inspections were also performed during the RPV pressure test at approximately 1000 lbs and no stub tube leakage was found (WO R2032133). A final undervessel inspection was performed during the plant startup drywell 1000lb. Inspection and no stub tube leakage was found.

14. Abstract of Results of Examinations and Tests

Welds categorized as B-J, C-F-1 and C-F-2 were examined to the requirements of PDI, in compliance with the amended requirements of 10CFR50.55a for implementation of appendix VIII of the ASME Section XI, 1995 Edition with 1996 Addenda.

The reactor pressure test of the reactor coolant pressure boundary was completed at the end of the outage. Leakage at bolted flange connections was observed on some control rod drives (CRDs) and this was evaluated in accordance with Code Case N-566-1. The alternative pressure test requirements of Code Case N-416-1 were utilized for welded repairs or installation of replacement items by welding.

There were challenges identified during this inspection on the Core Spray system piping supports and the NU-2-2 weld on the shutdown cooling system. In the area of supports, scope expansion (11 extra support exams) was required due to unacceptable as-found conditions on rod hangers. The NU-2-2 dissimilar metal weld in the drywell was initially identified by manual UT with a flaw. However, upon re-examination of this weld with improved and automated UT techniques (SMART UT) the flaw was no longer identified and a weld overlay repair was not required.

15. Abstract of Corrective Measures

See Table 2 for ISI and in-vessel visual inspection (IVVI) corrective measures. For ISI components not associated with IVVI no indications were detected by the examinations, and there was no through-wall leakage identified at the pressure boundary. Table 3 provides the statistics for ISI examinations completed during the 1R20 outage. Refer to Table 4 for the ISI examinations that were completed that did not meet ASME code coverage requirements. The limitations were due to nozzle geometry, restricted access, and one sided examinations due to attached nozzle or valve. A relief request will be submitted by the end of the interval for approval of these limited coverage examinations. See Table 5 for system pressure test results.

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA
 2.PLANT: OYSTER CREEK GENERATING STATION,
 ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731
 3.UNIT: OYSTER CREEK GENERATING STATION

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE
 5.COMMERCIAL SERVICE DATE: 12/23/1969
 6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: B-A							
B1.21	ISI	NR02 2-574 WELD CLOSURE HEAD CIRC WELD	UT	221	CIRC	1	09-Nov-04
B1.40	ISI	NR02 1-574 WELD REACTOR HEAD TO FLANGE	SUR	221	CIRC	1	05-Nov-04
CATEGORY: B-D							
B3.100	ISI	NR02 3-565C WELD N1C RECIRC OUTLET NOZZLE C-LOOP	UT	221	NOZZ	1	07-Nov-04
B3.100	ISI	NR02 3-565D WELD N1D RECIRC OUTLET NOZZLE D-LOOP	UT	221	NOZZ	1	08-Nov-04
B3.90	ISI	NR02 3-565C WELD N1C RECIRC OUTLET NOZZLE C-LOOP	UT	221	NOZZ	1	07-Nov-04
B3.90	ISI	NR02 3-565D WELD N1D RECIRC OUTLET NOZZLE D-LOOP	UT	221	NOZZ	1	08-Nov-04
CATEGORY: B-F							
B5.10	ISI	NR02 1-566A WELD N5A ISOLATION COND. NOZZLE SAFE E	SUR	221	CIRC	1	04-Nov-04
B5.10	ISI	NR02 1-566A WELD N5A ISOLATION COND. NOZZLE SAFE E	UT	221	CIRC	1	09-Nov-04
B5.10	ISI	NR02 1-566B WELD N5B ISOLATION COND. NOZZLE SAFE E	UT	221	CIRC	1	05-Nov-04
B5.10	ISI	NR02 1-566B WELD N5B ISOLATION COND. NOZZLE SAFE E	SUR	221	CIRC	1	04-Nov-04
B5.10	ISI	NR02 2-576 WELD N7A NOZZLE TO FLANGE	UT	221	CIRC	1	09-Nov-04
B5.10	ISI	NR02 4-565C WELD N1C RECIRC OUTLET NOZZLE SAFE EN	SUR	221	CIRC	1	08-Jan-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA
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 ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731
 3.UNIT: OYSTER CREEK GENERATING STATION

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE
 5.COMMERCIAL SERVICE DATE: 12/23/1969
 6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID.	COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: B-F								
B5.10	ISI	NR02	4-565C WELD N1C RECIRC OUTLET NOZZLE SAFE EN	UT	221	CIRC	1	07-Nov-04
B5.10	ISI	NR02	4-565D WELD N1D RECIRC OUTLET NOZZLE SAFE EN	SUR	221	CIRC	1	07-Nov-04
B5.10	ISI	NR02	4-576 WELD N7B FLANGE TO NOZZLE	UT	221	CIRC	1	09-Nov-04
CATEGORY: B-G-1								
B6.10	ISI	NR02	573-02 BOLT CLOSURE NUT (1-32)	VT-1	221		1	05-Nov-04
B6.50	ISI	NR02	573-07 BOLT CLOSURE WASHER BOTTOM (1-32)	VT-1	221		1	05-Nov-04
B6.50	ISI	NR02	573-08 BOLT CLOSURE WASHER UPPER (1-32)	VT-1	221		1	05-Nov-04
CATEGORY: B-G-2								
B7.50	ISI	216-BG-001	BOLT BOLTING ON 6"FLANGE RX SIDE REM	VT-1	216		1	15-Nov-04
B7.50	ISI	216-BG-002	BOLT BOLTING ON 2"FLANGE PIPE SIDE RE	VT-1	216		1	15-Nov-04
B7.50	ISI	216-BG-003	BOLT BOLTING ON TOP 2" FLANGEAT CAVIT	VT-1	216		1	15-Nov-04
B7.50	ISI	216-BG-004	BOLT BOLTING ON BOTTOM 2" FLANGE ON C	VT-1	216		1	15-Nov-04
B7.50	ISI	221-BG-001	BOLT FLANGE ON RX VESSEL HEAD	VT-1	221		1	15-Nov-04
B7.50	ISI	221-BG-002	BOLT FLANGE AT CAVITY FLOOR	VT-1	221		1	15-Nov-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA
 2.PLANT: OYSTER CREEK GENERATING STATION,
 ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731
 3.UNIT: OYSTER CREEK GENERATING STATION

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE
 5.COMMERCIAL SERVICE DATE: 12/23/1969
 6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: B-J							
B9.11	ISI	NZ-3-0042 PIPE TO ELBOW	WELD	UT	212	CIRC 1	11-Nov-04
B9.11	ISI	NZ-3-0073 PIPE TO ELBOW	WELD	UT	212	CIRC 1	12-Nov-04
B9.11	ISI	NZ-3-0086 ELBOW TO PIPE	WELD	UT	212	CIRC 1	10-Nov-04
B9.11	ISI	NZ-3-0089 ELBOW TO PIPE	WELD	UT	212	CIRC 1	10-Nov-04
B9.11	ISI	NZ-3-0095R1 PIPE TO REDUCER(WELD REPAIRED B	WELD	UT	212	CIRC 1	07-Nov-04
B9.11	ISI	NU-1-0007 ELBOW TO PIPE	WELD	UT	214	CIRC 1	12-Nov-04
B9.11	ISI	NU-2-0002 ELBOW TO PIPE (BI-METALIC WELD)	WELD	UT	214	CIRC 1	12-Nov-04
B9.11	ISI	NU-2-0022 ELBOW TO PIPE	WELD	UT	214	CIRC 1	14-Nov-04
B9.11	ISI	NU-3-0002 ELBOW TO PIPE	WELD	UT	214	CIRC 1	13-Nov-04
B9.11	ISI	NU-3-0004 ELBOW TO PIPE	WELD	UT	214	CIRC 1	13-Nov-04
B9.11	ISI	NG-C-0001A SAFE END TO ELBOW	WELD	UT	223	CIRC 1	10-Nov-04
B9.11	ISI	NG-C-0002 ELBOW TO PIPE	WELD	UT	223	CIRC 1	12-Nov-04
B9.11	ISI	NG-D-0001A SAFE END TO ELBOW	WELD	UT	223	CIRC 1	10-Nov-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE

2.PLANT: OYSTER CREEK GENERATING STATION,
ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731

5.COMMERCIAL SERVICE DATE: 12/23/1969

3.UNIT: OYSTER CREEK GENERATING STATION

6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: B-J							
B9.11	ISI	MS-1-0030 PIPE TO ELBOW	WELD	UT	411	CIRC 1	17-Nov-04
B9.11	ISI	MS-1-0050 SAFE END TO ELBOW	WELD	UT	411	CIRC 1	04-Nov-04
B9.11	ISI	RF-2-0014 ELBOW TO PIPE	WELD	UT	422	CIRC 1	06-Nov-04
B9.21	ISI	RHC-HS-1 REDUCER TO FLANGE PIPE SIDE	WELD	SUR	216	CIRC 1	08-Nov-04
B9.21	ISI	NG-B-0009 PIPE TO FLANGE (DECON CONN.)	WELD	SUR	223	CIRC 1	07-Nov-04
B9.21	ISI	NC-4-0017 PIPE TO ELBOW	WELD	SUR	225	CIRC 1	10-Nov-04
B9.31	ISI	MS-1-0057 PIPE TO BRANCH	WELD	SUR	411	BRNCH 1	09-Nov-04
B9.31	ISI	MS-1-0059 PIPE TO BRANCH	WELD	SUR	411	BRNCH 1	09-Nov-04
B9.32	ISI	NG-D-0017 PIPE TO BRANCH	WELD	SUR	223	BRNCH 1	06-Nov-04
B9.32	ISI	NG-E-0009 ELBOW TO BRANCH	WELD	SUR	223	BRNCH 1	09-Nov-04
B9.40	ISI	NP-2-0006 ELBOW TO PIPE SOCKET	WELD WELD	SUR	213	SOCKT 1	09-Nov-04
B9.40	ISI	NP-2-0017 ELBOW TO PIPE SOCKT	WELD	SUR	213	SOCKT 1	10-Nov-04
B9.40	ISI	NP-2-0018 PIPE-TO-ELBOW SOCKT	WELD	SUR	213	SOCKT 1	10-Nov-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INS

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA
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OUTAGE 1R20
TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: B-J							
B9.40	ISI	RHC-2-0032 WELD ELBOW TO PIPE SOCKT	SUR	216	SOCKT	1	11-Nov-04
B9.40	ISI	NG-D-0024 WELD PIPE TO ELBOW	SUR	223	SOCKT	1	06-Nov-04
B9.40	ISI	NG-D-0032 WELD VALVE V-37-0044 TO PIPE	SUR	223	SOCKT	1	07-Nov-04
B9.40	ISI	NG-E-0039 WELD PIPE TO PIPE	SUR	223	SOCKT	1	09-Nov-04
B9.40	ISI	MS-9-0001 WELD NOZZLE TO PIPE SOCKT	SUR	411	SOCKT	1	07-Nov-04
CATEGORY: B-K							
B10.20	ISI	211-BP-NE-5-H1-0057 WELD ATT NE-5-10" VARIABLE SPRING PIPE HANG	SUR	211	VAR	1	19-Nov-04
B10.20	ISI	211-BP-NE-5-H2-0038 WELD ATT NE-5-10" VARIABLE SPRING PIPE HANG	SUR	211	VAR	1	19-Nov-04
B10.20	ISI	211-BP-NE-5-H5-0059 WELD ATT FRAME (NE-5-H1A)	SUR	211	FRAME	1	11-Nov-04
CATEGORY: B-N-1							
B13.10	ISI	NR02 INTERNAL VESSEL RX VESSEL INTERIOR SURFACES	VT-3	222		1	11-Nov-04
CATEGORY: B-N-2							
B13.20	ISI	NR02 587-03-A WELDED A TOP GUIDE BRACKET	VT-1	222		1	11-Nov-04
B13.30	ISI	NR02 587-01-A WELDED AT STEAM DRYER LUG	VT-3	222		1	11-Nov-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

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 3.UNIT: OYSTER CREEK GENERATING STATION

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE
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 6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20
TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID.	COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY:		B-N-2						
B13.30	ISI	NR02	587-01-B STEAM DRYER LUG	WELDED AT	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-01-C STEAM DRYER LUG	WELDED AT	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-01-D STEAM DRYER LUG	WELDED AT	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-02-A STEAM DRYER BRACKET COLLAR	WELDED A	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-02-B STEAM DRYER BRACKET COLLAR	WELDED A	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-02-C STEAM DRYER BRACKET COLLAR	WELDED A	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-02-D STEAM DRYER BRACKET COLLAR	WELDED A	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-04-A1 FEDWATER SPARGER BRACKET	WELDED	VT-3	222	1	11-Nov-04
B13.30	ISI	NR02	587-04-C1 FEDWATER SPARGER BRACKET	WELDED	VT-3	222	1	11-Nov-04
CATEGORY:		C-C						
C3.10	ISI	241-BP-NQZ-T-H1(P-0111 SLIDE	WELD AT (NQZ-1-H12)	SUR	241	SLIDE	2	05-Nov-04
C3.20	ISI	212-BP-NZ-2-H7-0014 VARIABLE SPRING HANGER	WELD ATT (NZ-2-H9)	SUR	212	VAR	2	02-Nov-04
C3.20	ISI	241-BP-NQ-2-H35-0060 VARIABLE SPRING HANGER	WELD ATT (NQ-2-H19)	SUR	241	VAR	2	03-Nov-04
C3.20	ISI	241-BP-NQ-2-H51-0090 VARIABLE SPRING HANGER	WELD ATT (NQ-2-H8)	SUR	241	VAR	2	05-Nov-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

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 5.COMMERCIAL SERVICE DATE: 12/23/1969
 6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: C-C							
C3.20	ISI	241-BP-NQ-2-H56-0036 VARIABLE SPRING HANGER (NQ-2-H38)	WELD ATT	SUR	241	VAR	2 <u>27-Oct-04</u>
CATEGORY: C-F-1							
C5.11	ISI	NE-1-0233 PIPE TO PIPE STEAM SIDE	WELD	UT	211	CIRC	2 <u>12-Nov-04</u>
C5.11	ISI	NE-1-0234 PIPE TO PIPE STEAM SIDE	WELD	UT	211	CIRC	2 <u>12-Nov-04</u>
C5.11	ISI	NE-1-217 PIPE TO PIPE	WELD	UT	211	CIRC	2 <u>14-Nov-04</u>
C5.11	ISI	NE-1-227 PIPE TO PIPE	WELD	UT	211	CIRC	2 <u>12-Nov-04</u>
C5.11	ISI	NE-2-212 PIPE TO PIPE	WELD	UT	211	CIRC	2 <u>12-Nov-04</u>
CATEGORY: C-F-2							
C5.51	ISI	NZ-1-0058 ELBOW TO PIPE	WELD	UT	212	CIRC	2 <u>14-Nov-04</u>
C5.51	ISI	NZ-1-0058 ELBOW TO PIPE	WELD	SUR	212	CIRC	2 <u>14-Nov-04</u>
C5.51	ISI	NZ-1-0059 PIPE TO ELBOW	WELD	SUR	212	CIRC	2 <u>14-Nov-04</u>
C5.51	ISI	NZ-1-0059 PIPE TO ELBOW	WELD	UT	212	CIRC	2 <u>14-Nov-04</u>
C5.51	ISI	NCS-2-0025 PIPE TO ELBOW	WELD	UT	225	CIRC	2 <u>03-Nov-04</u>
C5.51	ISI	NCS-2-0025 PIPE TO ELBOW	WELD	SUR	225	CIRC	2 <u>03-Nov-04</u>

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE

2.PLANT: OYSTER CREEK GENERATING STATION,
ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731.

5.COMMERCIAL SERVICE DATE: 12/23/1969

6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

3.UNIT: OYSTER CREEK GENERATING STATION

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY:		C-F-2					
C5.51	ISI	NQ-2-0031 PIPE TO FLANGE	WELD	UT	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0031 PIPE TO FLANGE	WELD	SUR	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0035 PIPE TO FLANGE	WELD	SUR	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0035 PIPE TO FLANGE	WELD	UT	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0081 PIPE TO ELBOW	WELD	UT	241	CIRC 2	02-Nov-04
C5.51	ISI	NQ-2-0081 PIPE TO ELBOW	WELD	SUR	241	CIRC 2	02-Nov-04
C5.51	ISI	NQ-2-0088 FLANGE TO PIPE	WELD	UT	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0088 FLANGE TO PIPE	WELD	SUR	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0094 FLANGE TO PIPE	WELD	UT	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0094 FLANGE TO PIPE	WELD	SUR	241	CIRC 2	01-Nov-04
C5.51	ISI	NQ-2-0115 ELBOW TO PIPE	WELD	UT	241	CIRC 2	03-Nov-04
C5.51	ISI	NQ-2-0115 ELBOW TO PIPE	WELD	SUR	241	CIRC 2	03-Nov-04
C5.51	ISI	NQZ-1-0014 PIPE TO PIPE	WELD	UT	241	CIRC 2	13-Nov-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA
 2.PLANT: OYSTER CREEK GENERATING STATION,
 ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731
 3.UNIT: OYSTER CREEK GENERATING STATION

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE
 5.COMMERCIAL SERVICE DATE: 12/23/1969
 6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: C-F-2							
C5.51	ISI	NQZ-1-0014 PIPE TO PIPE	WELD	SUR	241	CIRC 2	13-Nov-04
C5.51	ISI	RF-2-0061 PIPE TO ELBOW	WELD	UT	422	CIRC 2	12-Nov-04
C5.51	ISI	RF-2-0061 PIPE TO ELBOW	WELD	SUR	422	CIRC 2	12-Nov-04
C5.90	AUG	NZ-1-0062A PIPE TO FLANGE SHIELD	WELD	UT	212	ATACH 2	14-Nov-04
C5.90	AUG	NZ-1-0062A PIPE TO FLANGE SHIELD	WELD	SUR	212	ATACH 2	14-Nov-04
C5.90	AUG	NZ-2-0001 FLANGE TO PIPE	WELD	UT	212	CIRC 2	03-Nov-04
C5.90	AUG	NZ-2-0001 FLANGE TO PIPE	WELD	SUR	212	CIRC 2	03-Nov-04
C5.90	AUG	NQ-2-0215 BRANCH TO PIPE	WELD	SUR	241	BRNCH 2	03-Nov-04
CATEGORY: F-A							
F1.10	ISI	211-BP-634-R13-0023 NE-2-10" MECHANICAL PIPE SNUBBER (SUPPORT	VT-3	211	MSNUB 1	09-Nov-04
F1.10	ISI	211-BP-634-R6-0012 NE-2-8" HYDRAULIC PIPE SNUBBER (N	SUPPORT	VT-3	211	HSNUB 2	16-Nov-04
F1.10	ISI	211-BP-NE-5-H3-0060 VARIABLE SPRING HANGER (NE-5-H3)	SUPPORT	VT-3	211	VAR 1	07-Nov-04
F1.10	ISI	211-BP-NE-5-H5-0059 FRAME (NE-5-H1A)	SUPPORT	VT-3	211	FRAME 1	11-Nov-04
F1.10	ISI	211-BP-X-3B-SS-2-0041 A REFER TO 211-BP-X-3B-SS-2-0041	SUPPORT	VT-3	211	HSNUB 1	04-Nov-04

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

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OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY: F-A							
F1.10	ISI	212-BP-461-R2-0051 MECHANICAL SNUBBER (NZ-3-S8)	SUPPORT	VT-3	212	MSNUB 1	<u>09-Nov-04</u>
F1.10	ISI	213-BP-150B-0009 U-BOLT (NP-2-H6)	SUPPORT	VT-3	213	UBOLT 1	<u>10-Nov-04</u>
F1.10	ISI	214-BP-556-R2-0002 MECHANICAL SNUBBER (NU-1-S1)	SUPPORT	VT-3	214	MSNUB 1	<u>07-Nov-04</u>
F1.10	ISI	215-BR-ND-10-R2-0002 GUIDE (ND-10-H2)	SUPPORT	VT-3	215	FRAME 1	<u>07-Nov-04</u>
F1.10	ISI	223-GR-H6(D)-0040 VARIABLE (NG-D-H12)	SUPPORT	VT-3	223	VAR 1	<u>07-Nov-04</u>
F1.10	ISI	223-GR-H6(E)-0051 VARIABLE (NG-E-H12)	SUPPORT	VT-3	223	VAR 1	<u>06-Nov-04</u>
F1.10	ISI	223-GR-SB-1(A)(A)-0003A SWAY BRACE (NG-A-S8)	SUPPORT	VT-3	223	SWYBR 1	<u>05-Nov-04</u>
F1.10	ISI	223-GR-SB-1(A)(A)-0003B SWAY BRACE (NG-A-S9)	SUPPORT	VT-3	223	SWYBR 1	<u>05-Nov-04</u>
F1.20	ISI	211-BP-634-R12-0020 NE-2-8" HYDRAULIC PIPE SNUBBER (N	SUPPORT	VT-3	211	HSNUB 2	<u>26-Oct-04</u>
F1.20	ISI	212-BP-NZ-2-H21-0087 ROD HANGER (NZ-2-H33)	SUPPORT	VT-3	212	ROD 2	<u>29-Oct-04</u>
F1.20	ISI	212-BP-NZ-2-H28-0059 VARIABLE SPRING HANGER (NZ-2-H47	SUPPORT	VT-3	212	VAR 2	<u>02-Nov-04</u>
F1.20	ISI	212-BP-NZ-2-H7-0014 VARIABLE SPRING HANGER (NZ-2-H9)	SUPPORT	VT-3	212	VAR 2	<u>02-Nov-04</u>
F1.20	ISI	225-SN-PSST-036-2-0026 RIGID STRUT (NCS-2-H19)	SUPPOR	VT-3	225	RGDST 2	<u>26-Oct-04</u>

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA
 2.PLANT: OYSTER CREEK GENERATING STATION,
 ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731
 3.UNIT: OYSTER CREEK GENERATING STATION

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE
 5.COMMERCIAL SERVICE DATE: 12/23/1969
 6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - ISI Program

ITEM NO	REQMNT	COMPONENT ID. COMPONENT DESCRIPTION	METHOD	SYSTEM	TYPE	CODE CLASS	EXAM DATE
CATEGORY:		F-A					
F1.20	ISI	241-BP-NQ-2-H15-0015 A SUPPOR ROD HANGER (NQ-2-H57)	VT-3	241	ROD	2	09-Nov-04
F1.20	ISI	241-BP-NQ-2-H15-0015 B SUPPOR ROD HANGER (NQ-2-H72)	VT-3	241	ROD	2	09-Nov-04
F1.20	ISI	241-BP-NQ-2-H56-0036 SUPPORT VARIABLE SPRING HANGER (NQ-2-H38)	VT-3	241	VAR	2	27-Oct-04
F1.20	ISI	422-1007 SUPPORT (RF-2-H18)	VT-3	422	VAR	2	06-Nov-04
F1.20	ISI	422-1009 SUPPORT VARIABLE (RF-2-H22)	VT-3	422	VAR	2	12-Nov-04
F1.20	ISI	541-1004 SUPPORT ANCHOR (CC-3-H1D)	VT-3	541	ANCHR	2	06-Nov-04
F1.40	ISI	211-0123 V-14-33 SUPPOR VARIABLE SPRING HANGER ON V-14-00	VT-3	211	VAR	1	10-Nov-04
F1.40	ISI	214-BR-H0303-0032 V-17-206 SUPPO VARIABLE SPRING HANGER ON VALV	VT-3	214	VAR	1	26-Oct-04
F1.40	ISI	223-GR-SS-1(C)-0031 A PUMP SUP MECHANICAL SNUBBER (NG-C-S3)	VT-3	223	MSNUB	1	05-Nov-04
F1.40	ISI	223-GR-SS-1(C)-0031 B PUMP SUP MECHANICAL SNUBBER (NG-C-S4)	VT-3	223	MSNUB	1	05-Nov-04
F1.40	ISI	223-GR-SS-2(C)-0032 A PUMP SUP MECHANICAL SNUBBER (NG-C-S5)	VT-3	223	MSNUB	1	05-Nov-04
F1.40	ISI	223-GR-SS-2(C)-0032 B PUMP SUP MECHANICAL SNUBBER (NG-C-S6)	VT-3	223	MSNUB	1	05-Nov-04

TABLE 2

Oyster Creek Generating Station
ISI / IVVI Corrective Measures

Component	Deficiency	References	Disposition
Dryer Tie Bar Repair Area	Small Indication observed from repair area	O2004-3498 ECR 04-00955	Eng Evaluation approved disposition for 1 cycle.
Dryer Lifting Lugs	Small Indications observed on Lock welds	O2004-3536 ECR 04-00955	Eng Evaluation approved use-as-is disposition.
SRM / IRM Dry Tubes	Dry Tubes observed to be Bowed	O2004-3694 ECR 04-00988	One of three bowed dry tubes is a known existing condition.
Top Guide	VT-6 Indication reported smaller than previous inspection	O2004-3747 SIR-04-138	Previous condition still bounded by existing flaw evaluation.
Top Guide	Small Gauge observed in location 26-07 on Top guide	O2004-3751 SIR-04-138	Engineering analysis of a UT detected indication located in the same cell as the gouge bounds the condition.
CSHX #2	Pressure boundary leakage identified	O2004-1153 A/R A2088856 Eval A2088856-03	Weld repair completed.
Core Spray Support	Supports 212-0019, 212-0024 & 212-0026 as-found spring settings found outside (below) the acceptable tolerance range	O2004-3341	Spring can supports were reset to acceptable range.
Core Spray System 1 Piping support 212-018	Core Spray pipe support 212-0018 found loose (rods not in tension).	O2004-3311	This trapeze rod hanger was reset to carry the proper load.
1R20 Nuclear Steam Supply System (NSSS) Leak Results	During the 1R20 NSSS Leak Test - Recordable Indications of leakage were identified	O2004-3946	Technical Evaluation A2084642-09 addressed the Bolted Flange leakage on CRDs in accordance with N-566-1

TABLE 3**STATISTICS FOR ISI EXAMINATIONS****Number of Exams
Completed**

		Period 1	
CATEGORY	TOTAL	1R20	PERCENT
B-A	40	3	
B-D	48	12	
B-F	49	11	
B-G-1	22	3	
B-G-2	13	6	
B-J	199	34	
B-K	29	4	
B-N-1	3	1	
B-N-2	32	11	
C-C	26	5	
C-F-1	29	5	
C-F-2	81	28	
F-A	160	40	
Total	731	163	22%

TABLE 4

ISI LIMITED COVERAGE EXAMS

Category	Item No.	Component ID Description	Method	Percent Coverage
B-A	B1.40	NR02 1-574 WELD REACTOR HEAD TO FLANGE	UT	58%
B-D	B3.100	NR02 3-576 WELD-INNER RADIUS N7B CLOSURE HEAD NOZZLE	UT	88%
B-D	B3.100	NR02 1-576 WELD-INNER RADIUS N7A CLOSURE HEAD NOZZLE	UT	74%
B-D	B3.100	NR02 2-566A WELD-INNER RADIUS N5A ISOLATION COND NOZZLE	UT	79%
B-D	B3.100	NR02 2-566B WELD-INNER RADIUS N5B ISOLATION COND NOZZLE	UT	69%
B-D	B3.90	NR02 2-566A WELD-INNER RADIUS N5A ISOLATION COND NOZZLE	UT	29%
B-D	B3.90	NR02 2-566B WELD-INNER RADIUS N5B ISOLATION COND NOZZLE	UT	22%
B-D	B3.90	NR02 3-565C WELD N1C RECIRC OUTLET NOZZLE	UT	22%
B-D	B3.90	NR02 3-565D WELD N1D RECIRC OUTLET NOZZLE	UT	22%
B-D	B3.90	NR02 1-576 WELD N7A CLOSURE HEAD NOZZLE	UT	76%
B-D	B3.90	NR02 3-576 WELD-INNER RADIUS N7B CLOSURE HEAD NOZZLE	UT	78%
B-F	B5.10	NR02 4-565D WELD N1D RECIRC OUTLET NOZZLE SAFE END	UT	70%
B-K	B10.10	221-0503 B WELD ATTACHMENT RX HORIZ STABILIZER	SUR	PT 35% VT 65%
C-F-2	C5.90	NQ-2-0215 WELD 6" BRANCH TO PIPE	UT	25%
B-J	B9.11	NE-5-0002 WELD SAFE END TO ELBOW (N5B)	UT	50%
B-J	B9.11	NE-5-0022A WELD SAFE END TO ELBOW (N5A)	UT	50%
B-J	B9.11	NG-D-0002 WELD ELBOW TO PIPE	UT	50%

TABLE 5

PRESSURE TESTING RESULTS

The 1R20 class 1 pressure test was completed satisfactorily on 11/19/04 (R2032133). The following is a list of periodic pressure tests conducted since 1R19. All tests were satisfactory.

SYSTEM	CLASS	PM TASK	WO#	COMPLETE
ESW 1	3	PMVT0005	R0805177	4/12/2004
LIQUID POISON	2	PMVT0007	R0805179	3/30/2004
CORE SPRAY 1	2	PMVT0001	R0805173	2/20/2004
RBCCW TO SD CLG	3	PMVT0004	R0805176	5/29/2004
AUG FUEL POOL	3	PMVT0011	R0805183	1/12/2005
RBCCW TO AFPC	3	PMVT0014	R2037578	1/12/2005
SD COOLING	2	PMVT0003	R2026577	10/21/2002
MAIN STEAM	2	PMVT0013	R2025617	12/3/2004
FEEDWATER	2	PMVT0009	R2026873	10/27/2002

Attachment 2

Form NIS-1 for Containment ISI Program - IWE

FORM NIS-1 (back)

- 8. Examination Dates 10/28/02 to 11/22/04
- 9. Inspection Period Identification: Second Period
- 10. Inspection Interval Identification: First Inspection Interval
- 11. Applicable Editions for Section XI 1992 Addenda 1992
- 12. Date / Revision of Inspection Plan: 12/31/01 Rev. 1
- 13. Abstract of Examinations and Tests. Include a list of examinations and tests and a statement concerning status of work required for the Inspection Plan. See attached
- 14. Abstract of Results of Examinations and Tests. See attached
- 15. Abstract of Corrective Measures. See attached

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (if applicable) None Expiration Date N/A
Date 2/9/05 Signed AmerGen Energy Co., L.L.C. By Greg Hauthoff ISI PROG. ENG.

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of New Jersey and employed by HSB of CT have inspected the components described in this Owners' Data Report during the period 10/28/02 to 11/22/04 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Date Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Scott R. Jolley Commissions NB 9364 (I) (N) NJ766
Inspector's Signature National Board, State, Province and No.
Date 2/10/ 2005

FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS
Form NIS-1 for Containment ISI Program - IWE

13. Abstract of Examinations and Tests

Oyster Creek is at the end of the second period of the first inspection interval for containment inspections. These examinations were performed to fulfill the requirements of ASME Section XI, 1992 Edition with 1992 Addenda. Examinations were performed on all accessible areas of the interior and exterior of the drywell and torus suppression chamber. The augmented examination of the drywell liner and exterior sand bed area was also performed.

A list identifying the components inspected has been attached as Table 1.

14. Abstract of Results of Examinations and Tests

Two (2) examinations resulted in recordable conditions and CAPs were initiated. A weld rod was found stuck to the underside of the Drywell head and reported on CAP O2004-3437. A loose locknut was identified on a spare drywell penetration and reported on CAP O2004-3586.

15. Abstract of Corrective Measures

The weld rod that was found stuck (welded) to the underside of the drywell head was visually examined and no indications were found in the area of the arc strike. An engineering evaluation was performed (C2007833-04) that dispositioned the weld rod as use-as-is. This weld rod appeared to be from plant construction. The loose locknut did not affect the seal of the penetration and was reported to maintenance to be tightened.

NIS-1 OWNER DATA REPORT FOR INSERVICE INSPECTION

1.OWNER: AMERGEN ENERGY CO. L.L.C. 200 EXELON WAY, KENNETT SQUARE, PA

4.OWNER CERTIFICATION OF AUTHORIZATION (IF REQUIRED): NONE

2.PLANT: OYSTER CREEK NUCLEAR GENERATING STATION,
ON ROUTE 9, TWO MILES SOUTH OF FORKED RIVER, NJ 08731

5.COMMERCIAL SERVICE DATE: 12/23/1969

3.UNIT: OYSTER CREEK NUCLEAR GENERATING STATION

6.NATIONAL BOARD NUMBER FOR UNIT REACTOR VESSEL: 14895

OUTAGE 1R20

TABLE 1 - Containment ISI Program - IWE

ITEM NO	COMPONENT ID DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	NR01 VESSEL DRYWELL	VT-G	187		MC	11/3/2004
E1.11	NR01 HEAD VESSEL DRYWELL HEAD	VT-G	187		MC	11/3/2004
E1.11	TORUS VESSEL TORUS SUPPRESSION CHAMBER	VT-G	187		MC	11/7/2004
E1.11	X-001 PENETR DRYWELL PERSONNEL AIRLOCK& EQUIPMENT HATCH	VT-G	245	MECH	MC	11/5/2004
E1.11	X-002A PENETR PIPING PENETRATION FOR A MAIN STEAM PIPING (S-49)	VT-G	244	MECH	MC	11/5/2004
E1.11	X-002B PENETR PIPING PENETRATION FOR B MAIN STEAM PIPING (S-49)	VT-G	244	MECH	MC	11/5/2004
E1.11	X-003A PENETR PIPING PENETR FOR EMERGYCONDENSER STEAM SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-003B PENETR PIPING PENETR FOR EMERGYCONDENSER STEAM SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-004A PENETR PIPING PENETRATION FOR RXFEEDWATER SYSTEM (S-50)	VT-G	244	MECH	MC	11/5/2004
E1.11	X-004B PENETR PIPING PENETRATION FOR RXFEEDWATER SYSTEM (S-50)	VT-G	244	MECH	MC	11/5/2004
E1.11	X-005A PENETR PIPING PENETRATION FOR EMERGENCY CONDNSR RETURN	VT-3	244	MECH	MC	11/5/2004

ITEM NO	COMPONENT ID DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-005B PENETR PIPING PENETRATION FOR EMERGENCY CONDNSR RETURN	VT-G	244	MECH	MC	11/5/2004
E1.11	X-006 PENETR PIPING PENETRATION FOR LIQUID POISON SYSTEM	VT-G	244	MECH	MC	11/5/2004
E1.11	X-007 PENETR PIPING PENETRATION FOR SHUTDOWN COOLING SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-008 PENETR PIPING PENETRATION FOR SHUTDOWN COOLING RETURN	VT-G	244	MECH	MC	11/5/2004
E1.11	X-009 PENETR PIPING PENETRATION FOR RWCLEANUP RETURN SYSTEM	VT-G	244	MECH	MC	11/5/2004
E1.11	X-010 PENETR PIPING PENETRATION FOR RWCLEANUP SUPPLY SYSYEM	VT-G	244	MECH	MC	11/5/2004
E1.11	X-011 PENETR ELECT.PENETRATION #59 FORRECIRCULATION PUMPS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-012A PENETR PIPING PENETRATION FOR REACTOR LEVEL INDICATORS	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-012B PENETR PIPING PENETRATION FOR CORE SPRAY SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-013A PENETR PIPING PENETR.FOR CONTROLROD DRIVE WITHDRAW/INSERT	VT-G	244	MECH	MC	11/5/2004
E1.11	X-013B PENETR PIPING PENETR.FOR CONTROLROD DRIVE WITHDRAW/INSERT	VT-G	244	MECH	MC	11/5/2004
E1.11	X-014A PENETR PIPING PENETR.FOR CONTROLROD DRIVE WITHDRAW/INSERT	VT-G	244	MECH	MC	11/5/2004
E1.11	X-014B PENETR PIPING PENETR.FOR CONTROLROD DRIVE WITHDRAW/INSERT	VT-G	244	MECH	MC	11/5/2004
E1.11	X-015 PENETR PIPING PENETRATION FOR INSTRUMENT AIR	VT-G	244	MECH	MC	11/5/2004

ITEM NO	COMPONENT ID DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-016 PENETR PIPING PENETRATION FOR DW OXYGEN SAMPLE LINE	VT-G	244	MECH	MC	11/5/2004
E1.11	X-017 A PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-017 B PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-018 PENETR PIPING PENETR FOR DRYWELLPURGE VENTILATION INTAKE	VT-G	244	MECH	MC	11/5/2004
E1.11	X-019 PENETR PIPING PENETR FOR DRYWELLPURGE VENTILATION EXHAUST	VT-G	244	MECH	MC	11/5/2004
E1.11	X-020A PENETR PIPING PENETRATION FOR DWCLOSED COOLING WTR SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-020B PENETR PIPING PENETRATION FOR DWCLOSED COOLING SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-021 PENETR PIPING PENETRATION FOR DWEDT & SUMP DISCHARGE	VT-G	244	MECH	MC	11/5/2004
E1.11	X-022A PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/8/2004
E1.11	X-022B PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/8/2004
E1.11	X-022C PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/8/2004
E1.11	X-022D PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/8/2004
E1.11	X-022E PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/8/2004
E1.11	X-022F PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/8/2004

ITEM NO	COMPONENT ID	DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-022G	PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/6/2004
E1.11	X-023	PENETR PIPING PENETRATION FOR DEMINERALIZED WATER	VT-G	244	MECH	MC	11/5/2004
E1.11	X-024 A	PENETR ELECT. PENETRATION #3 FORRECIRCULATION PUMPS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-024 B	PENETR ELECT.PENETRATION #20 FORRECIRCULATION PUMPS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-024 C	PENETR ELECTRICAL PENETRATION - (ABANDONED)	VT-G	774	SPARE	MC	11/5/2004
E1.11	X-024 D	PENETR ELECT.PENETRATION #53 FORRECIRCULATION PUMPS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-024 E	PENETR ELECT.PENETRATION #58 FORRECIRCULATION PUMPS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-025	PENETR PIPING PENETRATION FOR HIPRESS SEAL LEAK DETECTION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-026	PENETR ELECTRICAL PENETRATION #7SPARE	VT-G	774	SPARE	MC	11/5/2004
E1.11	X-027 A	PENETR ELECT.PENETRATION #10 FORVALVE OPERATORS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-027 B	PENETR ELECT.PENETRATION #13 FORVALVE OPERATORS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-027 C	PENETR ELECT.PENETRATION #18 FORVALVE OPERATORS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-027 D	PENETR ELECT.PENETRATION #19 FORVALVE OPERATORS	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-028 A	PENETR ELECT.PENETRATION #46 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004

ITEM NO	COMPONENT ID	DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-028 B	PENETR ELECT.PENETRATION #49 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-028 C	PENETR ELECTRICAL PENETRATION#52SPARE WITH TEMPORARY CAP	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-028 D	PENETR ELECTRICAL PENETRATION#58SPARE WITH TEMPORARY CAP	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-029 A	PENETR ELECT.PENETRATION #45 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-029 B	PENETR ELECT.PENETRATION #48 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-029 C	PENETR ELECTRICAL PENETRATION#51SPARE WITH TEMPORARY CAP	VT-G	774	SPARE	MC	11/5/2004
E1.11	X-029 D	PENETR ELECTRICAL PENETRATION#55SPARE WITH TEMPORARY CAP	VT-G	774	SPARE	MC	11/5/2004
E1.11	X-031 A	PENETR ELECT. PENETRATION #9 FORTHERMOCOUPLE CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-031 B	PENETR ELECT.PENETRATION #12 FORMISCELLANEOUS CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-031 C	PENETR ELECT.PENETRATION #15 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-031 D	PENETR ELECT.PENETRATION #17 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-031 E	PENETR ELECT. PENETRATION #8 FORMISCELLANEOUS CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-031 F	PENETR ELECT.PENETRATION #11 FORMISCELLANEOUS CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-031 G	PENETR ELECT.PENETRATION #14 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004

ITEM NO	COMPONENT ID	DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-031	H PENETR ELECT.PENETRATION #16 FORCONTROL ROD POSITION IND.	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	A PENETR ELECT.PENETRATION #27 FORPRM COAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	B PENETR ELECT.PENETRATION #30 FORPRM COAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	C PENETR ELECTRICAL PENETRATION#33SPARE WITH TEMPORARY CAP	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	D PENETR ELECTRICAL PENETRATION#36SPARE WITH TEMPORARY CAP	VT-G	774	SPARE	MC	11/5/2004
E1.11	X-032	E PENETR ELECT.PENETRATION #22 FORSRM/IRM TRIAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	F PENETR ELECT.PENETRATION #24 FORPRM COAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	G PENETR ELECT.PENETRATION #26 FORPRM COAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	H PENETR ELECTRICAL PENETRATION#29SPARE WITH TEMPORARY CAP	VT-G	774	SPARE	MC	11/5/2004
E1.11	X-032	I PENETR ELECTRICAL PENETRATION#32SPARE WITH TEMPORARY CAP	VT-G	774	SPARE	MC	11/5/2004
E1.11	X-032	J PENETR ELECT.PENETRATION #35 FORPRM COAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	K PENETR ELECT.PENETRATION #38 FORPRM PRM COAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	L PENETR ELECT.PENETRATION #41 FORSRM/IRM TRIAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004
E1.11	X-032	M PENETR ELECT.PENETRATION #21 FORPRM COAXIAL CABLE	VT-G	774	ELEC	MC	11/5/2004

ITEM NO	COMPONENT ID		METHOD	SYSTEM	TYPE	CODE	EXAM DATE	
	DESCRIPTION							
E1.11	X-032	N	PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECT.PENETRATION #23 FORPRM COAXIAL CABLE							
E1.11	X-032	P	PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECT.PENETRATION #25 FORPRM COAXIAL CABLE							
E1.11	X-032	Q	PENETR	VT-G	774	SPARE	MC	11/5/2004
	ELECTRICAL PENETRATION#28SPARE WITH TEMPORARY CAP							
E1.11	X-032	R	PENETR	VT-G	774	SPARE	MC	11/5/2004
	ELECTRICAL PENETRATION#31SPARE WITH TEMPORARY CAP							
E1.11	X-032	S	PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECT.PENETRATION #34 FORFOR PRM COAXIAL CABLE							
E1.11	X-032	T	PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECTRICAL PENETRATION#37FOR PRM COAXIAL CABLE							
E1.11	X-032	U	PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECT.PENETRATION #40 FORPRM COAXIAL CABLE							
E1.11	X-033		PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECT.PENETRATION #39 FORPRM CABLE							
E1.11	X-034		PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECT.PENETRATION #42 FORPRM COAXIAL CABLE							
E1.11	X-036		PENETR	VT-G	774	ELEC	MC	11/5/2004
	ELECT.PENETRATION #57 FORTHERMOCOUPLE CABLE							
E1.11	X-037A		PENETR	VT-G	774	SPARE	MC	11/5/2004
	ELEC.PENETRATION #1 SPARE							
E1.11	X-037B		PENETR	VT-G	774	SPARE	MC	11/5/2004
	ELEC.PENETRATION #1 SPARE-ABANDONED							
E1.11	X-037C		PENETR	VT-G	774	SPARE	MC	11/5/2004
	ELEC.PENETRATION #1 SPARE-ABANDONED							
E1.11	X-037D		PENETR	VT-G	774	SPARE	MC	11/5/2004
	ELEC.PENETRATION #1 SPARE-WITH TEMPORARY CAP							

ITEM NO	COMPONENT ID	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
	DESCRIPTION					
E1.11	X-038 A PENETR PIPING PENETRATION FOR RXWIDE RANGE WATER LVL(TOP)	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-038 B PENETR PIPING PENETRATION FOR RXLEVEL CONTROL(STATIC LEG)	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-038 C PENETR PIPING PENETRATION FOR PROTECTION SYSTEM LEVEL	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-038 D PENETR PIPING PENETRATION FOR EMERGENCY CONDENSER STEAM	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-038 E PENETR PIPING PENETRATION FOR EMERGENCY COND.CONDENSATE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-038 F PENETR PIPING PENETRATION FOR RXPROTECTION SYSTEM LEVEL	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-039 A PENETR PIPING PENETRATION FOR RXPROTECTION SYSTEM LEVEL	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-039 B PENETR PIPING PENETRATION FOR RXLEVEL CONTROL(STATIC LEG)	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-039 C PENETR PIPING PENETRATION FOR RXPROTECTION SYSTEM LEVEL	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-039 D PENETR PIPING PENETRATION FOR LEVEL CONTROL(LT-ID0013B)	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040 A PENETR PIPING PENETRATION FOR RXRECIRC LOOPS D/P SUCTION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040 B PENETR PIPING PENETRATION FOR RXRECIRC LOOPS D/P SUCTION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040 C PENETR PIPING PENETRATION FOR RXRECIRC LOOPS D/P SUCTION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040 D PENETR PIPING PENETRATION FOR RXRECIRC LOOPS D/P SUCTION	VT-G	244	INSTR	MC	11/5/2004

ITEM NO	COMPONENT ID	DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-040	E PENETR PIPING PENETRATION FOR RXRECIRC LOOPS D/P SUCTION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040	F PENETR PIPING PENETRATION FOR RXRECIRC LOOP D/P DISCHARGE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040	G PENETR PIPING PENETRATION FOR RXRECIRC LOOP D/P DISCHARGE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040	H PENETR PIPING PENETRATION FOR RECIRC.LOOP D/P DISCHARGE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040	I PENETR PIPING PENETRATION FOR RXRECIRC LOOP D/P DISCHARGE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-040	J PENETR PIPING PENETRATION FOR RXRECIRC LOOP D/P DISCHARGE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-041	A PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041	B PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041	C PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041	D PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041	E PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041	F PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041	G PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041	H PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004

ITEM NO	COMPONENT ID	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
	DESCRIPTION					
E1.11	X-041 I PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-041 J PENETR PIPING PENETRATION (ABANDONED)	VT-G	244		MC	11/5/2004
E1.11	X-042 PENETR PIPING PENETR.FOR REACTORLEVEL INSTRUMENTATION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-043 PENETR PIPING PENETRATION-SPARE	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-044 A PENETR PIPING PENETRATION SPARE TWO TUBES WITH TEMP. CAPS	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-044 B PENETR PIPING PENETRATION SPARE	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-045 A PENETR PIPING PENETR. FOR INCORECALIBRATION MONITOR TUBE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-045 B PENETR PIPING PENETR.FOR INCORE CALIBRATION MONITOR TUBE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-045 C PENETR PIPING PENETR.FOR STEAM FLOW NOZZLE IMPULSE LINES	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-045 D PENETR PIPING PENETR FOR RX COREDIFF.PRESSURE INDICATION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-045 E PENETR PIPING PENETR.FOR INCORE CALIBRATION MONITOR TUBE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-045 F PENETR PIPING PENETR.FOR INCORE CALIBRATION MONITOR TUBE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-045 G PENETR PIPING PENETR. FOR STEAM FLOW NOZZLE IMPULSE LINES	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-046 A PENETR PIPING PENETRATION SPARE WITH PERMANENT CAP	VT-G	244	SPARE	MC	11/5/2004

ITEM NO	COMPONENT ID	DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-046	B PENETR PIPING PENETRATION SPARE WITH PERMANENT CAP	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-047	A PENETR PIPING PENETRATION SPARE	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-047	B PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-047	C PENETR PIPING PENETRATION SPARE	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-047	D PENETR PIPING PENETRATION SPARE	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-049	A PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	B PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	C PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	D PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	E PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	F PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	G PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	H PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-049	I PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004

ITEM NO	COMPONENT ID	DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-049	J PENETR PIPING PENETRATION FOR DWVENT PIPE TO DRYWELL	VT-G	244	MECH	MC	11/7/2004
E1.11	X-050	A PENETR PIPING PENETR. FOR RECIRCPUMP 'E' SEAL CAVITY #2	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	B PENETR PIPING PENETR. FOR RECIRCPUMP 'E' SEAL CAVITY #1	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	C PENETR PIPING PENETR. FOR RECIRCPUMP 'D' SEAL CAVITY #2	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	D PENETR PIPING PENETR. FOR RECIRCPUMP 'D' SEAL CAVITY #1	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	E PENETR PIPING PENETR. FOR RECIRCPUMP 'C' SEAL CAVITY #2	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	F PENETR PIPING PENETR. FOR RECIRCPUMP 'C' SEAL CAVITY #1	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	G PENETR PIPING PENETR. FOR RECIRCPUMP 'B' SEAL CAVITY #2	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	H PENETR PIPING PENETR. FOR RECIRCPUMP 'B' SEAL CAVITY #1	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	I PENETR PIPING PENETR. FOR RECIRCPUMP 'A' SEAL CAVITY #2	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	J PENETR PIPING PENETR. FOR RECIRCPUMP 'A' SEAL CAVITY #1	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050	K PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050	L PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050	M PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004

ITEM NO	COMPONENT ID DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-050 N PENETR PIPING PENETR FOR DRYWELLPRESSURE INSTRUMENTATION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-050 P PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 Q PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 R PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 S PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 T PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 U PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 V PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 W PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 X PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 Y PENETR PIPING PENETRATION(SPARE)	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-050 Z PENETR PIPING PENETR.FOR DRYWELLPRESSURE INSTRUMENTATION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-051A PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/7/2004
E1.11	X-051B PENETR PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS	VT-G	244	MECH	MC	11/7/2004

ITEM NO	COMPONENT ID		METHOD	SYSTEM	TYPE	CODE	EXAM DATE
	DESCRIPTION						
E1.11	X-051C	PENETR	VT-G	244	MECH	MC	11/7/2004
	PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS						
E1.11	X-051D	PENETR	VT-G	244	MECH	MC	11/7/2004
	PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS						
E1.11	X-051E	PENETR	VT-G	244	MECH	MC	11/7/2004
	PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS						
E1.11	X-051F	PENETR	VT-G	244	MECH	MC	11/7/2004
	PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS						
E1.11	X-051G	PENETR	VT-G	244	MECH	MC	11/7/2004
	PIPING PENETRATION FOR DWTO TORUS VACUUM BREAKERS						
E1.11	X-052	PENETR	VT-G	244	MECH	MC	11/3/2004
	PIPING PENETRATION ACCESS MANHOLE						
E1.11	X-053	A PENETR	VT-G	244	MECH	MC	11/6/2004
	PIPING PENETRATION FOR SUPPR VESSEL ACCESS HATCH						
E1.11	X-053	B PENETR	VT-G	244	MECH	MC	11/7/2004
	PIPING PENETRATION FOR SUPPR VESSEL ACCESS HATCH						
E1.11	X-056	PENETR	VT-G	244	INSTR	MC	11/7/2004
	PIPING PENETR. FOR TORUS PRESSURE INSTRUMENTATION						
E1.11	X-057	PENETR	VT-G	244	MECH	MC	11/7/2004
	PIPING PENETRATION FOR O2TORUS SAMPLE SYSTEM						
E1.11	X-059	PENETR	VT-G	244	MECH	MC	11/5/2004
	PIPING PENETRATION- H2/O2 SAMPLE SUPPLY						
E1.11	X-060	A PENETR	VT-G	244	MECH	MC	11/5/2004
	PIPING PENETRATION H2/O2 SAMPLE RETURN						
E1.11	X-060	B PENETR	VT-G	244	SPARE	MC	11/5/2004
	PIPING PENETRATION SPARE WITH PERMANENT CAP						
E1.11	X-060	C PENETR	VT-G	244	SPARE	MC	11/5/2004
	PIPING PENETRATION SPARE WITH PERMANENT CAP						

ITEM NO	COMPONENT ID DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-060 D PENETR PIPING PENETRATION SPARE WITH PERMANENT CAP	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-060 E PENETR PIPING PENETRATION SPARE WITH PERMANENT CAP	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-060 F PENETR PIPING PENETRATION H2/O2 SAMPLE RETURN	VT-G	244	MECH	MC	11/5/2004
E1.11	X-060 G PENETR PIPING PENETRATION H2/O2 SAMPLE RETURN	VT-G	244	MECH	MC	11/5/2004
E1.11	X-061 PENETR PIPING PENETRATION FOR RXCRDHS SUPPLY IN DRYWELL	VT-G	244	MECH	MC	11/5/2004
E1.11	X-062 PENETR PIPING PENETRATION FOR REACTOR HEAD COOLING	VT-G	244	MECH	MC	11/5/2004
E1.11	X-063 A PENETR PIPING PENETRATION FOR CONTAINMENT SPRAY SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-063 B PENETR PIPING PENETRATION(SPARE)IN DRYWELL	VT-G	244	SPARE	MC	11/5/2004
E1.11	X-065 PENETR PIPING PENETRATION FOR VACUUM BREAKER TO ATMOS.	VT-G	244	MECH	MC	11/7/2004
E1.11	X-066 PENETR PIPING PENETRATION FOR CONTAINMENT SPRAY SUPPLY	VT-G	244	MECH	MC	11/5/2004
E1.11	X-067 PENETR PIPING PENETRATION (ILRT TESTING WITH CAP)	VT-G	244	MECH	MC	11/5/2004
E1.11	X-068A PENETR PIPING PENETRATION TO HDR FOR PASS&CORE SPRAY SUCT.	VT-G	244	MECH	MC	11/7/2004
E1.11	X-068B PENETR PIPING PENETRATION TO HDRFOR PASS&CORE SPRAY SUCT.	VT-G	244	MECH	MC	11/7/2004
E1.11	X-069 PENETR PIPING PENETRATION TO HDRFOR PASS&CORE SPRAY SUCT.	VT-G	244	MECH	MC	11/7/2004

ITEM NO	COMPONENT ID	DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-070	PENETR PIPING PENETRATION FOR CORE SPRAY SUPPLY SYSTEM	VT-G	244	MECH	MC	11/5/2004
E1.11	X-071	PENETR PIPING PENETRATION FOR RXRECIRC LOOP SAMPLE LINE	VT-G	244	MECH	MC	11/5/2004
E1.11	X-072	PENETR PIPING PENETRATION FOR MSSTEAM LINE DRAIN	VT-G	244	MECH	MC	11/5/2004
E1.11	X-073 A	PENETR PIPING PENETR FOR EMERGYCONDENSER STEAM SUPPLY	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-073 B	PENETR PIPING PENETRATION FOR EMERGENCY CONDENSER STEAM	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-073 C	PENETR PIPING PENETRATION FOR EMERGY COND'R CONDENSATE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-073 D	PENETR PIPING PENETRATION FOR RXPRESSURE INSTRUMENTATION	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-074	PENETR PIPING PENETRATION FOR RWCU DEMINERALIZER RELIEF LINE	VT-G	244	MECH	MC	11/7/2004
E1.11	X-075 A	PENETR PIPING PENETR.FOR RECIRC.FLOW TRANSM IMPULSE LINE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-075 B	PENETR PIPING PENETR.FOR RECIRC.FLOW TRANSM IMPULSE LINE	VT-G	244	INSTR	MC	11/5/2004
E1.11	X-076A	PENETR PIPING PENETRATION FOR EMRV CONN.TO VENT PIPES	VT-G	244	MECH	MC	11/7/2004
E1.11	X-076B	PENETR PIPING PENETRATION FOR EMRV CONN.TO VENT PIPES	VT-G	244	MECH	MC	11/7/2004
E1.11	X-077	PENETR DRYWELL PENETRATION FOR TORUS DRAIN, V-38-0034	VT-G	244	MECH	MC	11/7/2004
E1.11	X-078	PENETR DRYWELL PENETRATION FOR INSTRUMENTS LT 37 & LT 38	VT-G	244	INSTR	MC	11/7/2004

ITEM NO	COMPONENT ID DESCRIPTION	METHOD	SYSTEM	TYPE	CODE	EXAM DATE
E1.11	X-079A PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004
E1.11	X-079B PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004
E1.11	X-079C PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004
E1.11	X-079D PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004
E1.11	X-079E PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004
E1.11	X-079F PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004
E1.11	X-079G PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004
E1.11	X-079H PENETR BIO SHIELD STABILIZER MANWAY	VT-G	243		MC	11/5/2004

Attachment 3

FORM NIS-2 OWNER'S REPORT FOR REPAIRS AND REPLACEMENTS

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date February 13, 2003
 Name
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 Work Order # C0550852
 Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name
 Authorization No. N/A
200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address
4. Identification of System ISOLATION CONDENSER (211)
5. (a) Applicable Construction Code B31.1 1955 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See page 2							

7. Description of Work Modification of two pipe supports (211-11 and 211-12) in the Isolation Condenser system on 75 ft. Rx Bldg.
8. Tests Conducted NA Hydrostatic ___ Pneumatic ___ Nominal Operating Pressure ___
 Other ___ Pressure ___ psi Test Temp. ___ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: This modification was to restore the piping system to design compliance with ANSI B31.1 for normal operating and seismic loads. The following supports were upgraded and modified in the Isolation condenser system: 211-11 and 211-12.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Greg Hartcraft ISI PROGRAM ENG. Date FEB. 26th, 20 03
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 6-15-01 to 1-30-03 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Joseph A. Kelly
Inspector's Signature

Commissions NB 5478 (I) (N) NJ442
National Board, State, Province, and Endorsements

Date FEBRUARY 27 2003 ~~2002~~

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date February 13, 2003
Name

200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
Address

2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
Name

US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 C0511914
Address Repair Organization P.O. No., Job No., etc.

3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
Name

200 EXELON WAY, KENNETT SQUARE, PA Authorization No. N/A
Address Expiration Date N/A

4. Identification of System SERVICE WATER SYSTEM 531

5. (a) Applicable Construction Code B31.1 1955 Edition, N/A Addenda, N-416-1 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986

6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
See page 2							

7. Description of Work : Replace 20 x 24 x 24 pipe tee downstream of V-3-58/59 valves. Prefab the 20 x 24 x 24 tee and pipe spool.

8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure _____
 Other _____ Pressure 112 psi Test Temp. Ambient °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Replace 20 x 24 x 24 pipe tee downstream of V-3-58/59 valves. Prefab 20 x 24 x 24 tee and pipe spool. The spools pieces in the Fab shop was hydro and the tie in welds in site was pressure tested using Code Case N-416-1.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Gregory Hartman ISI PROGRAM MGR. Date 2/13/03, 2003
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 5/22/02 to 1-28-03 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Joseph S. Libby
Inspector's Signature

Commissions NB 5478 (I) (N) NJ442
National Board, State, Province, and Endorsements

Date FEBRUARY 13, 2003
2002

FORM NIS-2 (Back)

9. Remarks: This work was to inspect V-3-133 check valve and replace if required. The valve was replaced with a new valve. No welding was required.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Greg Northcraft ISI PROGRAM MGR. Date JULY 21, 2003
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 7/15/03 to 7/21/03, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Joseph L. Hubley Commissions NB 5478 (I)(N) NJ442
Inspector's Signature National Board, State, Province, and Endorsements

Date August 14, 2003

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date May 26, 2004
 Name _____
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address _____
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name _____
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2008004
 Address _____ Repair Organization P.O. No., Job No., etc. _____
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name _____ Authorization No. N/A
200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address _____
4. Identification of System Containment Spray System (241)
5. (a) Applicable Construction Code ASME Section VIII Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Containment Spray Heat Exchanger	Perfex Group			CSHX 1-2		Repaired	Yes

7. Description of Work: Weld repair of through-wall crack in nozzle to vessel dissimilar metal weld on CSHX 1-2.
8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Through wall crack was ground out and weld repaired. Code Case N-416-2 was used as PMT. CAP # O2004-1153.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] Date MAY 26th, 2004
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period _____ to _____, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NO 9364 (I)(N) NJ 766
Inspector's Signature ~~NB 5478 (I)(N) NJ 442~~
National Board, State, Province, and Endorsements

Date 5/28 2004
~~2003~~

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date December 1, 2004
 Name

200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address

2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name

US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2008862
 Address Repair Organization P.O. No., Job No., etc.

3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name

200 EXELON WAY, KENNETT SQUARE, PA Authorization No. N/A
 Address Expiration Date N/A

4. Identification of System Main Steam System (411)

5. (a) Applicable Construction Code ASME Section VIII Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995

6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
V-1-7 MSIV	Atwood & Morrill			NS03A		Repaired	No

7. Description of Work: Overhaul of internals of Main Steam Isolation Valve V-1-7 in the drywell.

8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This Form (E00030) may be obtained from the Order Dept., ASME, 345 E. 47th St., New York, NY 10017

FORM NIS-2 (Back)

9. Remarks: Valve bonnet (cover) was replaced. Valve body gasket surface area was weld repaired due to small indication in gasket seating area. Code Case N-416-2 was used as P.M.T.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed



Owner or Owner's Designee, Title

Date

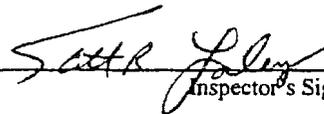
DEC 1

, 2004

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 11/1/04 to 12/15/04 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.



Inspector's Signature

Commissions NB 9364 (I) (N) NJ766

National Board, State, Province, and Endorsements

Date

12/15

2004

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 26, 2005
 Name _____
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address _____
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name _____
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2008343
 Address _____ Repair Organization P.O. No., Job No., etc. _____
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name _____ Authorization No. N/A
200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address _____
4. Identification of System Emergency Service Water (ESW) System (532)
5. (a) Applicable Construction Code ASME B31.1 1955 Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
ESW Piping				Keepfill Line		Replaced	No

7. Description of Work: Replace 2 inch ESW System 1 keepfill line with new AL6XN material piping.
8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure X
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This Form (E00030) may be obtained from the Order Dept., ASME, 345 E. 47th St., New York, NY 10017

FORM NIS-2 (Back)

9. Remarks: Replace 2 inch ESW System 1 keepfill line with new AL6XN material piping. Installed 2 new replacement valves (V-3-948, V-3-698 and V-3-134) of the same material and modified three pipe supports. Refer to ECR 03-00453.-

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Greg Hartley ISI PROGRAM ENG. Date JANUARY 26, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 7/29/04 to 2/3/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Scott R. Foley Commissions NB 9364 (I) (N) NJ766
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/3 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 26, 2005
 Name
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2008344
 Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name
 Authorization No. N/A
200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address
4. Identification of System Emergency Service Water (ESW) System (532)
5. (a) Applicable Construction Code ASME B31.1 1955 Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
ESW Piping				Keepfill Line		Replaced	No
Pipe Support				531-1061		Replaced	No

7. Description of Work: Install new pipe support 531-1061 and replace 2 inch keepfill piping between V-3-941 and V-3-949 with new AL6XN material piping.

8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure X
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Install new pipe support 531-1061 and replace 2 inch keepfill piping between V-3-941 and V-3-949 with new AL6XN material piping. Refer to ECR 03-00454.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Greg Hartman* ISI PROGRAM ENG. Date JANUARY 26, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 7/29/04 to 2/4/05 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Scott R. Foley Commissions NB 9364 (I) (N) NJ766
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/4 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 26, 2005
 Name

200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address

2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name

US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2008345
 Address Repair Organization P.O. No., Job No., etc.

3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name

Authorization No. N/A

200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address

4. Identification of System Emergency Service Water (ESW) System (532)

5. (a) Applicable Construction Code ASME B31.1 1955 Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
ESW Piping						Replaced	No

7. Description of Work: Replace 2 inch piping between V-3-941 and V-3-940 with new AL6XN material piping.

8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure X
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Replace 2 inch piping between V-3-941 and V-3-940 with new AL6XN material piping. Refer to ECR 03-00455.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Ray Northrup ISI PROGRAM ENG. Date JANUARY 26, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 7/29/04 to 2/3/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Seth J. Foley
Inspector's Signature

Commissions NB 9364 (I) (N) NJ766
National Board, State, Province, and Endorsements

Date 2/3 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 18, 2005
 Name
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# R2041057
 Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name
 Authorization No. N/A
200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address
4. Identification of System Emergency Service Water System (532)
5. (a) Applicable Construction Code ASME B31.1 1955 Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Check valve				V-3-133		Replaced	No

7. Description of Work: Replaced 2" ESW keep fill check valve V-3-133 with a new valve.
8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 ½ x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Replaced 2" ESW keep fill check valve V-3-133 with a new valve.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Greg Hawthorn ISI PROGRAM ENG. Date FEBRUARY 4, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 2/4/04 to 2/10/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

S. R. Foley
Inspector's Signature

Commissions NB 9364 (I) (N) NJ766
National Board, State, Province, and Endorsements

Date 2/10 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 18, 2005
Name

200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
Address

2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
Name

US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2007604 / ECR OC 03-00971
Address Repair Organization P.O. No., Job No., etc.

3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
Name

Authorization No. N/A

200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
Address

4. Identification of System Rx Head Vent Cooling System (216)

5. (a) Applicable Construction Code ASME Section I 1965 Edition, B31.1 1983/1984 Addenda, N-416-2 Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/1996 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Cross-Connect 1" and 2" Piping and Supports				System 216		Replacement (modification)	No
Restriction Orifice				RO-678		Replacement	No

7. Description of Work: Install reactor head cross-connect piping between head spray and head vent piping. Work included modification to 4 pipe supports and final VT-3 inspection of a total of 10 pipe supports. Utilized AWS D1.1 for support welding.

8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure X

Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82) This Form (E00030) may be obtained from the Order Dept., ASME, 345 E. 47th St., New York, NY 10017

FORM NIS-2 (Back)

9. Remarks: Installed new cross-connect piping and modified 4 pipe supports associated with this modification. Performed final PT and VT-3 inspections on the piping and supports that were installed / modified. Post Maintenance Testing (PMT) performed during Reactor pressure test prior to start-up.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Greg Hunter ISI PROG. ENG. Date JANUARY 18, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 1/14/04 to 2/4/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Scott R. Foley Commissions NB 9364 (I) (N) NJ766
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/4 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 10, 2005
 Name

200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address

2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name

US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2008993
 Address Repair Organization P.O. No., Job No., etc.

3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name

Authorization No. N/A

200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address

4. Identification of System Core Spray System (212)

5. (a) Applicable Construction Code ASME B31.1 1955 Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda

6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Spring Can Support	Bergen-Patterson			212-0018		Repaired	No

7. Description of Work: Repair hanger support and reset cold load setting on spring can on the Core Spray system.

8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure

Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Replaced bent rod on hanger support and reset spring can cold setting. Performed VT-3 inspection.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] ISI PROGRAM ENG. Date JANUARY 10th, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 11/4/04 to 2/4/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions NB 9364 (I) (N) NJ766
Inspector's Signature National Board, State, Province, and Endorsements

Date 2/4 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 10, 2005
 Name
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2007392
 Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name
200 EXELON WAY, KENNETT SQUARE, PA Authorization No. N/A
 Address Expiration Date N/A
4. Identification of System Main Steam System (411)
5. (a) Applicable Construction Code ASME Section I, 1965 Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
V-1-9 MSIV	Atwood & Morrill			NS04A		Repaired	No

7. Description of Work: Overhaul of internals of Main Steam Isolation Valve V-1-9 in the trunnion room.
8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Replaced both P-21-1A and P-21-1B containment spray pumps with new pumps. Refer to ECR 03-00828.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Greg Hartcraft ISI PROGRAM ENG. Date JANUARY 10th, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 1/03 to 2/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Scott R. Foley
Inspector's Signature

Commissions NB 9364 (I) (N) NJ766
National Board, State, Province, and Endorsements

Date 2/4 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date January 18, 2005
Name
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
Address
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
Name
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# R2049788
Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
Name
200 EXELON WAY, KENNETT SQUARE, PA Authorization No. N/A
Address Expiration Date N/A
4. Identification of System Control Rod Drive System (225)
5. (a) Applicable Construction Code ASME Section I 1965 Edition, N/A Addenda, NA Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Control Rod Drive Assembly	General Electric Company			CRD		Replaced	No

7. Description of Work: Replace Control Rod drive (CRD) assemblies during Refuel outage.
8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure X
Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

(12/82)

This Form (E00030) may be obtained from the Order Dept., ASME, 345 E. 47th St., New York, NY 10017

FORM NIS-2 (Back)

9. Remarks: Replaced 28 CRDs with new or overhauled units. Installed new flange bolting on all CRDs.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed [Signature] ISI PROGRAM ENG. Date JANUARY 18th, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 1/03 to 2/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions NB 9364 (I) (N) NJ766
National Board, State, Province, and Endorsements

Date 2/4 2005

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As required by the Provisions of the ASME Code Section XI

1. Owner AmerGen Energy Co L.L.C. Date February 10, 2005
 Name
200 EXELON WAY, KENNETT SQUARE, PA Sheet 1 of 2
 Address
2. Plant OYSTER CREEK GENERATING STATION Unit OYSTER CREEK
 Name
US ROUTE 9 SOUTH, FORKED RIVER, NJ 08731 WO# C2008225
 Address Repair Organization P.O. No., Job No., etc.
3. Work Performed by AmerGen Energy Co LLC Type Code Symbol Stamp N/A
 Name
 Authorization No. N/A
200 EXELON WAY, KENNETT SQUARE, PA Expiration Date N/A
 Address

4. Identification of System Spent Fuel Pool Cooling System (251)
5. (a) Applicable Construction Code ASME B31.1 1955 Edition, N/A Addenda, NA Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1995 w/ 1996 Addenda
6. Identification of Components Repaired or Replaced and Replacement Components.

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Spent Fuel Pool Cooling Pump				P-18-1A		Replaced	No

7. Description of Work: Replaced Spent Fuel Pool Cooling system pump P-18-1A with a new pump due to internal wear.
8. Tests Conducted Hydrostatic Pneumatic Nominal Operating Pressure X
 Other Pressure _____ psi Test Temp. _____ °F

NOTE: Supplemental sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 ½ x 11 in. (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks: Replaced Spent Fuel Pool Cooling system pump P-18-1A with a new pump due to internal wear and vibration.

Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement (repair or replacement) conforms to the rules of the ASME Code, Section XI.

Type of Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed _____ Date _____, 2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of NEW JERSEY and employed by HSB OF CT of HARTFORD, CONNECTICUT have inspected the components described in this Owner's Report during the period 10/11/04 to 2/10/05, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Satt R. Foley
Inspector's Signature

Commissions NB 9364 (I) (N) NJ766
National Board, State, Province, and Endorsements

Date 2/10 2005



UNITED STATES
 NUCLEAR REGULATORY COMMISSION

REGION I
 475 ALLENDALE ROAD
 KING OF PRUSSIA, PA 19406-1415

June 21, 2005

MEETING SUMMARY

LICENSEE: AMERGEN ENERGY COMPANY, LLC
 FACILITY: OYSTER CREEK GENERATING STATION
 SUBJECT: SUMMARY OF MAY 12, 2005, ANNUAL ASSESSMENT MEETING WITH AMERGEN

On May 12, 2005, at 7:00 p.m., an Annual Assessment Meeting was conducted at the Ocean County New Jersey Administration Building, between the Nuclear Regulatory Commission (NRC) and the AmerGen Energy Company (AmerGen). The NRC requested the meeting with AmerGen management to discuss NRC's assessment of the safety performance of the Oyster Creek Generating Station for calendar year 2004.

The NRC presented and discussed its assessment of the safety performance of Oyster Creek Generating Station for the period between January 1 through December 31, 2004, as documented in our letter dated March 2, 2005. The NRC presentation included background on performance assessment results, an overview of calendar year 2005 planned inspections and oversight, and a discussion of general topics including an update on security. The NRC presentation is available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Website at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room). As part of this regulatory performance meeting, AmerGen presented an overview of their performance at the facility.

Local officials, the public and the press observed this public meeting and were offered the opportunity to communicate with the NRC regarding AmerGen's performance and the role of the agency in ensuring safe plant operations at the end of the meeting. Enclosure 1 to this memorandum provides answers to questions asked at the end of the meeting by members of the public that were not fully answered during the meeting.

Sincerely,

A handwritten signature in black ink, appearing to read "Ronald R. Bellamy".

Ronald R. Bellamy, Chief
 Projects Branch 7
 Division of Reactor Projects

Attachments:
 Enclosure 1: Meeting Questions and Answers

the corrosion in the area above the sandbed, the licensee has concluded that the corrosion rates have been insignificant since adopting the measures to prevent the leakage from the equipment storage area.

Q. When was the containment liner last inspected? What percentage was inspected? Was any section left out?

A. The licensee has visually inspected the coating applied to the liner in the sandbed region in 1996, 2000, and 2004. The visual inspection determined that the coating repair is in very good condition. For regions above the sandbed, ultrasonic inspections have been periodically completed for the areas that exhibited the worst corrosion in 1992, 1996, 2000, and 2004. No significant degradation has been identified for the regions above the sandbed. The licensee monitoring also includes the need to conduct further detailed inspections if other sources of water are identified.

The licensee inspection program focuses on the worst corrosion locations with respect to measured thicknesses of the drywell liner. These locations were selected for inspection based on extensive drywell thickness measurements performed during the initial corrosion investigation (1986 through 1991). These locations exhibited the worst metal loss and were selected to monitor for any long-term degradation.

Q. Please confirm that it is true that Oyster Creek was granted a legal exemption to the timeliness filing provision for license renewal and why?

A. The licensee applied for the exemption in accordance with the provisions of 10 CFR 50.12. The licensee's reasons for applying for the exemption were set forth in the letter, J. A. Benjamin to NRC, dated August 10, 2004 (Accession No. ML042250155). The NRC granted the exemption on December 22, 2004 (Accession No. ML042960164), stating that it allows "the submittal of the Oyster Creek license renewal application with less than 5 years remaining prior to expiration of the operating license while maintaining the protection of the timely renewal provision in 10 CFR 2.109(b)." The exemption was granted contingent on two conditions: (1) on or before July 29, 2005, AmerGen must submit a sufficient license renewal application for OCNGS which the NRC finds acceptable for docketing in accordance with the regulations; and, (2) to ensure timely completion of the review process, AmerGen must provide any requested information as necessary to support the completion of the NRC safety and environmental reviews in accordance with the review schedule issued by the NRC. The technical and legal bases for the NRC approval of this exemption request are described in the exemption, dated December 22, 2004.

Q. Would the NRC license a new nuclear reactor with the same design as Oyster Creek? If not, then why?

A. NRC's issuance of a license is the culmination of a lengthy review of highly technical issues against the current regulatory requirements and guidance documents, and is contingent on the resulting finding that the applicant has met, and will continue to meet,



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20545-0001

November 1, 1995

PETITIONERS' EXHIBIT 9

Mr. John J. Barton
Vice President and Director
GPU Nuclear Corporation
Oyster Creek Nuclear Generating Station
P.O. Box 388
Forked River, NJ 08731

SUBJECT: CHANGES IN THE OYSTER CREEK DRYWELL MONITORING PROGRAM
(TAC NO. M93658)

Dear Mr. Barton:

In a letter dated September 15, 1995, GPU Nuclear Corporation (GPUN) stated that they assessed the condition of the drywell based upon inspections performed at Oyster Creek during the 15R refueling outage (15R) and submitted an extended drywell inspection plan for the remaining life of the plant. GPUN also stated that they remain committed, as stated in their letter of May 26, 1992, to continue taking drywell thickness measurements for the life of the plant.

The staff has reviewed the information provided by GPUN and concludes that changes in the drywell corrosion monitoring program as planned by GPUN is acceptable if GPUN commits to additional inspection within approximately 3 months after discovery of water leakage from the pools above the reactor cavity. Our safety evaluation is enclosed.

Within 30 days of the date of this letter, we request that you provide your intent to perform additional inspection within approximately 3 months after discovery of water leakage.

This requirement affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under P.L. No. 96-511.

Sincerely,

A handwritten signature in black ink, appearing to read "Alexander W. Dromerick".

Alexander W. Dromerick, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosure: Safety Evaluation

cc w/encl: See next page

Mr. John J. Barton
Vice President and Director
GPU Nuclear Corporation
Oyster Creek Nuclear Generating Station
P.O. Box 388
Forked River, NJ 08731

SUBJECT: CHANGES IN THE OYSTER CREEK DRYWELL MONITORING PROGRAM
(TAC NO. M93658)

Dear Mr. Barton:

In a letter dated September 15, 1995, GPU Nuclear Corporation (GPUN) stated that they assessed the condition of the drywell based upon inspections performed at Oyster Creek during the 15R refueling outage (15R) and submitted an extended drywell inspection plan for the remaining life of the plant. GPUN also stated that they remain committed, as stated in their letter of May 26, 1992, to continue taking drywell thickness measurements for the life of the plant.

The staff has reviewed the information provided by GPUN and concludes that changes in the drywell corrosion monitoring program as planned by GPUN is acceptable if GPUN commits to additional inspection within approximately 3 months after discovery of water leakage from the pools above the reactor cavity. Our safety evaluation is enclosed.

Within 30 days of the date of this letter, we request that you provide your intent to perform additional inspection within approximately 3 months after discovery of water leakage.

This requirement affects nine or fewer respondents and, therefore, is not subject to the Office of Management and Budget review under P.L. No. 96-511.

Sincerely,

Original signed by:

Alexander W. Dromerick, Senior Project Manager
Project Directorate I-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000219
P PDR

Enclosure: Safety Evaluation

cc w/encl: See next page

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J. Barton
GPU Nuclear Corporation

Oyster Creek Nuclear
Generating Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20545-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

DRYWELL MONITORING PROGRAM

GPU NUCLEAR CORPORATION

OYSTER CREEK NUCLEAR GENERATING STATION

DOCKET NO. 50-219

GPU Nuclear Corporation (GPUN), the Oyster Creek Nuclear Generating Station licensee, previously, in a letter dated May 26, 1992, committed to conduct ultrasonic thickness (UT) measurements of the drywell at refueling outages (RO) and at other outages of opportunity. The areas to be monitored are the upper elevations and the sandbed regions of the drywell where corrosion had been detected. During the 14th RO (December 1992) the sandbed region of the drywell was cleaned of sand and rust, and coated. During the 15th RO the licensee made UT measurements at the sandbed region and at the upper elevations (cylinder and sphere) of the drywell. In a letter dated September 15, 1995, GPUN stated that they assessed the results of the inspection and determined: (1) there is no evidence of ongoing corrosion in the upper elevations and (2) the corrosion of the sandbed region has been arrested. On the basis of this finding the licensee has proposed to reduce their inspection program as follows:

1. For the upper elevations, UT measurements will be made during the 16th RO (September 1996) and during every second RO, thereafter. After each inspection, a determination will be made if additional inspection is to be performed.
2. For the sandbed region visual inspection of the coating as well as UT measurement of the shell will be made during the 16th RO. The coating will be inspected again during the 18th RO (year 2000). Based on the results of inspection of the coating, determinations will be made for additional inspections.

The licensee has provided a table of UT measurement results from the 15th RO inspection. This table shows the locations of the measurements, the nominal as-constructed thickness, the minimum as measured thickness, the ASME Code required thickness and the corrosion margin available.

On the basis of the information provided, the staff finds the proposed change to the licensee's previous inspection commitment to be reasonable and acceptable. However, since water leaking from the pools above the reactor cavity has been the source of corrosion, the licensee should make a commitment to the effect that an additional inspection of the drywell will be performed about 3 months after the discovery of any water leakage.

Principal Contributor: C. P. Tan

Date: November 1, 1995
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PDR ADOCK 05000219
PDR



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