October 31, 2005

Mr. David H. Oatley, Acting Chief Nuclear Officer Pacific Gas and Electric Company Diablo Canyon Power Plant P.O. Box 56 Avila Beach, CA 93424

SUBJECT: DIABLO CANYON POWER PLANT, UNIT 1 - PHONE CONFERENCE RE: UPCOMING STEAM GENERATOR TUBE INSERVICE INSPECTION

Dear Mr. Oatley:

Inservice inspection of steam generator (SG) tubes play a vital role in assuring that adequate structural integrity of the tubes is maintained. A phone conference has been scheduled with members of your staff to discuss the results of the SG tube inspections to be conducted during the current Diablo Canyon Power Plant, Unit 1, refueling outage. This phone conference will be scheduled to occur towards the end of the planned SG tube inspection interval, but before the unit exits its refueling outage. Enclosed is a list of discussion points to facilitate this phone conference.

The Nuclear Regulatory Commission (NRC) staff plans to document a brief summary of the conference call as well as any material that you may provide to the NRC staff in support of the call.

Sincerely,

/RA/

Girija S. Shukla, Project Manager, Section 2 Project Directorate IV Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-275

Enclosure: List of Discussion Points

cc w/encl: See next page

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|--|
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| Girija S. Shukla, Project Manager, Section 2 |
| Project Directorate IV |
| Division of Licensing Project Management |
| Office of Nuclear Reactor Regulation |

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NRR-106

STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

PACIFIC GAS AND ELECTRIC COMPANY

DIABLO CANYON POWER PLANT, UNIT 1

DOCKET NO. 50-275

The following discussion points have been prepared to facilitate the phone conference arranged with Pacific Gas & Electric Company to discuss the results of the steam generator (SG) tube inspections to be conducted during the current Diablo Canyon, Unit 1, refueling outage. This phone call is scheduled to occur towards the end of the planned SG tube inspection interval, but before the unit completes its inspections and repairs.

The Nuclear Regulatory Commission staff plans to document a brief summary of the conference call as well as any material that is provided in support of the call.

- 1. Discuss any trends in the amount of primary-to-secondary leakage observed during the recently completed cycle.
- 2. Discuss whether any secondary side pressure tests were performed during the outage and the associated results.
- 3. Discuss any exceptions taken to the industry guidelines.
- 4. For each SG, provide a description of the inspections performed including the areas examined and the probes used (e.g., dents/dings, sleeves, expansion-transition, U-bends with a rotating probe), the scope of the inspection (e.g., 100 percent of dents/dings greater than 5 volts and a 20 percent sample between 2 and 5 volts), and the expansion criteria. Also, discuss the extent of the rotating probe inspections performed in the portion of tube below the expansion transition region (reference NRC Generic Letter 2004-01, "Requirements for Steam Generator Tube Inspections").
- 5. For each area examined (e.g., tube supports, dent/dings, sleeves, etc.), provide a summary of the number of indications identified to-date of each degradation mode (e.g., number of circumferential primary water stress corrosion cracking indications at the expansion transition). For the most significant indications in each area, provide an estimate of the severity of the indication (e.g., provide the voltage, depth, and length of the indication). In particular, address whether tube integrity (structural and accident-induced leakage integrity) was maintained during the previous operating cycle. In addition, discuss whether any location exhibited a degradation mode that had not previously been observed at this location at this unit (e.g., observed circumferential

primary water stress corrosion cracking at the expansion transition for the first time at this unit).

- 6. Describe repair/plugging plans.
- 7. Describe in-situ pressure test and tube pull plans and results (as applicable and if available).
- 8. Provide the schedule for SG-related activities during the remainder of the current outage.
- 9. Discuss the following regarding loose parts:
 - what inspections are performed to detect loose parts?
 - a description of any loose parts detected and their location within the SG
 - if the loose parts were removed from the SG
 - indications of tube damage associated with the loose parts
 - the source or nature of the loose parts, if known

Diablo Canyon Power Plant, Units 1 and 2

cc: NRC Resident Inspector Diablo Canyon Power Plant c/o U.S. Nuclear Regulatory Commission P.O. Box 369 Avila Beach, CA 93424

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