

November 16, 2005

Mr. R. T. Ridenoure
Vice President - Chief Nuclear Officer
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - REQUEST FOR ADDITIONAL
INFORMATION RELATED TO TECHNICAL SPECIFICATIONS FOR STEAM
GENERATOR BLOWDOWN ISOLATION (TAC NO. MC7524)

Dear Mr. Ridenoure:

By letter dated July 1, 2005 (Agencywide Documents Access and Management System Accession No. ML051950401), Omaha Public Power District (OPPD/the licensee) requested a license amendment for changing the Updated Safety Analyses Report to credit operator action to isolate steam generator blowdown during a loss-of-main feedwater event. As a result of our review of OPPD's submittal, we have determined that additional information is needed to complete our review. The Nuclear Regulatory Commission staff's request for additional information (RAI) related to OPPD's request is attached. This request was discussed with Thomas Matthews of your staff and it was agreed that a response would be provided within 30 days of receipt of this letter.

Fort Calhoun has been selected to be part of the RAI Data Collection-Pilot Effort. As discussed previously, the RAIs have been assigned a category to be used in our pilot effort. If you have any questions or comments on the RAIs or their categorization please contact me at 301-415-1445.

Sincerely,

/RA

Alan B. Wang, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure: Request for Additional Information

cc w/encl: See next page

November 16, 2005

Mr. R. T. Ridenoure
Vice President - Chief Nuclear Officer
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - REQUEST FOR ADDITIONAL
INFORMATION RELATED TO TECHNICAL SPECIFICATIONS FOR STEAM
GENERATOR BLOWDOWN ISOLATION (TAC NO. MC7524)

Dear Mr. Ridenoure:

By letter dated July 1, 2005 (Agencywide Documents Access and Management System Accession No. ML051950401), Omaha Public Power District (OPPD/the licensee) requested a license amendment for changing the Updated Safety Analyses Report to credit operator action to isolate steam generator blowdown during a loss-of-main feedwater event. As a result of our review of OPPD's submittal, we have determined that additional information is needed to complete our review. The Nuclear Regulatory Commission staff's request for additional information (RAI) related to OPPD's request is attached. This request was discussed with Thomas Matthews of your staff and it was agreed that a response would be provided within 30 days of receipt of this letter.

Fort Calhoun has been selected to be part of the RAI Data Collection-Pilot Effort. As discussed previously, the RAIs have been assigned a category to be used in our pilot effort. If you have any questions or comments on the RAIs or their categorization please contact me at 301-415-1445.

Sincerely,

/RA

Alan B. Wang, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure: Request for Additional Information

cc w/encl: See next page

DISTRIBUTION:

PUBLIC

LPLIV Reading

RidsNrrDorlLpliv (DTerao)

RidsNrrLALFeizollahi

RidsNrrPMAWang

RidsOgcRp

RidsAcrsAcnwMailCenter

RidsRegion4MailCenter (DGraves)

RidsNrrDorlDpr

ADAMS ACCESSION NO.: ML053050279

OFFICE	LPLIV/PM	LPLIV/LA	LPLIV/BC
NAME	AWang	LFeizollahi	DTerao
DATE	11/14/05	11/10/05	11/16/05

OFFICIAL RECORD COPY

REQUEST FOR ADDITIONAL INFORMATION
TECHNICAL SPECIFICATION RELATED TO
STEAM GENERATOR BLOWDOWN ISOLATION
PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION
OMAHA PUBLIC POWER DISTRICT
FORT CALHOUN STATION, UNIT 1
DOCKET NO. 50-285

By letter dated July 1, 2005, Omaha Public Power District (OPPD) requested a license amendment for changing the Updated Safety Analyses Report (USAR) to credit operator action to isolate steam generator blowdown during a loss-of-main feedwater event (LMFW). As a result of our review of OPPD's submittal, we have determined that the following additional information is needed to complete our review:

(1) Steam Generator (SG) Blowdown Isolation

Confirm that the SG blowdown isolation system is a safety grade system. If it is not a safety grade system, justify that its isolation function can be achieved on demand during the limiting transient for determining the required operator action for SG blowdown isolation. Does this function need to be included as a technical specification surveillance requirement to assure the reliability of the isolation system?

Also, confirm that taking credit of the operator action for SG blowdown isolation will not inadvertently affect the results of any loss-of-coolant accident (LOCA) and non-LOCA transients. (2a)

(2) SG Heat Transfer Model

Discuss the SG heat transfer model in S-RELAP5 for events that experience SG dryout. Provide the values of SG dryout heat transfer coefficients used in the analysis of non-LOCA transients, in particular, the LMFW and feedwater line break (FWLB) events.

The semiscale test data for FWLB (as discussed in Section 4.3.3.1 of NUREG/CR-4945, dated July 1987) demonstrate that SG heat transfer capacity remains unchanged until the SG liquid inventory is nearly deleted. This is followed by a rapid reduction to zero percent heat transfer with little further reduction in the SG water inventory. In light of these test data, OPPD is requested to verify that the heat transfer model used in the LMFW and FWLB events (that experience SG dryout) is conservative as compared to the semiscale test data. In addition, please provide:

- a) a sketch of the four vertically stacked control volumes showing the volumes from top to bottom and associated void fractions,
- b) combine Figures 4 and 5 so that the conservatism can be readily apparent, and
- c) a justification for why S-RELAP5 is conservative with respect to semi-scale tests for FWLBs from 14 to 40 percent. (2b)

(3) Initial Conditions

Discuss the effects of each parameter of the plant's initial conditions listed in Table 1 and 3 of Attachment 3 to the July 1, 2005, letter on the LMFW and FWLB analyses, respectively. Justify that the values of the parameters used in the analysis are conservative in determination of the shortest required operator action time for SG blowdown isolation. The discussion should include all of the parameters of Table 1 (such as initial pressurizer pressure and level, initial SG pressure, and reactor protection system trip on a low SG level, etc.) and justify why the values used in the analysis are conservative with respect to the shortest time required for operator action to isolate blowdown. (2a)

(4) Non-Safety Grade Components and/or Systems

Identify any non-safety grade systems and/or components that were credited for consequence mitigation in the LMFW and FWLB analysis, and justify that the use of identified non-safety systems and/or components for event mitigation is acceptable. Provide examples of non-safety grade systems that exacerbate the consequences of the LMFW and/or FWLB event. (2a)

(5) CESEC and S-RELAP5 Comparison

Identify the differences of SG blowdown isolation models used in CESEC and S-RELAP5. (2b)

Both codes were approved by the Nuclear Regulatory Commission for use in the analysis of non-LOCA transients. Identify the fuel cycle number that started using S-RELAP5 to replace CESEC for the reload analysis. Explain why the need of operation action for SG blowdown was not identified in the current Cycle 23 reload analysis, instead, it was identified for Cycle 24 reload analysis that simulated the replacement SG, pressurizer, and reactor vessel head to be installed in 2006. Are there any effects on the LOCA and non-LOCA transient analyses (in particular, the LMFW and FWLB analysis) caused by the replacement SG, pressurizer, and reactor vessel? (3e)

Provide a statement discussing why Criterion 2 on page 8 of the auxiliary feedwater actuating system setpoint report is acceptable. In particular, justify that the conditions with $T_h < 600^\circ \text{F}$ and reactor coolant system subcooling $\geq 20^\circ \text{F}$ will assure fuel cladding integrity in meeting the plant-specific departure from nucleate boiling safety limit and fuel centerline melting limit. (2b)

RAI CATEGORIES

(Select only one, most dominant category for each RAI question)

1. More information is needed because of:
 - a. complexity of request
 - b. first-of-a-kind nature of request
 - c. NRC change in regulatory significance or focus
 - d. NRC questions on previously used methodology or guidance
 - e. licensee change to previously used methodology
 - f. licensee reduction in current safety margin

2. The review can not be completed without additional explanation or clarification of:
 - a. input variables or analytical assumptions
 - b. methodology used or results obtained
 - c. applicability or bounding nature of third party analyses or data correlations
 - d. differences from NRC guidance documents (SRP, RG, etc.)
 - e. no significant hazards consideration discussion
 - f. environmental considerations discussion
 - g. applicable regulatory requirements discussion
 - h. information that appears to be incorrect and needs to be corrected
 - i. response to previous RAI appears inadequate

3. Reviewer requesting information even though the question is, or the question asks for:
 - a. not directly related to the request
 - b. inconsistent with applicable codes, standards, RGs, or SRP sections
 - c. information accessible from readily available sources and was explicitly referenced
 - d. Information does not appear needed given the precedent cases discussed in the request
 - e. information that is not safety significant or pertinent to the regulatory finding
 - f. information that is known to engineers who work in the general technical area
 - g. going beyond the current licensing basis and doesn't need to be asked
 - h. a formal commitment

4. Other (please specify)

Ft. Calhoun Station, Unit 1

cc:

Winston & Strawn
ATTN: James R. Curtiss, Esq.
1400 L Street, N.W.
Washington, DC 20005-3502

Chairman
Washington County Board of Supervisors
P.O. Box 466
Blair, NE 68008

Mr. John Hanna, Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 310
Fort Calhoun, NE 68023

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-4005

Ms. Sue Semerera, Section Administrator
Nebraska Health and Human Services
Systems
Division of Public Health Assurance
Consumer Services Section
301 Centennial Mall, South
P.O. Box 95007
Lincoln, NE 68509-5007

Mr. David J. Bannister, Manager
Fort Calhoun Station
Omaha Public Power District
Fort Calhoun Station FC-1-1 Plant
P.O. Box 550
Fort Calhoun, NE 68023-0550

Mr. Joe L. McManis
Manager - Nuclear Licensing
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
P.O. Box 550
Fort Calhoun, NE 68023-0550

Mr. Daniel K. McGhee
Bureau of Radiological Health
Iowa Department of Public Health
Lucas State Office Building, 5th Floor
321 East 12th Street
Des Moines, IA 50319

September 2005