

WOLF CREEK

NUCLEAR OPERATING CORPORATION

November 10, 2005

Terry J Garrett
Vice President, Engineering

ET 05-0023

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

- References:
- 1) Letter ET 95-0112, dated October 31, 1995, from R. C. Hagan, WCNOC, to USNRC
 - 2) Letter WO 95-0189, dated December 27, 1995, from O. L. Maynard, WCNOC, to USNRC
 - 3) NRC Letter dated February 9, 1996, from USNRC to Neil S. Carns, WCNOC
 - 4) Letter WM 05-0004, dated February 17, 2004, from R. A. Muench, WCNOC, to USNRC

Subject: Docket 50-482: Inservice Inspection Program Second Interval,
Third Period Owner's Activity Reports

Gentlemen:

In References 1 and 2, Wolf Creek Nuclear Operating Corporation (WCNOC) requested use of ASME Code Case N-532 "Alternative Requirements to Repair and Replacement Documentation Requirements and Inservice Summary Report Preparation and Submission as Required by IWA-4000 and IWA-6000" in lieu of current ASME Section XI reporting requirements.

In Reference 3, the USNRC concluded that the proposed alternative to use Code Case N-532 and the clarifications contained within References 1 and 2 provide an acceptable level of quality and safety and approved the use of Code Case N-532 for use at Wolf Creek Generating Station. Code Case N-532 requires that an Owner's Activity Report (Form OAR-1) be prepared and certified upon completion of each refueling outage. Each Form OAR-1 prepared during an inspection period shall be submitted following the end of the inspection period.

The enclosures provide the Owner's Activity Reports for the period January 1, 2003 through September 2, 2005. This timeframe constitutes the third period of the second interval of the WCNOC Inservice Inspection Program. Within this period, Refueling Outages 13 (RF13) and 14 (RF14) occurred. The enclosed Forms OAR-1 (Reports WCRE-10, I2-P3-RF13 and WCRE-10, I2-P3-RF14) correspond to these outages.

Reference 4 identified errors in the First and Second Period Owner's Activity Reports for WCNO's Second Inservice Inspection Interval. These errors involved the incorrect indication of the completion status of the examinations for integral attachments for the pressurizer. In that letter WCNO committed to complete the required examinations for the Second ISI Interval during Refueling Outage 14 (RF14), reflect the correct status of the examinations on the OAR-1 Forms for RF13 and RF14, and submit them to the NRC. Submittal of this letter and the OAR-1 Forms for RF13 and RF14 satisfies that commitment.

The Attachment provides a list of commitments made in the enclosures to this letter.

If you have any questions concerning this matter, please contact me at (620) 364-4084, or Mr. Kevin Moles at (620) 364-4126.

Very truly yours,



Terry J. Garrett

TJG/rig

Attachment
Enclosures

cc: J. N. Donohew (NRC), w/a, w/e
W. B. Jones (NRC), w/a, w/e
B. S. Mallett (NRC), w/a, w/e
Senior Resident Inspector (NRC), w/a, w/e

LIST OF COMMITMENTS

The following table identifies those actions committed to by Wolf Creek Nuclear Operating Corporation in this document. Any other statements in this letter are provided for information purposes and are not considered to be regulatory commitments. Please direct questions regarding these commitments to Mr. Kevin Moles, Manager Regulatory Affairs at Wolf Creek Generating Station, (620) 364-4126.

<u>Regulatory Commitment</u>	<u>Due Date</u>
Submit 10 CFR 50.55a Request(s) for seven Category B-D components in which the examinations performed did not achieve greater than 90% total coverage of the ASME Code required volume. Note: These requests will be submitted to allow for NRC review/approval by September 2, 2006, which is 12 months after the end of ISI Interval 2.	March 2, 2006
Submit 10 CFR 50.55a Request(s) for two Category B-A components in which the examinations performed did not achieve greater than 90% total coverage of the ASME Code required volume. Note: These requests will be submitted to allow for NRC review/approval by September 2, 2006, which is 12 months after the end of ISI Interval 2.	March 2, 2006
Submit 10 CFR 50.55a Request for one Category C-A component in which the examinations performed did not achieve greater than 90% total coverage of the ASME Code required volume. Note: This request will be submitted to allow for NRC review/approval by September 2, 2006, which is 12 months after the end of ISI Interval 2.	March 2, 2006
Submit 10 CFR 50.55a Request(s) for the twelve RI-ISI components in which the examinations performed did not achieve greater than 90% total coverage of the ASME Code required volume. Note: These requests will be submitted to allow for NRC review/approval by September 2, 2006, which is 12 months after the end of ISI Interval 2.	March 2, 2006

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number WCRE-10, I2-P3-RF-13

Owner Wolf Creek Nuclear Operating Corporation, 1550 Oxen Lane Northeast, Burlington, Kansas 66839
(Name and Address of Owner)

Plant Wolf Creek Generating Station, 1550 Oxen Lane Northeast, Burlington, Kansas 66839
(Name and Address of Plant)

Unit No. 1 Commercial service date 9-3-85 Refueling outage no. 13
(If applicable)

Current inspection interval 2nd
(1st. 2nd. 3rd. 4th. other)

Current inspection period 3rd
(1st. 2nd. 3rd)

Edition and Addenda of Section XI applicable to the inspection plan 1989 edition with no addenda

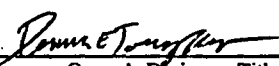
Date and revision of inspection plan WCRE-10 Rev. 5, dated 10-17-2003

Edition and Addenda of Section XI applicable to repairs and replacements, if different than the inspection plan same

CERTIFICATE OF CONFORMANCE

I certify that the statements made in this Owner's Activity Report are correct, and that the examinations, tests, repairs, replacements, evaluations, and corrective measures represented by this report conform to the requirements of Section XI.

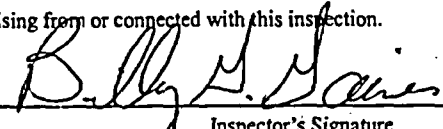
Certificate of Authorization No. N/A Expiration Date N/A
(If applicable)

Signed Dennis E. Tougaw  Engineer Date 10/17/2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Kansas and employed by The Hartford Steam Boiler Inspection and Insurance Company of Connecticut of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, during the period January 1, 2003 to December 2, 2003, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, repairs, replacements, evaluations and corrective measures described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

 Commissions KS#586 A, B, N, I
Inspector's Signature National Board, State, Province, and Endorsements

Date 10/18/05

This form (E00127) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300

TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS
Completion of RF-13 (I-2, P-3)

Examination Category	Total Examinations Required For The Interval	Total Examinations Credited for This Period*	Total Examinations Credited (%) for The Period**	Total Examinations Credited (%) to Date for the Interval	Remarks
B-A	25	2	8	28	Note 10
B-B	5	1	20	80	
B-D	36	1	3	53	Note 11
B-E	113	0	0	66	
B-F	8	0	0	100	Note 9
B-G-1	219	52	24	89	
B-G-2	15	2	13	73	Note 1
B-H	2	1	50	50	Note 12
B-J	45	0	0	100	Note 9
B-K	-	-	-	-	
B-L-2	1	0	0	100	Note 2
B-M-2	7	0	0	71	Note 3
B-N-1	3	0	0	67	
B-N-2	6	-	-	-	
B-N-3	1	-	-	-	
B-O	4	0	0	100	
B-P	Note 4	Note 4	Note 4	Note 4	Note 4
C-A	9	0	0	78	
C-B	7	0	0	71	
C-C	19	1	5	68	
C-D	1	0	0	100	
C-F-1	26	0	0	100	Notes 5, 9
C-F-2	16	0	0	100	Note 9
C-G	5	0	0	40	
C-H	Note 13	Note 13	Note 13	Note 13	Note 13
AUG	325	3	1	98	Note 6
D-A	44	10	23	89	Note 7
D-B	-	-	-	-	Note 7
D-C	-	-	-	-	Note 7
F-A	282	66	23	90	Note 8
RI-ISI	82	25	30	66	Note 9

* This column is interpreted to represent the cumulative number of exams performed to date in this period.

** This column is interpreted to represent the cumulative percentage of scheduled exams for the interval which have completed in this period to date.

- Note 1: For pumps and valves, examinations are limited to components selected for examination under Examination Categories B-L-2 and B-M-2.
- Note 2: Examination is required only when a pump is disassembled for maintenance, repair or examination. Examination is required only once during the inspection interval, thus completed percentages indicated may not be consistent with Table IWB-2412-1 schedule. In RFO-9, RCP-D was disassembled, and the pump casing surfaces were examined.
- Note 3: Examination is required only when a valve is disassembled for maintenance, repair or examination. Examination is required only once during the inspection interval, thus completed percentages indicated may not be consistent with Table IWB-2412-1 schedule.
- Note 4: Visual Examinations (VT-2) are performed as required for a fuel cycle and a 10-year system hydrostatic testing basis. All required VT-2 exams have been performed for the time frame covered by this report.
- Note 5: For simplicity, numbers and percentages are based only on circumferential welds selected for examination. When a circumferential weld is intersected by a longitudinal weld, that longitudinal weld is selected by default in the WCNOG Program. This approach results in greater than the 7.5% sample of longitudinal seam welds being inspected, exceeding the ASME Code requirements.
- Note 6: Includes augmented exams required by NUREG 0800 and RCP Flywheel Exams. In Period 3, WCNOG implemented a risk informed program for the examination required by NUREG 0800 (the high energy line break welds). In Period 3, only the welds selected for examination in this program (that had not been previously examined in the interval) were scheduled for examination. In the column of "Total Examination Required for the Interval", the number shown includes the number of examinations completed prior to implementation of the program, plus the number of selected welds to be examined by the risk informed program to complete 100% examination of the risk informed program selected welds. WCNOG requested and received approval of a license amendment to extend the examination interval of the RCP flywheels to 20 years (this to allow the flywheels to be examined during motor refurbishment). Thus, only 2 flywheel exams are credited to Interval 2.
- Note 7: Numbers and percentages do not include Visual Examinations (VT-2) that are performed as required for 40-month and 10-year system hydrostatic testing basis. All required exams have been performed for this fuel cycle and period.
- Note 8: Wolf Creek has implemented changes that has removed many snubbers and associated supports from plant configuration. The sample percentage requirements of Code Case N-491-2 continue to be met.
- Note 9: Categories B-F, B-J, C-F-1, and C-F-2 welds are now examined under the RI-ISI program. In the safety evaluation of the WCNOG RI-ISI program, the NRC approved WCNOG's proposal that since 33 percent of the ASME Section XI exams had been performed in Period 1 and the first part of Period 2, then 67 percent of the RI-ISI scope would be examined during the remainder of Period 2 and Period 3, so that 100% of selected exams are completed during the interval. In the column of "Total Examination Required for the Interval", the number shown reflects the number of examinations performed on the B-F, B-J, C-F-1, and C-F-2 categories prior to implementation of the RI-ISI program and the number of RI-ISI welds to be examined in the remainder of Period 2 and Period 3.
- Note 10: On previous OAR-1 forms, the percentages were calculated as if the RV flange to shell weld was completely examined (both from the flange face and the shell) in Period 1. Only the examination from flange face should have been credited to the Period 1 examination. The total examination percent complete for the interval is now calculated with this correction. This has been captured in the Wolf Creek corrective action program by PIR 2005-0405.

- Note 11: On previous OAR-1 forms, the percentages were calculated as if the RV outlet nozzle to shell welds were completely examined (both from the shell and bore) in Period 1. Only the examination from the bore and the inner radius exam should have been credited to the Period 1 examinations. The total examination percent complete for the interval is now calculated with this correction. This has been captured in the Wolf Creek corrective action program by PIR 2005-0405.
- Note 12: The examination of the pressurizer skirt weld in Period 1 was examined from the OD only. The percentages on previous OAR-1 forms were calculated as if this examination was complete. WCNOG corrective action document PIR 2004-3194 details that pressurizer skirt weld examination should have been completed from both the OD and the ID in Period 1 prior to claiming for credit for examination of this component.
- Note 13: All Category C-H components have a VT-2 examination performed as a minimum of once per Period. For Period 3, there are 51 surveillance procedures (STS) which will complete the examination requirements for Category C-H. In RF13, 19 of the STS procedures were completed.

TABLE 2
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT
REQUIRED EVALUATION FOR CONTINUED SERVICE

Examination Category	Item Number	Item Description	Flaw Characterization (IWA-3300)	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes or No)
D-A	D.110	Struthers-Wells Horizontal Heat Exchanger – EEG01A, 3/4" Supply Line For EGPI0033, 3/4" Dresser Globe Valve – EGV0028	Through Wall Leak Note A	No

Note A: On 4/13/03, a through wall leak was discovered in the supply line between the heat exchanger and the isolation valve to the pressure indicator. The engineering evaluation performed in accordance with Code Case N-513 determined that the component was suitable for continued service (reference CCP 11058). This evaluation directed that the component be monitored and repaired prior to a specified later date. A Code repair was performed on this component by Repair/Replacement Plan 2003-022 prior to the date specified on the evaluation.

TABLE 3
ABSTRACT OF REPAIRS, REPLACEMENTS, OR CORRECTIVE MEASURES
REQUIRED FOR CONTINUED SERVICE

Code Class	Repair, Replacement, or Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes/No)	Date Complete	Repair/ Replacement Plan Number
2	Replacement	Richmond Engineering Horizontal Heat Exchanger - EBG01 6" Dravo Corporation Pipe Spool - EG06-S020/113 6" Pullman Power Products Pipe Spools: BG03-S026/113, EG06-S021/113	Replaced heat exchanger EBG01, S/N N-2284.10 with S/N 2623. Some of the associated piping was also replaced.	No	08/20/2004	2002-061
3	Replacement	Carrier Corporation Type 28NW Room Cooler Coil - SGL09A	Replace leaking H-bend connecting row 13 tube 7 to row 13 tube 8.	No	05/05/2003	2003-007
3	Replacement	Struthers-Wells Horizontal Heat Exchanger - EEG01A, 3/4" Supply Line For EGPI0033, 3/4" Dresser Globe Valve - EGV0028	Install an encapsulation around the 3/4" line upstream of EGV0028 per the design of CCP 11058.	No	12/05/2003	2003-022
3	Replacement	Diesel Jacket Water Heat Exchanger - EKJ06A	Replace 24 tubes in the jacket water heat exchanger with like design new tubes..	No	12/09/2003	2003-040
3	Replacement	Carrier Corporation Type 28NW Room Cooler Coil - SGL12A	Replace the tube nut and/or the entire H-bend at 7th row from bottom, 5th tube from west.	No	11/02/2003	2003-048
1*	Repair/ Replacement	3/4" pipe between EPV0109 and EP-032-BCA-6"	Repair a thru wall crack in the 3/4" pipe upstream of EPV0109 by cutting the weld at the half coupling upstream of EPV0109, removing a short section of the 3/4" pipe and re-welding. *The 6" line is ASME Code Class 1, however the 3/4" branch line is ASME Code Class 2.	No	01/16/2004	2003-060

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number WCRE-10, I2-P3-RF-14

Owner Wolf Creek Nuclear Operating Corporation, 1550 Oxen Lane Northeast, Burlington, Kansas 66839
(Name and Address of Owner)

Plant Wolf Creek Generating Station, 1550 Oxen Lane Northeast, Burlington, Kansas 66839
(Name and Address of Plant)

Unit No. 1 Commercial service date 9-3-85 Refueling outage no. 14
(If applicable)

Current inspection interval 2nd
(1st. 2nd. 3rd. 4th. other)

Current inspection period 3rd
(1st. 2nd. 3rd)

Edition and Addenda of Section XI applicable to the inspection plan 1989 edition with no addenda

Date and revision of inspection plan WCRE-10 Rev. 8, dated 8-26-2005

Edition and Addenda of Section XI applicable to repairs and replacements, if different than the inspection plan same

CERTIFICATE OF CONFORMANCE

I certify that the statements made in this Owner's Activity Report are correct, and that the examinations, tests, repairs, replacements, evaluations, and corrective measures represented by this report conform to the requirements of Section XI.

Certificate of Authorization No. N/A Expiration Date N/A
(If applicable)

Signed Dennis E. Tougaw *Dennis E. Tougaw* Engineer Date 10/20/2005
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Kansas and employed by The Hartford Steam Boiler Inspection and Insurance Company of Connecticut of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, during the period December 3, 2003 to September 2, 2005, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, repairs, replacements, evaluations and corrective measures described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Billy H. Na Commissions KS#586 A, B, N, I
Inspector's Signature National Board, State, Province, and Endorsements

Date 11/1/05

This form (E00127) may be obtained from the Order Dept., ASME, 22 Law Drive, Box 2300, Fairfield, NJ 07007-2300

TABLE 1
ABSTRACT OF EXAMINATIONS AND TESTS
Completion of RF-14 (I-2, P-3)

Examination Category	Total Examinations Required For The Interval	Total Examinations Credited for This Period*	Total Examinations Credited (%) for The Period**	Total Examinations Credited (%) to Date for the Interval	Remarks
B-A	25	18	72	92	Note 10, 15
B-B	5	2	40	100	
B-D	36	11	31	81	Notes 11, 14
B-E	113	38	34	100	
B-F	8	0	0	100	Note 9
B-G-1	219	74	34	100	
B-G-2	15	6	40	100	Note 1
B-H	2	2	100	100	Note 12
B-J	45	0	0	100	Note 9
B-K	-	-	-	-	
B-L-2	1	0	0	100	Note 2
B-M-2	7	0	0	Note 3	Note 3
B-N-1	3	1	33	100	
B-N-2	6	6	100	100	
B-N-3	1	1	100	100	
B-O	4	0	0	100	
B-P	Note 4	Note 4	Note 4	Note 4	Note 4
C-A	9	1	11	89	Note 16
C-B	7	2	29	100	
C-C	19	7	37	100	
C-D	1	0	0	100	
C-F-1	26	0	0	100	Notes 5, 9
C-F-2	16	0	0	100	Note 9
C-G	5	3	60	100	
C-H	Note 13	Note 13	Note 13	Note 13	Note 13
AUG	325	9	3	100	Note 6
D-A	44	15	34	100	Note 7
D-B	-	-	-	-	Note 7
F-A	282	96	34	100	Note 8
RI-ISI	82	41	50	85	Note 9, 17

* This column is interpreted to represent the cumulative number of exams performed to date in this period.

** This column is interpreted to represent the cumulative percentage of scheduled exams for the interval which have completed in this period to date.

- Note 1: For pumps and valves, examinations are limited to components selected for examination under Examination Categories B-L-2 and B-M-2.
- Note 2: Examination is required only when a pump is disassembled for maintenance, repair or examination. Examination is required only once during the inspection interval, thus completed percentages indicated may not be consistent with Table IWB-2412-1 schedule. In RFO-9, RCP-D was disassembled, and the pump casing surfaces were examined.
- Note 3: Examination is required only when a valve is disassembled for maintenance, repair or examination. Examination is required only once during the inspection interval, thus completed percentages indicated may not be consistent with Table IWB-2412-1 schedule. There were 2 valve groups (out of the 7 groups in the B-M-2 Category) that did not have any valve in the group disassembled, resulting in 5 of the 7 examinations for the Interval being performed.
- Note 4: Visual Examinations (VT-2) are performed as required for a fuel cycle and a 10-year system hydrostatic testing basis. All required VT-2 exams have been performed for the time frame covered by this report.
- Note 5: For simplicity, numbers and percentages are based only on circumferential welds selected for examination. When a circumferential weld is intersected by a longitudinal weld, that longitudinal weld is selected by default in the WCNOG Program. This approach results in greater than the 7.5% sample of longitudinal seam welds being inspected, exceeding the ASME Code requirements.
- Note 6: Includes augmented exams required by NUREG 0800 and RCP Flywheel Exams. In Period 3, WCNOG implemented a risk informed program for the examination required by NUREG 0800 (the high energy line break welds). In Period 3, only the welds selected for examination in this program (that had not been previously examined in the interval) were scheduled for examination. In the column of "Total Examination Required for the Interval", the number shown includes the number of examinations completed prior to implementation of the program, plus the number of selected welds to be examined by the risk informed program to complete 100% examination of the risk informed program selected welds. WCNOG requested and received approval of a license amendment to extend the examination interval of the RCP flywheels to 20 years (this to allow the flywheels to be examined during motor refurbishment). Thus, only 2 flywheel exams are credited to Interval 2.
- Note 7: Numbers and percentages do not include Visual Examinations (VT-2) that are performed as required for 40-month and 10-year system hydrostatic testing basis. All required exams have been performed for this fuel cycle and period.
- Note 8: Wolf Creek has implemented changes that has removed many snubbers and associated supports from plant configuration. The sample percentage requirements of Code Case N-491-2 continue to be met.
- Note 9: Categories B-F, B-J, C-F-1, and C-F-2 welds are now examined under the RI-ISI program. In the safety evaluation of the WCNOG RI-ISI program, the NRC approved WCNOG's proposal that since 33 percent of the ASME Section XI exams had been performed in Period 1 and the first part of Period 2, then 67 percent of the RI-ISI scope would be examined during the remainder of Period 2 and Period 3, so that 100% of selected exams are completed during the interval. In the column of "Total Examination Required for the Interval", the number shown reflects the number of examinations performed on the B-F, B-J, C-F-1, and C-F-2 categories prior to implementation of the RI-ISI program and the number of RI-ISI welds to be examined in the remainder of Period 2 and Period 3.
- Note 10: On the OAR-1 forms completed for Periods 1 and 2, the percentages were calculated as if the RV flange to shell weld was completely examined (both from the flange face and the shell) in Period 1. Only the examination from flange face should have been credited to the Period 1 examination. The total examination percent complete for the interval is now calculated with this correction. This has been captured in the Wolf Creek corrective action program by PIR 2005-0405. The examination of the RV flange to shell weld from the shell was completed in RF14.

- Note 11: On the OAR-1 forms completed for Periods 1 and 2, the percentages were calculated as if the RV outlet nozzle to shell welds were completely examined (both from the shell and bore) in Period 1. Only the examination from the bore and the inner radius exam should have been credited to the Period 1 examinations. The total examination percent complete for the interval is now calculated with this correction. This has been captured in the Wolf Creek corrective action program by PIR 2005-0405. The examination of the nozzle to shell welds from the shell were completed in RF14.
- Note 12: The examination of the pressurizer skirt weld in Period 1 was examined from the OD only. The percentages on OAR-1 forms completed for Periods 1 and 2 were calculated as if this examination was complete. WCNOCC corrective action document PIR 2004-3194 details that pressurizer skirt weld should have been completed from both the OD and the ID in Period 1 prior to claiming for credit for examination of this component. The examination of the ID portion of this component was completed in RF14.
- Note 13: All Category C-H components have a VT-2 examination performed as a minimum of once per Period. For Period 3, there are 51 surveillance procedures (STS) which will complete the examination requirements for Category C-H. All of the STS procedures have been completed.
- Note 14: Seven of the Category B-D components had examinations performed that did not achieve greater than 90% total coverage of the code required volume. Relief will be requested and approval from the NRC obtained prior to claiming credit for the examination of these components. It should be noted that five of these components had previous NRC approval for limited examination (Relief Requests I2R-07 and I2R-32) based on the examinations performed in Interval 1. However; the results of the examinations in Interval 2 is that the basis of the limitations of the examination are not the same as those shown on the previously approved request for relief, thus new requests for relief for these limited exams, showing the actual limitations, should be submitted to NRC. The Wolf Creek administrative procedure requires that the examinations are to be considered incomplete until the NRC approval of a relief request is obtained. The relief requests will be submitted to the NRC, and approval obtained prior to claiming credit for these examinations. After credit is taken for these 7 examinations, all examinations required for the Interval will be complete for this Category.
- Note 15: On two of the examinations of the B-A components, the coverage obtained was not greater than 90%. The Wolf Creek administrative procedure requires that the examinations are to be considered incomplete until the NRC approval of a relief request is obtained. The relief requests will be submitted to the NRC, and approval obtained prior to claiming credit for these examinations. After credit is taken for these 2 examinations, all examinations required for the Interval will be complete for this Category.
- Note 16: On one of the examinations of the C-A components, the coverage obtained was not greater than 90%. The Wolf Creek administrative procedure requires that the examinations are to be considered incomplete until the NRC approval of a relief request is obtained. The relief request will be submitted to the NRC, and approval obtained prior to claiming credit for this examination. After credit is taken for this examination, all examinations required for the Interval will be complete for this Category.
- Note 17: Of the examinations on the 82 RI-ISI components, the coverage on 12 of the components did not meet the requirements of Appendix VIII. Due to the configuration of these components, it is not possible to claim credit for greater than 90% coverage per the Appendix VIII procedure. The Wolf Creek administrative procedure requires that the examinations are to be considered incomplete until the NRC approval of a relief request is obtained. The relief requests will be submitted to the NRC, and approval obtained prior to claiming credit for these examinations. After credit is taken for these 12 examinations, all examinations required for the Interval will be complete for this Category.

TABLE 2
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT
REQUIRED EVALUATION FOR CONTINUED SERVICE

Examination Category	Item Number	Item Description	Flaw Characterization (IWA-3300)	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes or No)
D-A	D.110	Piping EF-054-HBC-8	Through Wall Leak Note A	No

Note A: On 4/6/05, a through wall leak was discovered in a section of piping in the line EF054HBC-8. The engineering evaluation performed in accordance with Code Case N-513 determined that the component was suitable for continued service (reference Operability Evaluation OE EF-05-002). This evaluation directed that the component be monitored and restored during the next refueling outage (RF14). This component was replaced by Repair/Replacement Plan 2005-020 during RF14.

TABLE 3
ABSTRACT OF REPAIRS, REPLACEMENTS, OR CORRECTIVE MEASURES
REQUIRED FOR CONTINUED SERVICE

Code Class	Repair, Replacement, or Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes/No)	Date Complete	Repair/ Replacement Plan Number
3	Replacement	Containment Air Cooler, SGN01D	Remove one leaking cooling coil from service by cutting off nozzles and blind flanging the supply and return header connections per DCP 07111.	No	06/08/2004	2004-005
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGL15A	Replace one leaking H-bend connecting Row 31 Tube 3 and Row 30 Tube 3 and replace one leaking tube nut adapter at Row 31 Tube 2.	No	06/08/2004	2004-008
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGL10A	Cut tubes at Row 43 Tube 1 and Row 36 Tube 4 and install caps on header side of cut to remove one leaking tube circuit from service.	No	06/08/2004	2004-009
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGL09A	Remove H-bends connecting Row 25 Tube 2 to Row 24 Tube 2 and Row 27 Tube 1 to Row 26 Tube 1 and install a jumper from Row 27 Tube 1 to Row 24 Tube 2 due to a leaking fitting and Tube in Row 25 Tube 2.	No	07/01/2004	2004-019
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGL12A	Replace H-bend connecting Row 33 Tube 3 And Row 32 Tube 3 and a tube nut adapter at Row 1 Tube 4 due to leakage.	No	10/04/2005	2004-050
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGL15A	Replace leaking H-bend which connects Row 9 Tube 4 to Row 8 Tube 4.	No	10/04/2005	2004-053
3	Repair	Standby Diesel Generator "A" Jacket Water Heat Exchanger	Repair by welding a pit in the existing weld on the slip-on flange (inside fillet weld) on the east end of EKJ06A	No	03/08/2005	2005-007
3	Replacement	Daniel International, Bergen-Paterson, Pacific Scientific Snubber Pipe Supports, EG03-R013/141, EG03-R014/141	Replace snubbers on pipe support per CCP# 011739 due to functional failure per STS MT-027.	No	10/12/2005	2005-018
3	Repair/ Replacement	8" Pullman Power Products Pipe Spool, EF06-S044/125	Replace a portion of spool EF06-S044/125 due to a through wall leak.	No	05/25/2005	2005-020
3	Repair/ Replacement	¾" Pipe Spool, EG-HBC-3/4", Upstream of EGV0028	Replace line EG-HBC-3/4", upstream of EGV0028 due to a leak in the pipe to half coupling weld.	No	05/25/2005	2005-021

1	Repair/ Replacement	Westinghouse Steam Generator Model "F", EBB01C	Contract Repair Organization PCI replaced the 3/8" boss in the bottom of EBB01C per PCI Travelers PI-900568-01 SGC and PI-900568-02 SGC, PO 730466, CCP 11743 and the approved PCI QA Program	No	10/04/2005	2005-032
1	Repair/ Replacement	Westinghouse Steam Generator Model "F", EBB01D	Contract Repair Organization PCI replaced the 3/8" boss in the bottom of EBB01D per PCI Travelers PI-900568-01 SGD and PI-900568-02 SGD, PO 730466, CCP 11743 and the approved PCI QA Program	No	10/04/2005	2005-033
3	Replacement	Daniel International, Bergen-Paterson, Pacific Scientific Pipe Support, EF03-RO16/142	Replace snubber S/N 10376 with S/N 4940 due to failure of the hand stroke test.	No	05/13/2005	2005-039
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGF02A	Replace leaking H-bend assembly which connects Row 1 Tube 2 to Row 1 Tube 3.	No	09/27/2005	2005-040
1	Replacement	Westinghouse Steam Generator Model "F", EBB01C	Replace Steam Generator Hot Leg Primary Manway Stud, CH4, with new material due to failure to pass a VT-1 Exam.	No	09/28/2005	2005-041
3	Replacement	Daniel International, Bergen Paterson, Pacific Scientific Snubber Pipe Support, EG04-R011/131	Replace snubber on pipe support EG04R011131 in accordance with CCP# 011772 due to functional test failure.	No	10/12/2005	2005-042
1	Replacement	Daniel International, Bergen Paterson Pipe Support, BG22-H009/232	Replaced a Bergen Paterson sway strut which had a bent paddle	No	05/24/2005	2005-043
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGL09A	Cut tubing at Row 34 Tube 4 and Row 41 Tube 1 near the headers, replace the remaining tube stub at Row 34 Tube 4 due to leakage at the tube to header joint and cap the tube stubs for Row 34 Tube 4 and Row 41 Tube 1 at the supply and return headers to remove that circuit from service.	No	09/28/2005	2005-045
3	Replacement	Carrier Type 28NW Room Cooler Coil, SGL10A	Removed 7 tubes identified by ECT as having excessive wall thinning in an attempt to eliminate one leaking tube. Additional tubes were removed from service as needed to cap circuits and jumper over the worn tubes. Circuit containing Row29 Tube 1 and Row 22 Tube 4 was cut and capped near the headers and jumpers were installed from Row 39 Tube 1 to Row 36 Tube 2, Row 23 Tube 2 to Row 20 Tube 3, Row 19 Tube 1 to Row 14 Tube 3 and Row 7 Tube 2 to Row 4 Tube 3.	No	09/28/2005	2005-048